



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 56  
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated April 18, 1980 superseded by April 21, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 56, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

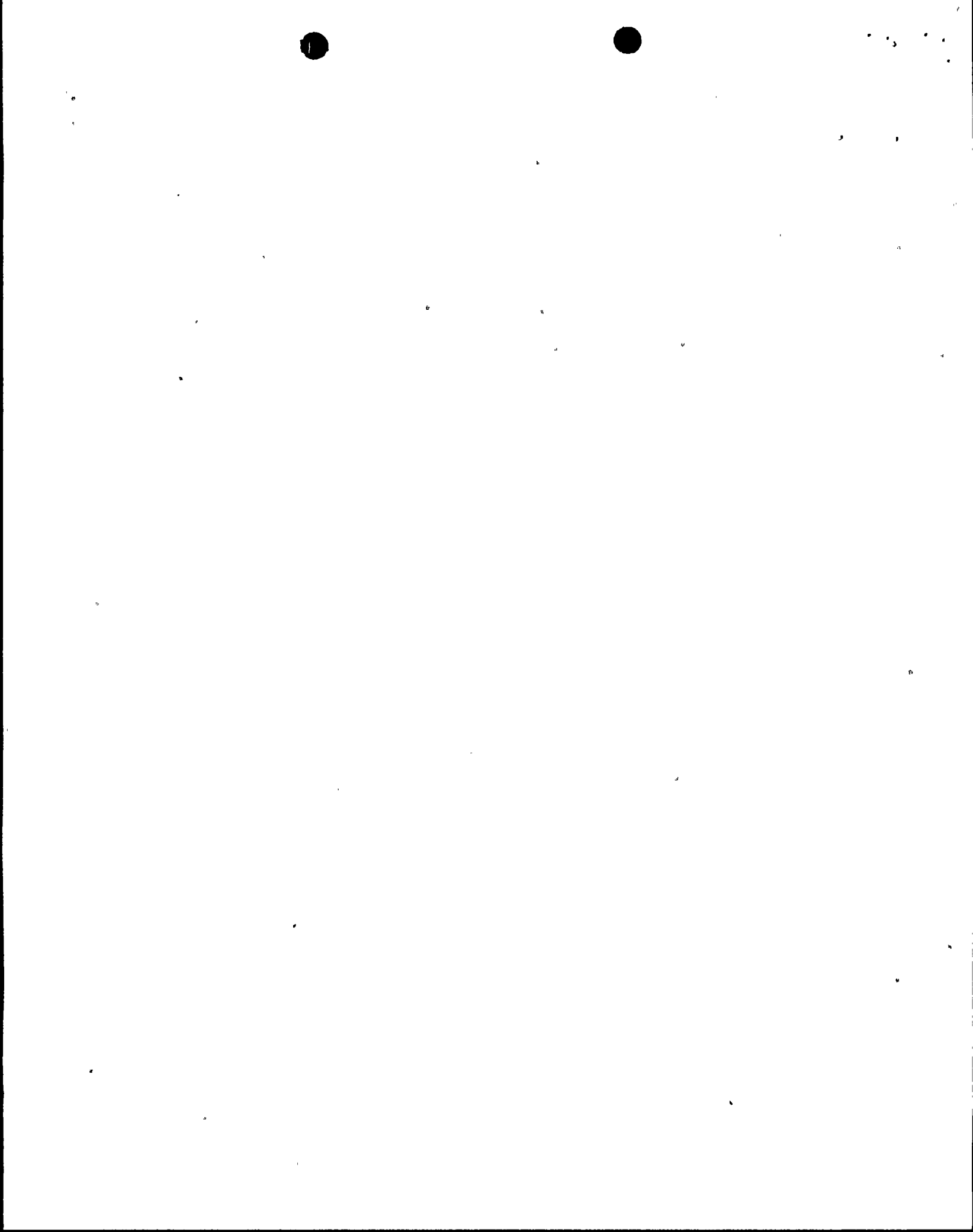
3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 20, 1984



ATTACHMENT TO LICENSE AMENDMENT NO. 56

FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise the Appendix A Technical Specifications by removing and inserting the following pages. The revised area is indicated by marginal lines.

<u>Existing Page</u>	<u>Revised Page</u>
245a	245a
246	246
247	247
249	250
251	252
255	255
256	256
263	263



Facility Staff (Continued)

- f. A Fire Brigade of five (5) members shall be maintained on site as defined by 5.1 at all times.

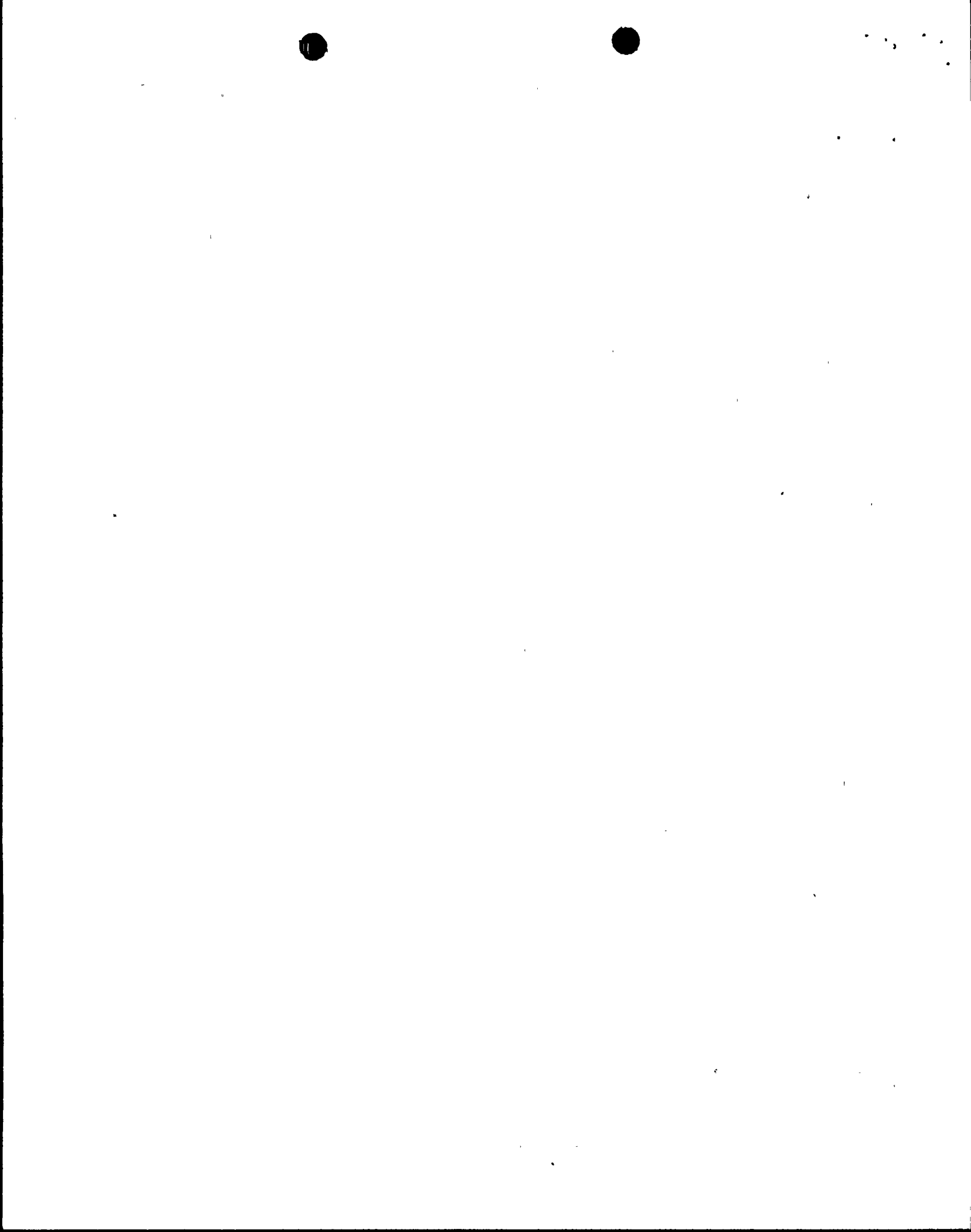




FIGURE 6.2.-1

NINE MILE POINT NUCLEAR STATION  
MANAGEMENT ORGANIZATION CHART

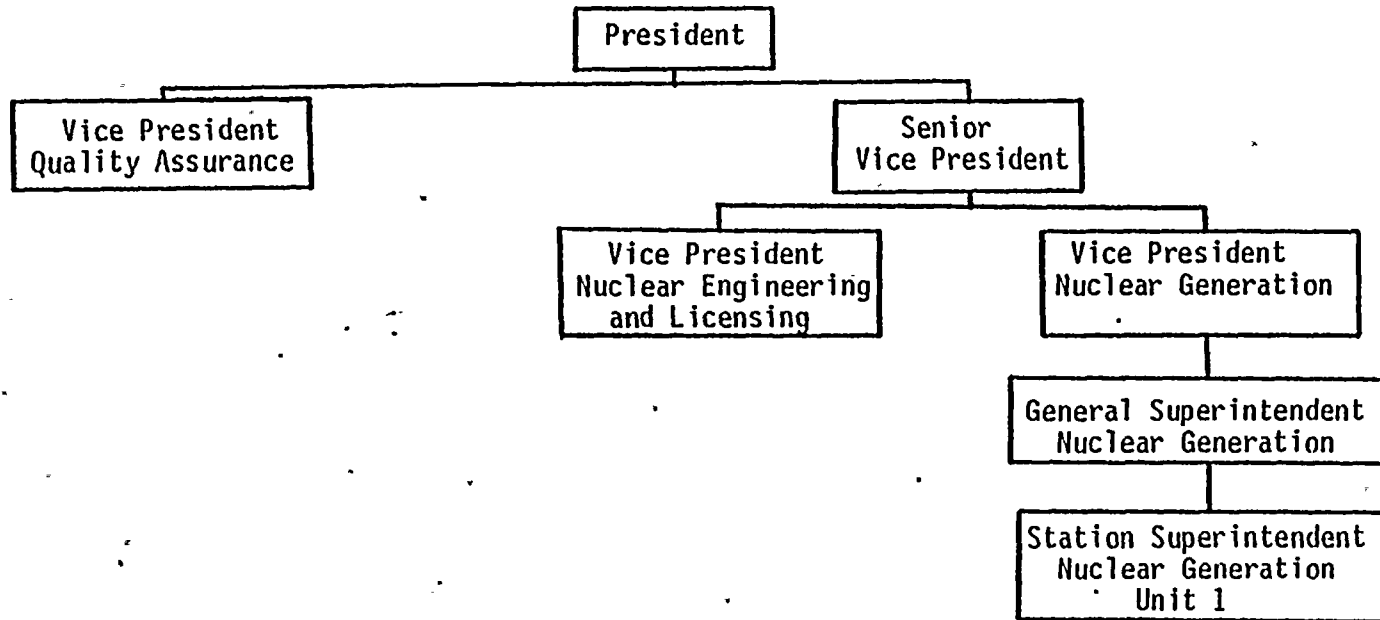
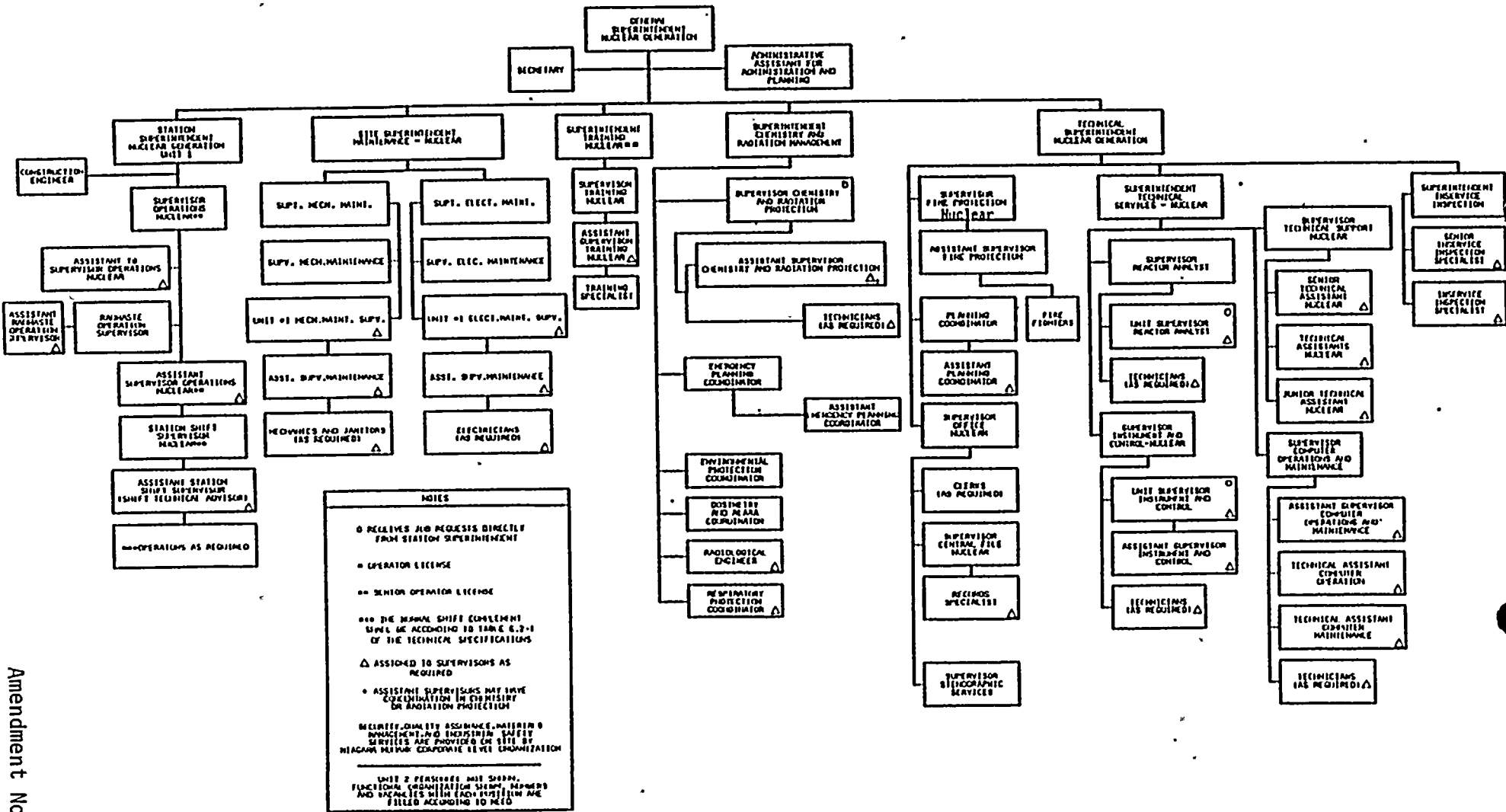




FIGURE 6.2-2

ORGANIZATION CHART



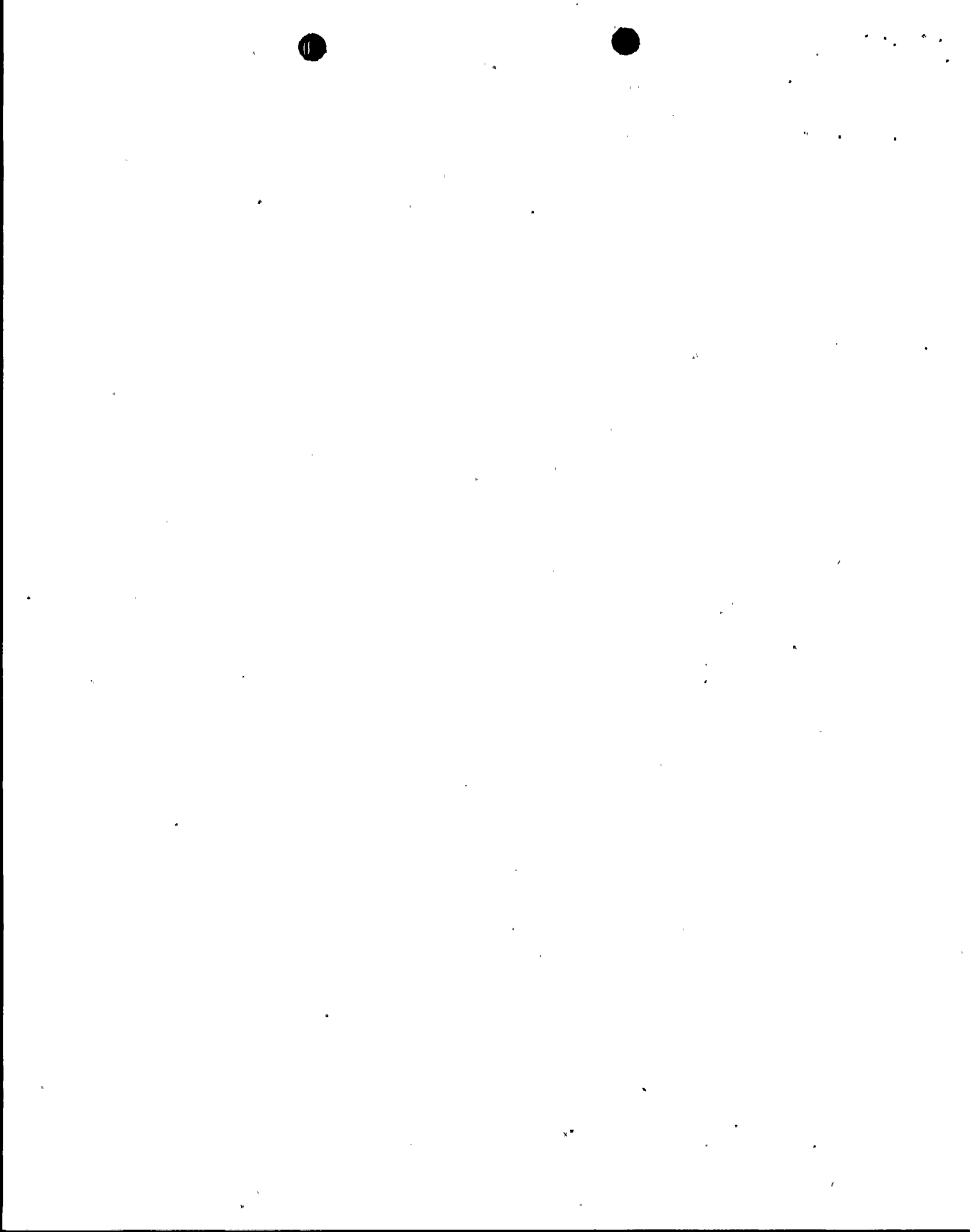
**NOTES**

- RECEIVES AND REQUESTS DIRECTLY FROM STATION SUPERINTENDENT
- OPERATOR LICENSE
- ◐ SENIOR OPERATOR LICENSE
- ◑ THE NORMAL SHIFT COMPLEMENT SHALL BE ACCORDING TO TABLE 6.2-1 OF THE TECHNICAL SPECIFICATIONS
- △ ASSIGNED TO SUPERVISORS AS REQUIRED
- ASSISTANT SUPERVISORS MAY HAVE COORDINATION IN THE AREA OF RADIATION PROTECTION

MAINTENANCE, QUALITY ASSURANCE, MATERIALS MANAGEMENT AND INSPECTION SAFETY SERVICES ARE PROVIDED ON SITE BY NIAGARA MOHAWK CORPORATE LEVEL ORGANIZATION

UNIT 2 PERSONNEL NOT SHOWN. FUNCTIONAL ORGANIZATION SHOWN. MEMBERS AND POSITIONS WITHIN EACH POSITION ARE FILLED ACCORDING TO NEED

Amendment No. 56



6.3 Facility Staff Qualifications

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents.

6.4 Training

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Superintendent-Training Nuclear and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Superintendent-Training Nuclear and Supervisor-Fire Protection, Nuclear and shall meet or exceed the requirements of Appendix R to 10CFR50.

6.5 Review and Audit

6.5.1 Site Operations Review Committee (SORC)

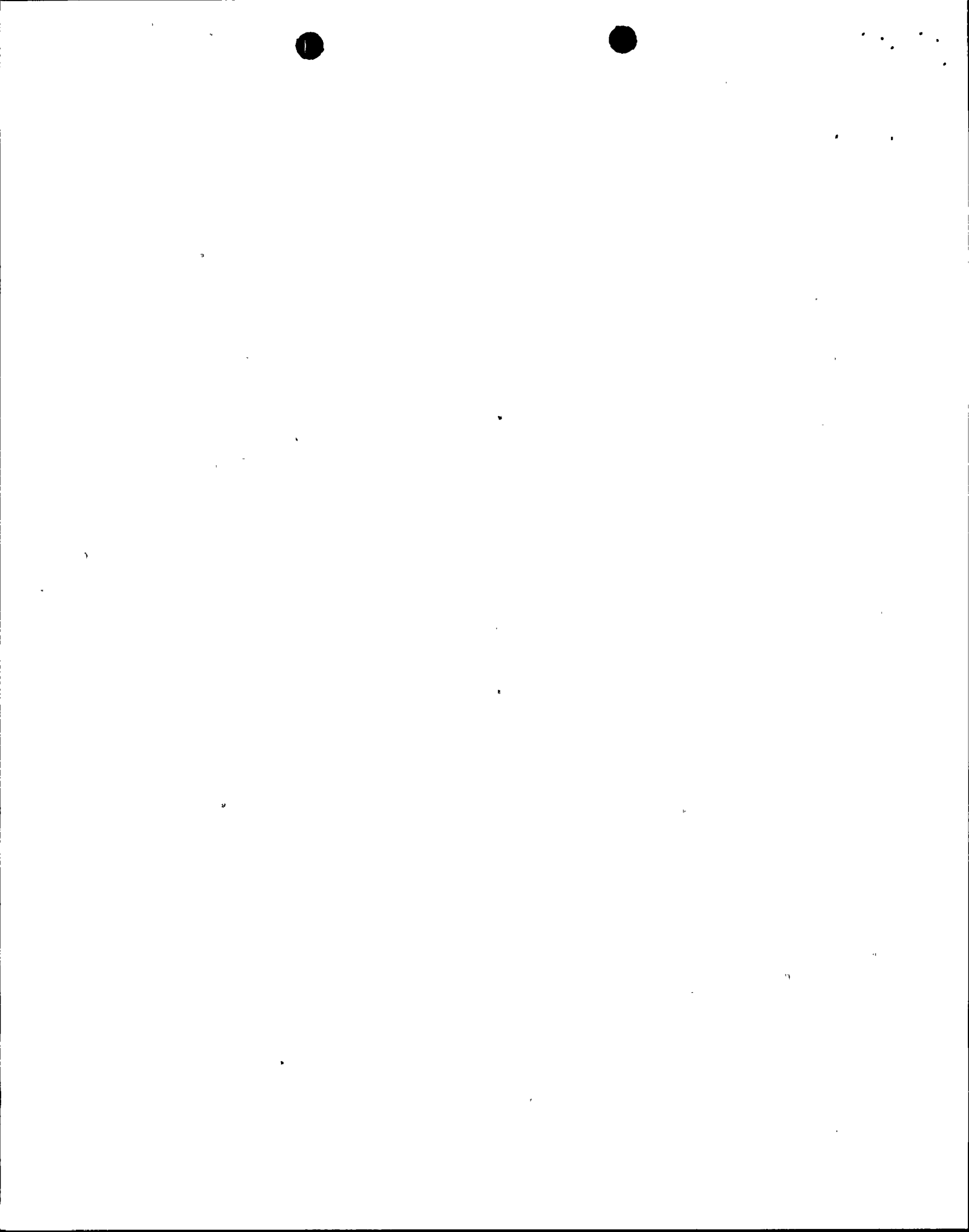
Function

6.5.1.1 The Site Operations Review Committee shall function to advise the General Superintendent-Nuclear Generation on all matters related to nuclear safety.

Composition

6.5.1.2 The Site Operations Review Committee shall be composed of the:

Chairman:	General Superintendent - Nuclear Generation
Member:	Station Superintendent - Nuclear Generation
Member:	Technical Superintendent - Nuclear Generation
Member:	Superintendent Technical Services - Nuclear
Member:	Site Superintendent Maintenance - Nuclear
Member:	Supervisor Instrument and Control - Nuclear
Member:	Superintendent Chemistry and Radiation Management



### Alternates

6.5.1.3 Alternate members shall be appointed in writing by the SORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in SORC activities at any one time.

### Meeting Frequency

6.5.1.4 The SORC shall meet at least once per calendar month and as convened by the SORC Chairman.

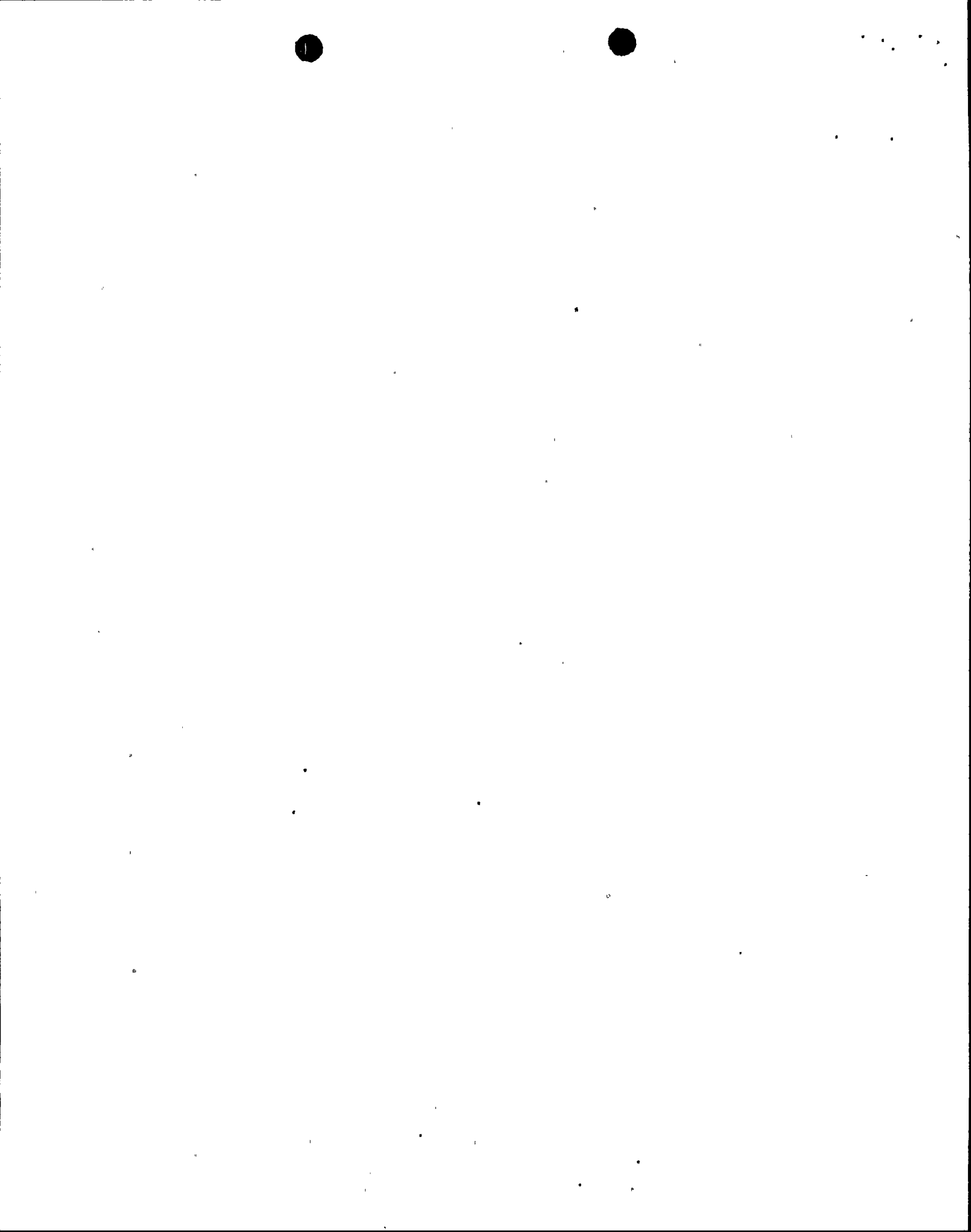
### Quorum

6.5.1.5 A quorum of the SORC shall consist of the Chairman and four members including alternates.

### Responsibilities

6.5.1.6 The Site Operations Review Committee shall be responsible for: -

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the General Superintendent-Nuclear Generation to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to the Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications and shall prepare and forward a report covering evaluation and recommendations to prevent recurrence to the Vice President - Nuclear Generation and to the Chairman of the Safety Review and Audit Board.





## Responsibilities (Continued)

- f. Review of facility operations to detect potential safety hazards.
- g. Performance of special reviews and investigations and reports thereon as requested by the Chairman of the Safety Review and Audit Board.
- h. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Chairman of the Safety Review and Audit Board.
- i. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the Safety Review and Audit Board.

## Authority

6.5.1.7 The Site Operations Review Committee shall:

- a. Recommend to the General Superintendent-Nuclear Generation written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6 (a) through (e) above constitutes an unreviewed safety question.
- c. Provide immediate written notification to the Vice President-Nuclear Generation and Chairman of the Safety Review and Audit Board of disagreement between the SORC and the General Superintendent-Nuclear Generation; however, the General Superintendent-Nuclear Generation shall have the responsibility for resolution of such disagreements pursuant to 6.1.1 above.

## Records

6.5.1.8 The Site Operations Review Committee shall maintain written minutes of each meeting and copies shall be provided to the Vice President - Nuclear Generation and Chairman of the Safety Review and Audit Board.



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6.5.2 Safety Review and Audit Board (SRAB)

Function

6.5.2.1 The Safety Review and Audit Board shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control.
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices
- i. (other appropriate fields associated with the unique characteristics of the nuclear power plant)

Composition

6.5.2.2 The Safety Review and Audit Board shall be composed of the:

Chairman:	Staff Engineer or Manager or Vice President
Member:	General Superintendent - Nuclear Generation
Member:	Staff Engineer - Nuclear
Member:	Staff Engineer - Mechanical or Electrical
Member:	Staff Engineer - Environmental
Member:	Consultant (See 6.5.2.4)



## Audits (Continued)

- d. The performance of all activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10CFR50, at least once per two years.
- e. The Facility Emergency Plan and implementing procedures at least once every 12 months.
- f. The Facility Security Plan and implementing procedures at least once every 12 months.
- g. The Facility Fire Protection Program and implementing procedures at least once per two years.
- h. Any other area of facility operation considered appropriate by the SRAB, the Vice President-Nuclear Generation or the Vice President-Nuclear Engineering and Licensing.

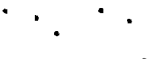
## Authority

6.5.2.9 The SRAB shall report to and advise the Vice President-Nuclear Generation and Vice President-Nuclear Engineering and Licensing on those areas of responsibility specified in Section 6.5.2.7 and 6.5.2.8.

## Records

6.5.2.10 Records of SRAB activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each SRAB meeting shall be prepared, approved and forwarded to the Vice President-Nuclear Generation and Vice President-Nuclear Engineering and Licensing within 30 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 e, f, g and h above, shall be prepared, approved and forwarded to the Vice President-Nuclear Generation and Vice President-Nuclear Engineering and Licensing within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above shall be forwarded to the Vice President-Nuclear Generation and Vice President-Nuclear Engineering and Licensing within 14 days following completion of the review.



6.6 Reportable Occurrence Action

6.6.1 The following actions shall be taken in the event of a REPORTABLE OCCURRENCE:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each Reportable Occurrence Report submitted to the Commission shall be reviewed by the SORC and submitted to the SRAB and the Vice President - Nuclear Generation.

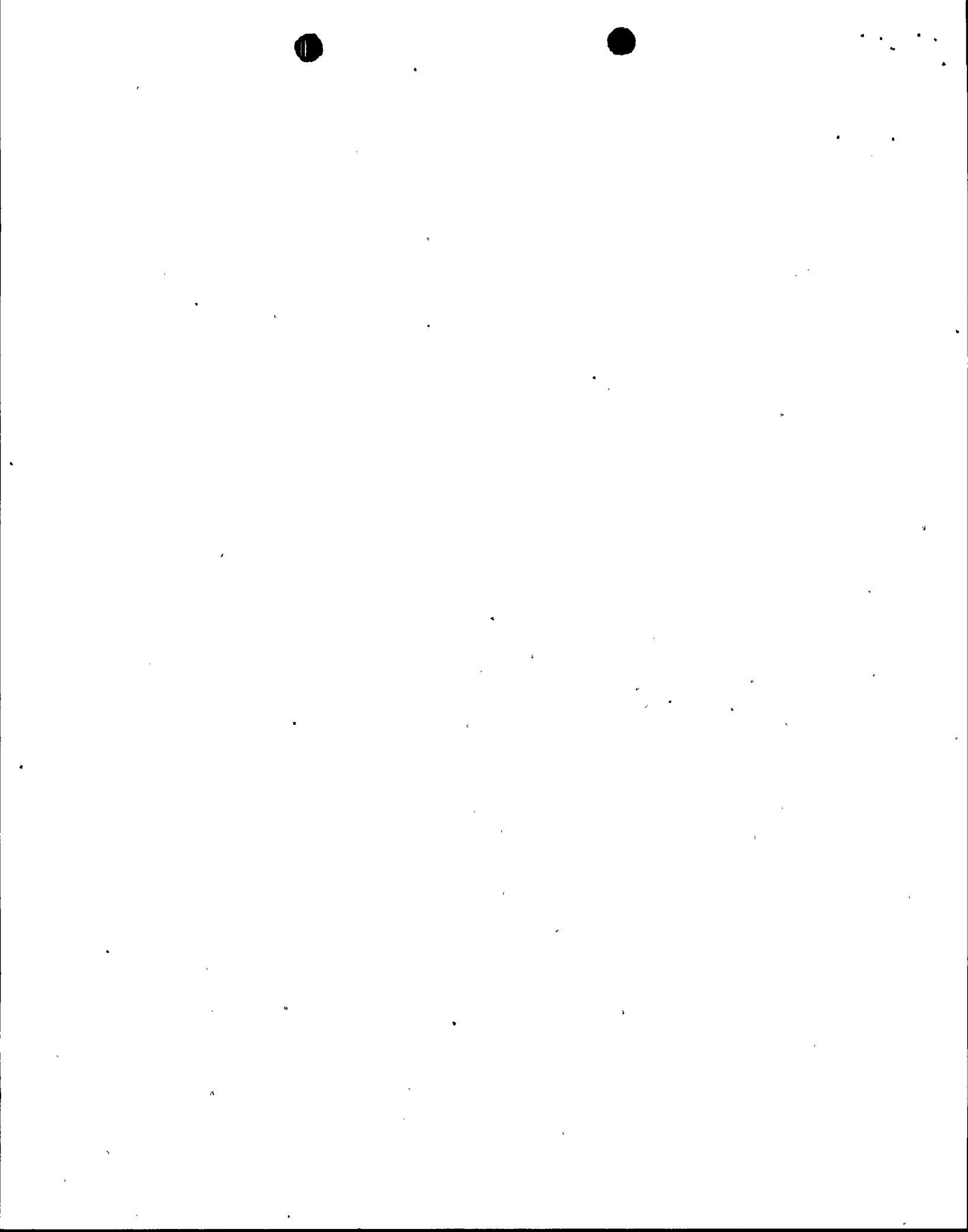
6.7 Safety Limit Violation

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10CFR50.36(c)(1)(i) shall be complied with immediately.
- b. The Safety Limit violation shall be reported to the Commission, the Vice President - Nuclear Generation and to the SRAB immediately.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the SRAB and the Vice President - Nuclear Generation within 10 days of the violation.

6.8 Procedures

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Sections 5.1 and 5.3 of ANSI N18.7-1972 and Appendix "A" of USAEC Regulatory Guide 1.33 except as provided in 6.8.2 and 6.8.3 below.





6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20,203(c)(2) of 10 CFR 20, each high radiation area normally accessible\* by personnel in which the intensity of radiation is greater than 100 mrem/hr\*\* but less than 1000 mrem/hr\*\* shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit in accordance with site-approved procedures. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rates in the area have been established and personnel have been made knowledgeable of them.
- c. An individual qualified in radiation protection, with a radiation dose rate monitoring device, who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the unit Radiation Protection Supervisor in the Radiation Work Permit.

6.12.2 In addition to the requirements of 6.12.1 areas accessible to personnel with radiation levels such that a major portion of the body could receive in one hour a dose greater than 1000 mrem\*\* shall be provided with locked doors to prevent unauthorized entry, and the hard keys or access provided by magnetic keycard shall be maintained under the administrative control of the Station Shift Supervisor or designate on duty and/or the unit Radiation Protection Supervisor or designate. Doors shall remain locked except during periods of access by personnel under an approved RWP which shall specify the appropriate dose rate levels in the immediate work area and the maximum allowable stay time for individuals in that area. In lieu of the stay time specification of the RWP, continuous surveillance, direct or remote, such as use of closed circuit TV cameras, may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities within the area. For individual areas accessible to personnel with radiation levels such that a major portion of the body could receive in one hour a dose in excess of 1000 mrem\*\* that are located within large areas, such as the drywell, where no enclosure exists for purpose of locking, and no enclosure can be reasonably constructed around the individual areas, then that area shall be roped off, conspicuously posted and a flashing light shall be activated as a warning device.

\* by accessible passage and permanently fixed ladders  
\*\* measurement made at 18" from source of radioactivity

