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 FACIL: 50-220 Nine Mile Point Nuclear Station, Unit 1, Niagara Powe    05000220  
 AUTH. NAME    AUTHOR AFFILIATION  
 MANGAN, C.V.    Niagara Mohawk Power Corp.  
 RECIP. NAME    RECIPIENT AFFILIATION  
 VASSALLO, D.B.    Operating Reactors Branch 2

SUBJECT: Forwards summary of changes made to fire brigade training program. Fire brigade training condensed steam & mechanical fundamental course from 80 h to 40 h.

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THE UNITED STATES OF AMERICA  
 DISTRICT COURT OF THE DISTRICT OF COLUMBIA  
 IN RE: [Name], Debtor.  
 Chapter 11, Title 11, U.S.C.

The undersigned, [Name], being duly sworn, deposes and says that the above-named debtor is a resident of the District of Columbia, and that the above-named debtor is a citizen of the United States of America.

Executed on this [Date] day of [Month], 19[Year].  
 [Signature]  
 [Name], Clerk of the Court.

Case No.	Case Name	Case No.	Case Name
1	AMERICAN	1	AMERICAN
2	AMERICAN	2	AMERICAN
3	AMERICAN	3	AMERICAN
4	AMERICAN	4	AMERICAN
5	AMERICAN	5	AMERICAN
6	AMERICAN	6	AMERICAN
7	AMERICAN	7	AMERICAN
8	AMERICAN	8	AMERICAN
9	AMERICAN	9	AMERICAN
10	AMERICAN	10	AMERICAN

September 4, 1984

Director of Nuclear Reactor Regulation  
Attention: Mr. Domenic B. Vassallo, Chief  
Operating Reactors Branch No. 2  
Division of Licensing  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Re: Nine Mile Point Unit 1  
Docket No. 50-220  
DPR-63

Dear Mr. Vassallo:

During a telephone conversation on June 25, 1982, members of your staff indicated the approach to fire brigade training presented by Niagara Mohawk would meet the intent of 10CFR50 Appendix R regarding training of the fire brigade. A portion of this approach was based on an 80-hour course on steam and mechanical fundamentals, including engineered safeguard systems, auxiliary plant systems and electrical distribution. Recently, Niagara Mohawk has condensed this steam and mechanical fundamentals course into a 40 hour course. As a result, the duration of the initial fire brigade training has been changed, although the material content remains the same. The purpose of this letter is to summarize the changes we have made to the fire brigade training program.

Initial training that new members of the fire brigade receive is currently divided into two cycles, an outline of which is provided in Attachment 1. Cycle I is the revised steam and mechanical fundamentals course. At the conclusion of Cycle I, the individuals will have a general understanding of Boiling Water Reactor operation and the importance of nuclear safety systems. Cycle II is an 80-hour detailed systems description course which will provide the fire department with sufficient training to understand the effects of fire and fire suppressants on safe shutdown capability. The scope of the Cycle II training is equivalent to that given to a reactor operator and will be given

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to the fire brigade members on an annual basis. The fire brigade training still includes the general employee training (i.e. radiation protection, quality assurance, administrative procedure indoctrination) and the required fire training (i.e. strategy, tactic, fire systems, etc.).

Sincerely,

NIAGARA MOHAWK POWER CORPORATION



C. V. Mangan  
Vice President

Nuclear Engineering and Licensing

JTD/djm  
Attachment

THE UNIVERSITY OF CHICAGO  
DEPARTMENT OF CHEMISTRY  
5800 S. DICKINSON DRIVE  
CHICAGO, ILLINOIS 60637

RECEIVED  
JAN 10 1964

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## ATTACHMENT 1

### I. Purpose:

Cycle I - The purpose of Cycle I training is to provide Nine Mile Point Fire Department Personnel with an introduction to Boiling Water Reactor systems operation and to prepare them to receive the in-depth training provided in Cycle II.

Particular emphasis will be placed on nuclear safety systems and electrical distribution, as needed, to ensure a safe shutdown can be achieved from any operating condition.

At the conclusion of Cycle I, the student should have a general understanding of Boiling Water Reactor operation and the importance of nuclear safety systems.

Cycle II - The purpose of Cycle II training is to provide the Fire Chiefs and members of the Nine Mile Point Fire Department with sufficient training to understand the effects of fire and fire suppressants on safe shutdown capability. The scope of that training shall be equivalent to that received by a reactor operator.

### II. Requirements for Class:

Plant specific Nuclear Safety Systems training is conducted in accordance with the requirements of:

10CFR50, Appendix R  
Branch Technical Position - 9.5.1  
Regulatory Guide - 1.120

### III. Prerequisites:

- Instructor - The instructor for this material has a knowledge of the course equivalent to that gained in a reactor operator license class.
- Student - The students for these classes shall be members of the Nine Mile Point Fire Department or other persons designated as having a need for this level of knowledge.

SECRET

The following information was obtained from a review of the files of the [redacted] and is being furnished to you for your information. It is to be used only for the purpose for which it was obtained and is not to be disseminated outside the [redacted] without the express written approval of the [redacted].

The information was obtained from a review of the files of the [redacted] and is being furnished to you for your information. It is to be used only for the purpose for which it was obtained and is not to be disseminated outside the [redacted] without the express written approval of the [redacted].

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ATTACHMENT 1 (Continued)

IV. Lesson Material:

A. Schedule/Outline - Attached

B. Topics Covered:

- Cycle 1 - "Introduction to Operations Technology"
- Chapter 5.0 - The Boiling Water Reactor
    - 5.1 - Overall Description of the Steam to Power Conversion System
    - 5.2 - Basic Steam Loop
    - 5.3 - The Reactor
    - 5.4 - Reactor Recirculation System
    - 5.5 - Main Steam System
    - 5.6 - Main Turbine
    - 5.7 - Condensate and Feedwater Systems
  - Chapter 6.0 - Instrumentation and Control
    - 6.1 - Introduction
    - 6.2 - Basic Instrumentation and Control
    - 6.3 - Reliability Consideration
    - 6.4 - Control of the Reactor and Coolant System
    - 6.5 - Control of the Turbine
    - 6.6 - Protection of the Public
    - 6.7 - Protection of Plant Equipment
  - Chapter 7.0 - Plant Electrical Systems
    - 7.1 - General
    - 7.2 - Station Generating and Distribution System
    - 7.3 - Auxiliary AC Power Distribution
    - 7.4 - Station Auxiliary DC Equipment
  - Chapter 8.0 - Core Standby Cooling Systems
    - 8.1 - General
    - 8.2 - Primary Containment
    - 8.3 - Primary Containment Isolation System
    - 8.4 - Drywell Inerting and Containment Atmospheric Dilution System
    - 8.5 - Secondary Containment
    - 8.6 - Reactor Building Ventilation
    - 8.7 - Emergency Ventilation
    - 8.8 - Emergency Cooling
    - 8.9 - Automatic Depressurization System
    - 8.10 - High Pressure Coolant Injection
    - 8.11 - Core Spray
    - 8.12 - Containment Spray
    - 8.13 - Shutdown Cooling System
    - 8.14 - Head Spray
    - 8.15 - Liquid Poison System

SECTION 1: GENERAL INFORMATION

ITEM 1.01: INTRODUCTION

1.01.01.01: General Information	1.01.01.01.01: General Information	1.01.01.01.01: General Information
1.01.01.02: Project Description	1.01.01.02.01: Project Description	1.01.01.02.01: Project Description
1.01.01.03: Objectives	1.01.01.03.01: Objectives	1.01.01.03.01: Objectives
1.01.01.04: Scope	1.01.01.04.01: Scope	1.01.01.04.01: Scope
1.01.01.05: Stakeholders	1.01.01.05.01: Stakeholders	1.01.01.05.01: Stakeholders
1.01.01.06: Risks	1.01.01.06.01: Risks	1.01.01.06.01: Risks
1.01.01.07: Assumptions	1.01.01.07.01: Assumptions	1.01.01.07.01: Assumptions
1.01.01.08: Deliverables	1.01.01.08.01: Deliverables	1.01.01.08.01: Deliverables
1.01.01.09: Milestones	1.01.01.09.01: Milestones	1.01.01.09.01: Milestones
1.01.01.10: Budget	1.01.01.10.01: Budget	1.01.01.10.01: Budget
1.01.01.11: Resources	1.01.01.11.01: Resources	1.01.01.11.01: Resources
1.01.01.12: Communication	1.01.01.12.01: Communication	1.01.01.12.01: Communication
1.01.01.13: Reporting	1.01.01.13.01: Reporting	1.01.01.13.01: Reporting
1.01.01.14: Review	1.01.01.14.01: Review	1.01.01.14.01: Review
1.01.01.15: Approval	1.01.01.15.01: Approval	1.01.01.15.01: Approval
1.01.01.16: Sign-off	1.01.01.16.01: Sign-off	1.01.01.16.01: Sign-off
1.01.01.17: Final Report	1.01.01.17.01: Final Report	1.01.01.17.01: Final Report
1.01.01.18: Archiving	1.01.01.18.01: Archiving	1.01.01.18.01: Archiving
1.01.01.19: Distribution	1.01.01.19.01: Distribution	1.01.01.19.01: Distribution
1.01.01.20: Summary	1.01.01.20.01: Summary	1.01.01.20.01: Summary

ATTACHMENT 1 (Continued)

IV. Lesson Material: (Continued)

B. Topics Covered: (Continued)

Cycle	II - Operations Technology		
Module	IV	Section 1	- Primary Containment System
		Section 2	- Primary Containment Isolation System
		Section 3	- Drywell Inerting and CAD
		Section 4	- Secondary Containment
		Section 5	- Reactor Building Ventilation
		Section 6	- Emergency Ventilation
		Section 7	- Emergency Cooling
		Section 8	- Automatic Depressurization System (ADS)
		Section 9	- High Pressure Coolant Injection (HPCI)
		Section 10	- Core Spray
		Section 11	- Containment Spray
		Section 13	- Shutdown Cooling System
		Section 15	- Liquid Poison System
Module	II	Section 5	- Control Rod Drive Hydraulics
Module	VII	Section 9	- Electrical Distribution
Module	VIII	Section 4	- Fuel Pool Cooling and Cleanup

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