UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 71
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated May 1, 1984 as supplemented and clarified October 22, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 71, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for Robert A. Hermann
Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 1, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 71

FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise the Appendix A Technical Specifications by removing and inserting the following pages:

Existing Page	Revised Page
iiia	iiia
--	241ii4
--	241ii5
--	241ii6
--	241ii7
--	241ii8

The revised areas are indicated by marginal lines.

* to be submitted at a future date

LIMITING CONDITION FOR OPERATION

3.6.13 REMOTE SHUTDOWN PANELS

Applicability:

Applies to the operating status of the remote shutdown panels.

Objective:

To assure the capability of the remote shutdown panels to provide 1) initiation of the emergency condensers independent of the main/auxiliary control room 2) control of the motor-operated steam supply valves independent of the main/auxiliary control room and 3) parameter monitoring outside the control room.

Specification:

a. During power operation and whenever the reactor coolant temperature is greater than 212°F, at least one remote shutdown panel shall be operable.

Amendment No. 71

SURVEILLANCE REQUIREMENT

4.6.13 REMOTE SHUTDOWN PANELS

Applicability:

Applies to the periodic testing requirements for the remote shutdown panels.

Objective:

To assure the capability of the remote shutdown panels to provide 1) initiation of the emergency condensers independent of the main/auxiliary control room 2) control of the motor-operated steam supply valves independent of the main/auxiliary control room and 3) parameter monitoring outside the control room.

Specification:

The remote shutdown panels surveillance shall be performed as indicated below:

a. Each remote shutdown panel monitoring instrumentation channel shall be demonstrated operable by performance of the operations and frequencies shown in Table 4.6.13-1.

b. During each major refueling outage

1. Each remote shutdown panel shall be demonstrated to initiate the emergency condensers independent of the main/auxiliary control room.

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LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT
3.6.13 REMOTE SHUTDOWN PANELS (Continued)	4.6.13 REMOTE SHUTDOWN PANELS (Continued)
b. A remote shutdown panel shall be considered inoperable if either the emergency condenser condensate return valve control switch is inoperable, either motor-operated steam supply valve control switch is inoperable, or the number of operable instrumentation channels is less than that required by Table 3.6.13-1.	2. Each remote shutdown panel shall be demonstrated to open both the motor-operated steam valves.
c. If Specification 3.6.13.a cannot be met, commence an orderly shutdown within 24 hours and be in cold shutdown within 36 hours.	

TABLE 3.6-13-1

REMOTE SHUTDOWN PANEL MONITORING

Limiting Condition for Operation

INSTRUMENT	MINIMUM NUMBER OF OPERABLE CHANNELS
Reactor Pressure	1
Reactor Water Level	1
Reactor Water Temperature	1
Torus Water Temperature	1
Drywell Pressure	1
Emergency Condenser Water Level	1
Drywell Temperature	1
"All Rods In" Light	1

TABLE 4.6.13-1

REMOTE SHUTDOWN PANEL MONITORING

Surveillance Requirement

Parameter	Sensor Check	Instrument Channel Calibration
Reactor Pressure	Once per day	Once per 3 months (a)
Reactor Water Level	Once per day	Once per 3 months (a)
Reactor Water Temperature	Once per day	Once per refueling cycle
Torus Water Temperature	Once per day	Once per refueling cycle
Drywell Pressure	Once per day	Once per 3 months (a)
Emergency Condenser Water Level	Once per day	Once per refueling cycle
Drywell Temperature	Once per day	Once per refueling cycle
"All Rods In" Light	Once per refueling cycle	N/A

(a) The indicator located at the remote shutdown panel will be calibrated at the frequency listed in Table 4.6.13-1. Calibration of the remaining channel instrumentation is provided by Specification 4.6.2.

BASES FOR 3.6.13 AND 4.6.13 REMOTE SHUTDOWN PANELS

The remote shutdown panels provide 1) manual initiation of the emergency condensers 2) manual control of the steam supply valves and 3) parameters monitoring independent of the main/auxiliary control room. Two panels are provided, each located in a separate fire area, for added redundancy. Both panels are also in separate fire areas from the main/auxiliary control room. One remote shutdown panel provides the necessary capabilities consistent with 10CFR50 Appendix R. Therefore, only one remote shutdown panel is required to be operable. The electrical design of the panels is such that no single fire can cause loss of both emergency condensers.

Each remote shutdown panel is provided with controls for one emergency condenser loop. The emergency condensers are designed such that automatic initiation is independently assured in the event of a fire 1) in the Reactor Building (principle relay logic located in the auxiliary control room or 2) in the main/auxiliary control room or Turbine Building (redundant relay logic located in the Reactor Building). Each remote shutdown panel also has controls to operate the two motor-operated steam supply valves on its respective emergency condenser loop. A key operated bypass switch is provided to override the automatic isolation signal to these valves. Once the bypass switch is activated, the steam supply valves can be manually controlled from the remote shutdown panels. Since automatic initiation of the emergency condenser is assured, the remote shutdown panels serve as additional manual controlling stations for the emergency condensers. In addition, certain parameters are monitored at each remote shutdown panel.

The remote shutdown panels are normally de-energized, except for the monitoring instrumentation, which is normally energized. To energize the remaining functions on a remote shutdown panel, a power switch located on each panel must be activated. Once the panels are completely energized, the emergency condenser condensate return valve and steam supply valve controls can be utilized.