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 FACIL: 50-220 Nine Mile Point Nuclear Station, Unit 1, Niagara Power 05000220
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 MANGAN, C.V. Niagara Mohawk Power Corp.
 RECIPIENT NAME: RECIPIENT AFFILIATION
 VASSALLO, D.B. Operating Reactors Branch 2

SUBJECT: Provides addl info re Type C testing of emergency cooling & reactor cleanup inboard isolation valves & method for extrapolation of leakage test results from reduced pressure to accident pressure, per 831101 Tech Spec amend application.

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 TITLE: OR Submittal: Append J. Containment Leak Rate Testing

NOTES:

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March 20, 1984

Director of Nuclear Reactor Regulation
Attention: Mr. Domenic B. Vassallo, Chief
Operating Reactors Branch No. 2
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Re: Nine Mile Point Unit 1
Docket No. 50-220
DPR-63

Dear Mr. Vassallo:

In response to a request by members of your staff to provide additional information regarding the Niagara Mohawk November 1, 1983 technical specification amendment application, the following information is provided. This information addresses Type C testing of the emergency cooling and reactor cleanup inboard isolation valves. In addition, the method used for extrapolation of leakage test results from reduced pressure to accident pressure is also provided.

Emergency Cooling System

A separate Type C test cannot be performed on the inboard steam supply isolation valve with the present system configuration. A reactor vessel nozzle plug will be procured in order to test the valves in the proper direction, but is not expected to be available during the present outage. It will be available and used for subsequent Type C tests. In the interim, a pneumatic Type C test will be performed by applying test pressure between the inboard and outboard steam supply isolation valves. The measured leakage will be assigned to the penetration and the results will be reviewed against the criteria stated in the Technical Specifications.

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Reactor Cleanup System

The reactor cleanup inboard isolation valves cannot be Type C tested. The present system piping configuration does not include a manual valve which would isolate the inboard isolation valves from the reactor vessel. Also, installation of a reactor vessel nozzle plug is not feasible. Therefore, these isolation valves will be hydraulically tested with acceptance criteria of less than one gallon per minute as stated in our November 1, 1983 letter.

Extrapolation of Reduced Test Pressure Results

Our present technique for the reduced pressure test of containment airlock doors is based on experimental data. A series of leakage tests were performed at test pressures of 22 and 35 psig at Nine Mile Point Unit 1. The results of these tests formed the basis for our extrapolation methodology which is consistent with 10CFR50 Appendix J. Therefore, Niagara Mohawk considers our present technique acceptable.

Sincerely,

NIAGARA MOHAWK POWER CORPORATION

C. V. Mangan

C. V. Mangan
Vice President

Nuclear Engineering and Licensing

CVM/PAM:djm

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Very truly yours,

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