

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION SUPPORTING AMENDMENT NO. 56 TO FACILITY OPERATING LICENSE NO. DPR-63

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-220

Introduction

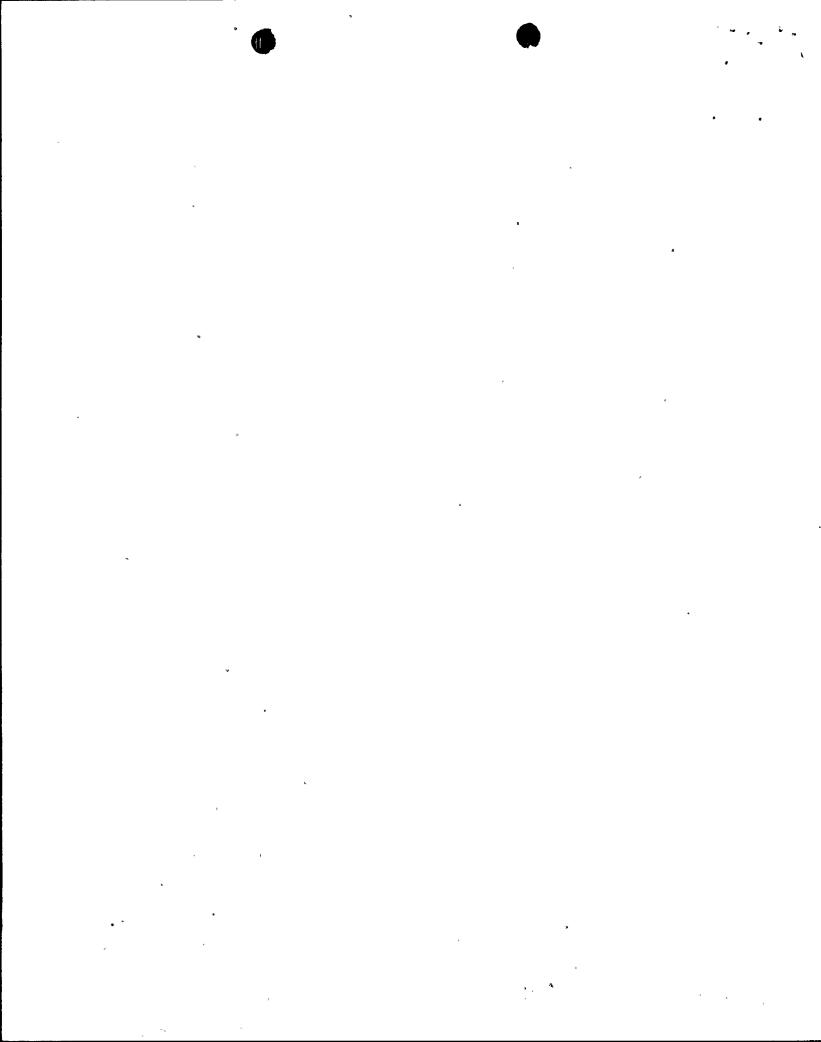
By letter dated April 18, 1980 superseded by letter dated April 21, 1983 Niagara Mohawk Power Corporation (the licensee) proposed changes to the Technical Specifications (TS) of Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station, Unit No. 1. The revision to the Technical Specifications addressed in this Safety Evaluation regards the following changes to the Administrative Controls (Section 6.0): (1) accommodate an increase and improved staff in the on-site organization, correct title changes in the organization and reflect the strengthening of the Quality Assurance (QA) function by elevation of the Manager of QA to a Vice President reporting driectly to the President; (2) include changes for the frequency of audits related to the Safeguards Contingency Plan and Emergency Preparedness program from every two years to annually; (3) change the provisions for entering and controlling entry to high radiation areas by imposing additional conditions similar to those included in current Standard Technical Specifications for BWRs; and (4) correct a typographical error with regard to the provisions for the fire brigade staff.

Evaluation

The licensee has proposed changes to Section 6.0, Administrative Controls of the Technical Specifications to reflect the increased on-site staffing, correct organizational titles for consistency with the increased site staffing, and to reflect the strengthening of the Quality Assurance function. The staff has reviewed the changes and finds them to be acceptable. In particular, the changes made to the QA function should tend to provide an administrative framework for a strong organization. Changes made with regard to the site Operations Review Committee and the Safety Review and Audit Board and associated actions and records are strictly administrative in that they only correct changes in titles.

The licensee has proposed changes to the frequency of audits for the Safeguards Contingency Plan and Emergency Preparedness program. The staff issued Generic Letter 82-23 and Generic Letter 82-17 requesting licensees to eliminate inconsistencies in their Technical Specifications with changes in the Code of Federal Regulations for audits of Safeguards and Emergency Preparedness programs. The changes submitted by the licensee are acceptable in that audit frequencies were changed from every two years to annually meeting the current Regulations.

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The licensee has proposed changes to the High Radiation Area part of Section 6.0 of the Administrative Controls section to impose additional conditions for entering and controlling entry into high radiation areas. The licensee in a telecon on January 12, 1984 agreed with the staff to some slight changes with the language of the section to achieve closer conformity with the approved Standard Technical Specifications. In particular, the paragraph 6.12 was changed to remove Extended Radiation Work Permits from the proposed Technical Specification but to allow issuance of a Radiation Work Permit in accordance with site approved procedures. With this change to the proposed Technical Specification for the High Radiation Area, the specification is virtually identical to the Standard Technical Specifications. The changes made did not have any substantive effect regarding the description of the action contained in the Federal Register notice of this action. The staff finds the changes submitted to be acceptable since they are essentially similar to those contained in Standard Technical Specifications which meet the staff criteria for controlling entry and entering into high radiation areas.

The last proposed change is the correction of "four" to "five" on page 245a of the Technical Specification. This changes a typographical error to achieve consistency with the number five contained in current specifications. The staff finds this administrative change acceptable.

Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact, and pursuant to $10 \ \text{CFR} \ \S 51.5(d)(4)$, that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Robert A. Hermann

Dated: March 20, 1984

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