



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 72  
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated June 29, 1984 as supplemented and clarified December 3, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

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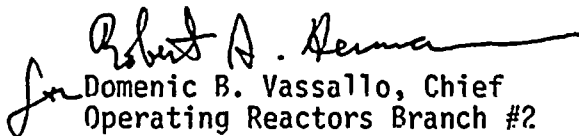


(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 72, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 1, 1985



ATTACHMENT TO LICENSE AMENDMENT NO. 72

FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise the Appendix A Technical Specifications by removing and inserting the following pages:

<u>Existing Page</u>	<u>Revised Page</u>
47	47
47a	-
48	48
50	50
241ee	241ee
241ff	241ff
241gg	241gg
--	241gg1
241hh	241hh
241ii	241ii

The revised areas are indicated by marginal lines.



## LIMITING CONDITION FOR OPERATION

### 3.1.3 EMERGENCY COOLING SYSTEM

#### Applicability:

Applies to the operating status of the emergency cooling system.

#### Objective:

To assure the capability of the emergency cooling system to cool the reactor coolant in the event the normal reactor heat sink is not available.

#### Specification:

- a. During power operating conditions and whenever the reactor coolant temperature is greater than 212F, both emergency cooling systems shall be operable except as specified in 3.1.3.b and c.
- b. If one emergency cooling system becomes inoperable, Specification 3.1.3.a shall be considered fulfilled, provided that the inoperable system is returned to an operable condition within 7 days and the additional surveillance required is performed.

## SURVEILLANCE REQUIREMENT

### 4.1.3 EMERGENCY COOLING SYSTEM

#### Applicability:

Applies to periodic testing requirements for the emergency cooling system.

#### Objective:

To assure the capability of the emergency cooling system for cooling of the reactor coolant.

#### Specification:

The emergency cooling system surveillance shall be performed as indicated below:

- a. At least once every five years -  
The system heat removal capability shall be determined.
- b. At least once daily -  
The shell side water level and makeup tank water level shall be checked.
- c. At least once per month -  
The makeup tank level control valve shall be manually opened and closed.





## LIMITING CONDITION FOR OPERATION

- c. Make up water shall be available from the two gravity feed makeup water tanks.
- d. During Power Operating Conditions, each emergency cooling system high point vent to torus shall be operable.
  - 1. With a vent path for one emergency cooling system inoperable, restore the vent path to an operable condition within 30 days.
  - 2. With vent paths for both emergency cooling systems inoperable, restore one vent path to an operable condition within 14 days and both vent paths within 30 days.
- e. If Specification 3.1.3.a, b, c or d are not met, a normal orderly shutdown shall be initiated within one hour and the reactor shall be in the cold shutdown condition within ten hours.

## SURVEILLANCE REQUIREMENT

- d. At least once each shift -  
The area temperature shall be checked.
- e. During each major refueling outage -  
Automatic actuation and functional system testing shall be performed during each major refueling outage and whenever major repairs are completed on the system.  
  
Each emergency cooling vent path shall be demonstrated operable by cycling each power-operated valve (05-01R, 05-11, 05-12, 05-04R, 05-05 and 05-07) in the vent path through one complete cycle of full travel and verifying that all manual valves are in the open position.
- f. Surveillance with an Inoperable System  
  
When one of the emergency cooling systems is inoperable, the level control valve and the motor-operated isolation valve in the operable system shall be demonstrated to be operable immediately and daily thereafter.



### BASES FOR 3.1.3 AND 4.1.3 EMERGENCY COOLING SYSTEM

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Nearly all maintenance can be completed within a few days. Infrequently, however, major maintenance might be required. Replacement of principal system components could necessitate outages of more than 7 days. In spite of the best efforts of the operator to return equipment to service, some maintenance could require up to 6 months.

The system heat removal capability shall be determined at five-year intervals. This is based primarily on the low corrosion characteristics of the stainless steel tubing. During normal plant operation the water level will be observed at least once daily on emergency condensers and makeup water tanks. High and low water level alarms are also provided on the above pieces of equipment. The test frequency selected for level checks and valve operation is to assure the reliability of the system to operate when required.

The emergency cooling system is provided with high point vents to exhaust noncondensable gases that could inhibit natural circulation cooling. Valve redundancy in the vent path serves to minimize the probability of inadvertent or irreversible actuation while ensuring that a single failure of a vent valve, power supply or control system does not prevent isolation of the vent path. The function, capabilities and testing requirements of the emergency cooling vent paths are consistent with the requirements of item II.B.1 of NUREG 0737, "Clarification of TMI Action Plan Requirement," November 1980.



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LIMITING CONDITION FOR OPERATION

SURVEILLANCE REQUIRMENT

3.6.11 ACCIDENT MONITORING INSTRUMENTATION

Applicability:

Applies to the operability of the plant instrumentation that performs an accident monitoring function.

Objective:

To assure high reliability of the accident monitoring instrumentation.

Specification:

- a. During the power operating condition, the accident monitoring instrumentation channels shown in Table 3.6.11-1 shall be operable except as specified in Table 3.6.11-2.

4.6.11 ACCIDENT MONITORING INSTRUMENTATION

Applicability:

Applies to the surveillance of the instrumentation that performs an accident monitoring function.

Objective:

To verify the operability of accident monitoring instrumentation.

Specification:

Instrument channels shall be tested and calibrated at least as frequently as listed in Table 4.6.11.



TABLE 3.6.11-1

ACCIDENT MONITORING INSTRUMENTATION

<u>Parameters</u>	<u>Total Number of Channels</u>	<u>Minimum Number of Operable Channels</u>	<u>Action (See Table 3.6.11-2)</u>
1) Relief Valve Position Indication	2/Valve	1/Valve	1
2) Safety Valve Position Indication	2/Valve	1/Valve	1
3) Reactor Vessel Water Level	2	1	2
4) Drywell Pressure Monitor	2	1	4
5) Suppression Chamber Water Level	2	1	4
6) Containment Hydrogen Monitor	2	1	4
7) Containment High Range Radiation Monitor	2	1	3





TABLE 3.6.11-2

ACCIDENT MONITORING INSTRUMENTATION  
ACTION STATEMENTS

ACTION - 1

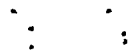
- a. With the number of OPERABLE accident monitoring instrumentation channels 1 less than the total number shown in Table 3.6.11-1, restore to an OPERABLE status during the next cold shutdown when there is access to the drywell.
- b. With the number of OPERABLE accident monitoring instrumentation channels less than the minimum number shown in Table 3.6.11-1, restore the inoperable channel to an OPERABLE status within 30 days or be in at least a HOT SHUTDOWN within the next 12 hours.
- c. The total number of channels shown in Table 3.6.11-1 will be OPERABLE prior to the beginning of each cycle.

ACTION -2

- a. With the number of OPERABLE accident monitoring instrumentation channels less than the total Number of Channels shown in Table 3.6.11-1, restore the inoperable channel(s) to OPERABLE status within seven days or be in at least HOT SHUTDOWN within the next 12 hours.
- b. With the number of OPERABLE accident monitoring instrumentation channels less than the minimum Channels OPERABLE requirements of Table 3.6.11-1, restore the inoperable channel(s) to OPERABLE status within 48 hours or be in at least HOT SHUTDOWN within the next 12 hours.

ACTION - 3

- a. With the number of OPERABLE channels less than the total Number of Channels shown in Table 3-6.11-1, prepare and submit a Special Report to the Commission within 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.
- b. With the number of OPERABLE channels less than required by the minimum channels OPERABLE requirements, initiate the pre-planned alternate method of monitoring the appropriate parameter(s) within 72 hours, and :
  - 1) either restore the inoperable channel(s) to OPERABLE status within seven days of the event, or
  - 2) prepare and submit a Special Report to the Commission within 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.



ACTION - 4

- a. With the number of OPERABLE channels less than the total Number of Channels shown in Table 3-6.11-1, prepare and submit a Special Report to the Commission within 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.
- b. With the number of OPERABLE channels less than required by the minimum channels OPERABLE requirements, initiate the pre-planned alternate method of monitoring the appropriate parameter(s) within 72 hours, and:
  - 1) either restore the inoperable channel(s) to OPERABLE status within seven days of the event, or
  - 2) prepare and submit a Special Report to the Commission within 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE system.
- c. If the pre-planned alternate method of monitoring the appropriate parameter(s) is not available, either restore the inoperable channel(s) to OPERABLE status within seven days or be in at least HOT SHUTDOWN within the next 12 hours.



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Table 4.6.11

Accident Monitoring InstrumentationSurveillance Requirement

<u>Parameter</u>	<u>Instrument Channel Test</u>	<u>Instrument Channel Calibration</u>
(1) Relief valve position indicator (Primary - Acoustic)	Once per month	Once during each major refueling outage
Relief valve position indicator (Backup - Thermocouple)	Once per month	Once during each major refueling outage
(2) Safety valve position indicator (Primary - Acoustic)	Once per month	Once during each major refueling outage
Safety valve position indicator (Backup - Thermocouple)	Once per month	Once during each major refueling outage
(3) Reactor vessel water level	Once per month	Once during each major refueling outage
(4) Drywell Pressure Monitor	Once per month	Once during each major refueling outage
(5) Suppression Chamber Water Level Monitor	Once per month	Once during each major refueling outage
(6) Containment Hydrogen Monitor	Once per month	Once per quarter
(7) Containment High Range Radiation Monitor	Once per month	Once during each major refueling outage



### BASES 3.6.11 AND 4.6.11 ACCIDENT MONITORING INSTRUMENTATION

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Accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. This capability is consistent with the recommendations of NUREG-0578, "TMI-2 Lessons Learned Task Force Status Report and Short-Term Recommendations" and/or NUREG-0737, "Clarification of TMI Action Plan Requirements," November 1980.

