



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 70 TO FACILITY OPERATING LICENSE NO. DPR-63
NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT NUCLEAR STATION, UNIT NO. 1
DOCKET NO. 50-220

1.0 Introduction

By application dated July 11, 1984 and clarified by letter dated October 24, 1984, Niagara Mohawk Power Corporation (the licensee) requested an amendment to Appendix A of Facility Operating License No. DPR-63 for Nine Mile Point Nuclear Power Station, Unit No. 1. The proposed amendment would modify the definition section, the limiting conditions for operations, surveillance requirements and bases section of the Technical Specifications with regard to the reactor coolant leakage limits. More specifically, initiation of inspection and corrective actions are required when identified leakage increases at a rate of two gallons per minute within a twenty-four hour period or less; the frequency of reactor coolant leakage checks is increased; and operability and surveillance requirements are imposed on the leakage detection systems.

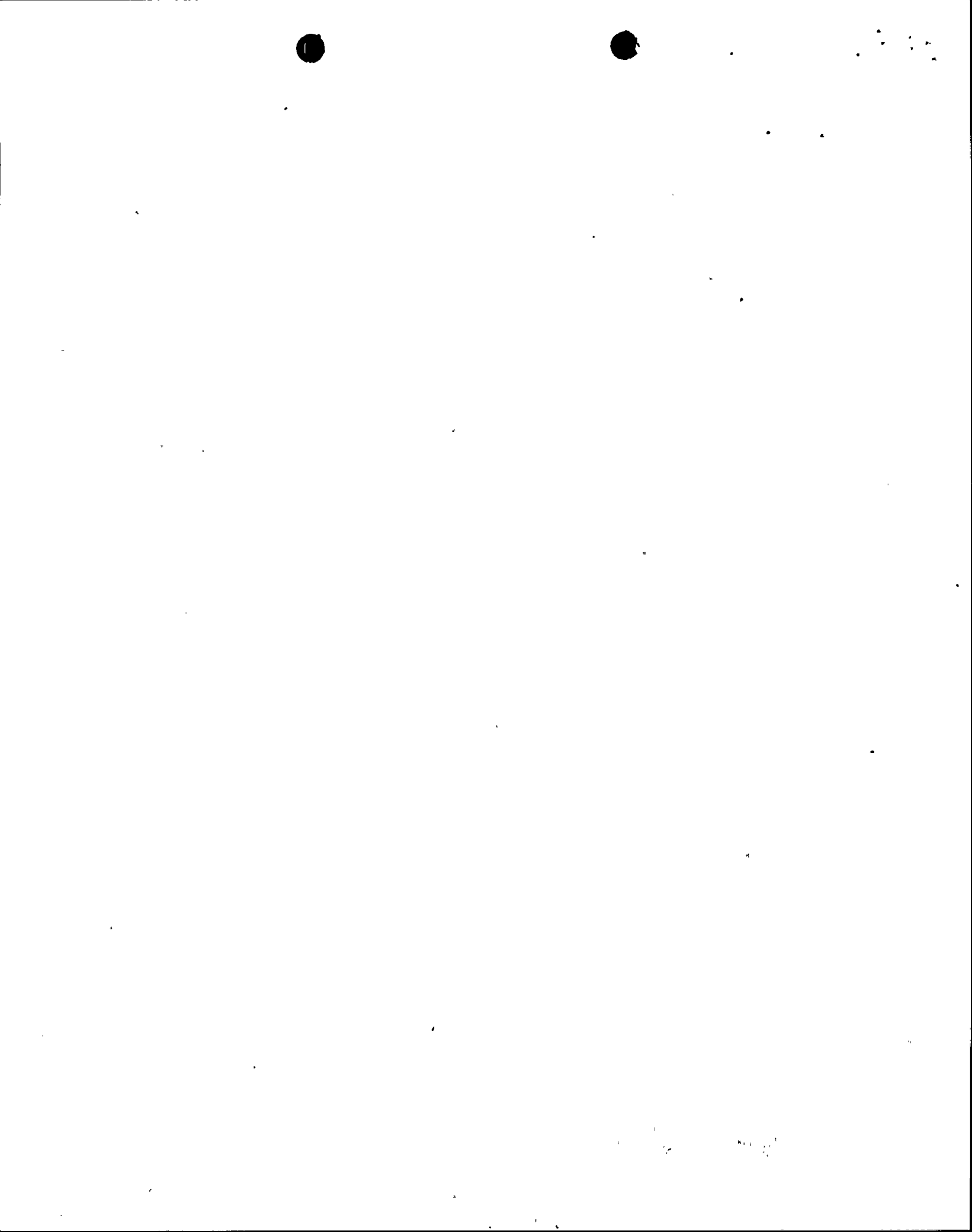
2.0 Evaluation

By letter dated June 6, 1984 the staff issued a safety evaluation of the implementation of NUREG-0313, Rev. 1 for Nine Mile Point Unit 1. At that time the staff requested the licensee to submit a Technical Specification change to incorporate the leakage detection requirements contained in Generic Letter 84-11. Although the licensee replaced the recirculation system and most branch piping, the change was requested since some service sensitive piping remains within the primary containment and cracking had been discovered in the reactor vessel stub tubes. The licensee requested the change as stated above.

The information provided by the licensee in their submittal of October 24, 1984 was clarifying in nature and did not provide substantive change to the amendment request as described in the Federal Register notice.

We have reviewed the changes proposed by the licensee and find they are consistent with the guidance contained in Attachment 1 - Leakage Detection and Leakage Limits of Generic Letter 84-11, Inspection of BWR Stainless Steel Piping dated April 19, 1984. Therefore, we find the requested change acceptable.

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3.0 Environmental Considerations

This amendment involves a change in the installation or use of a facility component located within the restricted area and a change in a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 Conclusion

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Hermann

Dated: February 27, 1985

