UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 60
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated January 5, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 60, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 8, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 60

FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise the Appendix A Technical Specifications by removing and inserting the following pages:

Existing Page	Revised Page
118	118
205	205
206	206
235	235

The revised areas are indicated by marginal lines.

LIMITING CONDITIONS FOR OPERATION
Table 3.2.7

REACTOR COOLANT SYSTEM ISOLATION VALVES

Line or System	No. of Valves (Each Line)	Location Relative to Primary Containment	Normal Position	Motive Power	Maximum Oper. Time (Sec)	Action on Initiating Signal	Initiating Signal (All Valves Have Remote Manual Backup)
Main Steam	1	Inside	Open	A.I.P.O.*	10	Close	Reactor water level low-low, or main steam line high radiation, or main steam line high flow, or low condenser vacuum, or high temperature in the pipe tunnel
(Two Lines)	1	Outside	Open	A.I.P.O.*	10	Close	
Main Steam Warm-up (Two Lines)	1	Outside	Closed	A.I.P.O.	8	Close	
Main Steam-Emergency Cooling Vents (Two Lines)	2	Outside	Open	A.I.P.O.	5	Close	-
Feedwater	1	Outside	Open	R.H.P.O.*	60	-	-
(Two Lines)	1	Outside	-	Self Act. Ck.	--	-	-
Emergency Cooling							
Steam Leaving Reactor	1	Outside	Open	A.I.P.O.	38	Close	High system flow
(Two Lines)	1	Outside	Open	A.I.P.O.	38	Close	
Condenser Return to Reactor	1	Inside	-	Self Act. Ck.	--	-	
(Two Lines)	1	Outside	Closed	A.I.P.O.	60	Close	
Reactor Cleanup							
Water Leaving Reactor	1	Inside	Open	A.I.P.O.	18	Close	Reactor water level low-low, or high area temperature, liquid poison initiation or high system pressure, or low system flow, or high system temperature
(One Line)	1	Outside	Open	A.I.P.O.	18	Close	
Water Return to Reactor	1	Inside	Open	A.I.P.O.	18	Close	
(One Line)	1	Outside	-	Self Act. Ck.	--	-	
Shutdown Cooling							
Water Leaving Reactor	1	Inside	Closed	A.I.P.O.	40	Close	Reactor water level low-low, or high area temperature
(One Line)	1	Outside	Closed	A.I.P.O.	40	Close	
Water Return to Reactor	1	Inside	Closed	A.I.P.O.	40	Close	
(One Line)	1	Outside	-	Self Act. Ck.	--	-	

Table 3.6.2c

INSTRUMENTATION THAT INITIATES OR ISOLATES EMERGENCY COOLING

Limiting Condition for Operation

Parameter	Minimum No. of Tripped or Operable Trip Systems	Minimum No. of Operable Instrument Channels per Operable Trip Sytem	Set-Point	Reactor Mode Switch Position in Which Function Must Be Operable			
				Shutdown	Refuel	Startup	Run
EMERGENCY COOLING INITIATION							
(1) High-Reactor Pressure	2	2	≤1080 psig	(b)		x	x
(2) Low-Low Reactor Water Level	2	2	≥5 inches (Indicator Scale)	(b)		x	x
EMERGENCY COOLING ISOLATION (for each of two systems)							
(3) High Steam Flow Emergency Cooling System	2	2(a)	.19 psid			x	x

Table 4.6.2c

INSTRUMENTATION THAT INITIATES OR ISOLATES EMERGENCY COOLING

Parameter	Surveillance Requirement		
	Sensor Check	Instrument Channel Test	Instrument Channel Calibration
EMERGENCY COOLING INITIATION			
(1) High Reactor Pressure	None	Once per month(c)	Once per 3 months(c)
(2) Low-Low Reactor Water Level	Once/day	Once per month(c)	Once per 3 months(c)
EMERGENCY COOLING ISOLATION (for each of two systems)			
(3) High Steam Flow Emergency Cooling System	None	Once per 3 months(c)	Once per 3 months(c)

a. The set points included in the tables are those used in the transient analysis and the accident analysis. The high flow set point for the main steam line is 105 psi differential. This represents a flow of approximately 4.4×10^6 lb/hr. The high flow set point for the emergency cooling system supply line is 19 psi differential. This represents a flow of approximately 8.7×10^5 at rated conditions.

Normal background for the main steam line radiation monitors is defined as the radiation level which exists in the vicinity of main steam lines after 1 hour or more of sustained full rated power. The dose rate at the monitor due to activity from the control rod drop accident of Appendix E or from gross failure of one rod with complete fission product release from the rod would exceed the normal background at the monitor. The automatic initiation signals for the emergency cooling systems have to be sustained for more than 10 seconds to cause opening of the return valves. If the signals last for less than 10 seconds, the emergency cooling system operating will not be automatically initiated.

The high level in the scram discharge volume is provided to assure that there is still sufficient free volume in the discharge system to receive the control rod drives discharge. Following a scram, bypassing is permitted to allow draining of the discharge volume and resetting of the reactor protection system relays. Since all control rods are completely inserted following a scram and since the bypass of this particular scram initiates a control rod block, it is permissible to bypass this scram function. The scram trip associated with the shutdown position of the mode switch can be reset after 10 seconds.

The condenser low vacuum, low-low vacuum and the main steam line isolation valve position signals are bypassed in the startup and refuel positions of the reactor mode switch when the reactor pressure is less than 600 psig. These are bypassed to allow warmup of the main steam lines and a heat sink during startup.