

ATTACHMENT A

NIAGARA MOHAWK POWER CORPORATION

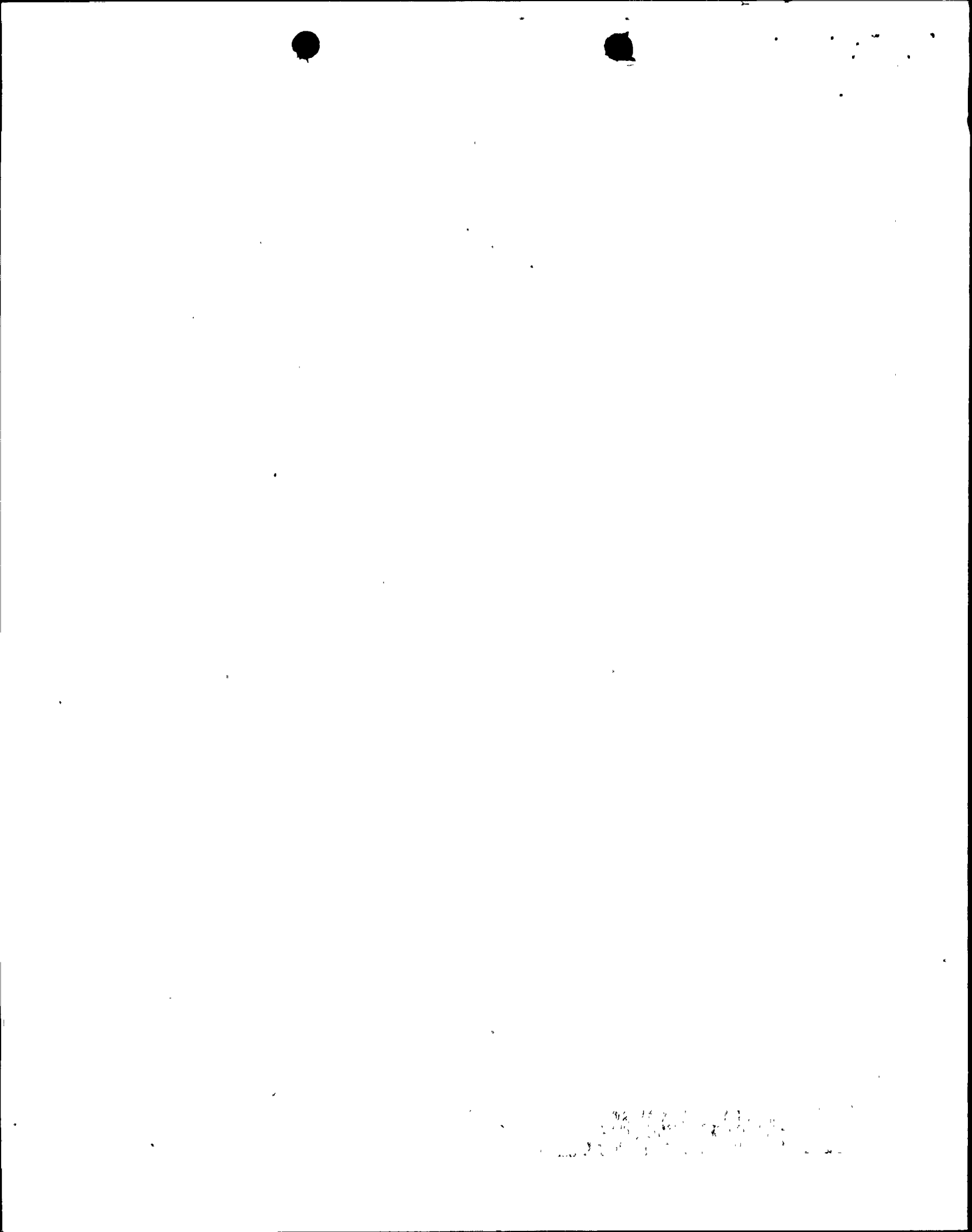
LICENSE NO. DPR-63

DOCKET NO. 50-220

Proposed Changes to Technical Specifications (Appendix A)

Replace page 92-115 and 258 with the attached revised pages. Pages 92-115 have been entirely revised with marginal markings indicating the changes. However, the intentionally blank pages do not contain marginal markings. Page 258 has been retyped entirely with a marginal marking to indicate the changes.

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LIMITING CONDITION FOR OPERATION

3.2.6 INSERVICE INSPECTION AND TESTING

Applicability:

Applies to components which are part of the reactor coolant pressure boundary and their supports and other safety-related pressure vessels, piping, pumps, and valves.

Objective:

To assure the integrity of the reactor coolant pressure boundary and the operational readiness of safety-related pressure vessels, piping, pumps, and valves.

Specification:

a. Inservice Inspection

1. To be considered operable, Quality Group A, B and C components shall satisfy the requirements contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda for continued service of ASME Code Class 1, 2 and 3 components, respectively, except where relief has been granted by the Commission pursuant to 10CFR50, Section 50.55a(g)(6)(i). (1)

SURVEILLANCE REQUIREMENT

4.2.6 INSERVICE INSPECTION AND TESTING

Applicability:

Applies to the periodic inspection and testing of components which are part of the reactor coolant pressure boundary and their supports and other safety-related pressure vessels, piping, pumps, and valves.

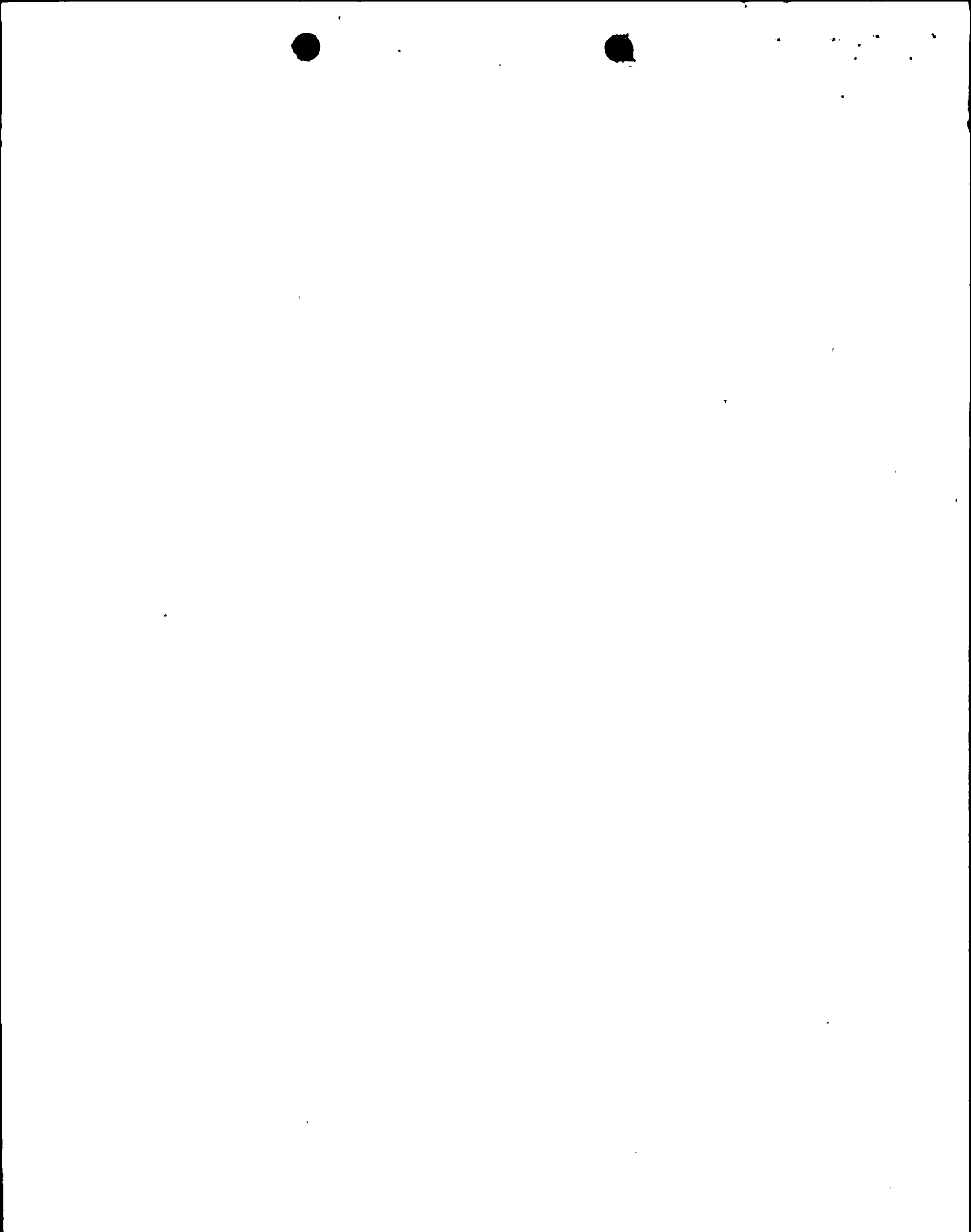
Objective:

To verify the integrity of the reactor coolant pressure boundary and the operational readiness of safety-related pressure vessels, piping, pumps, and valves.

Specification:

a. Inservice Inspection

1. Inservice inspection of Quality Group A, B and C components shall be performed in accordance with the requirements for ASME Code Class 1, 2 and 3 components, respectively, contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10CFR50, Section 50.55a(g), except where relief has been granted by the Commission pursuant to 10 CFR Part 50 Section 50.55a(g)(6)(i). (1)



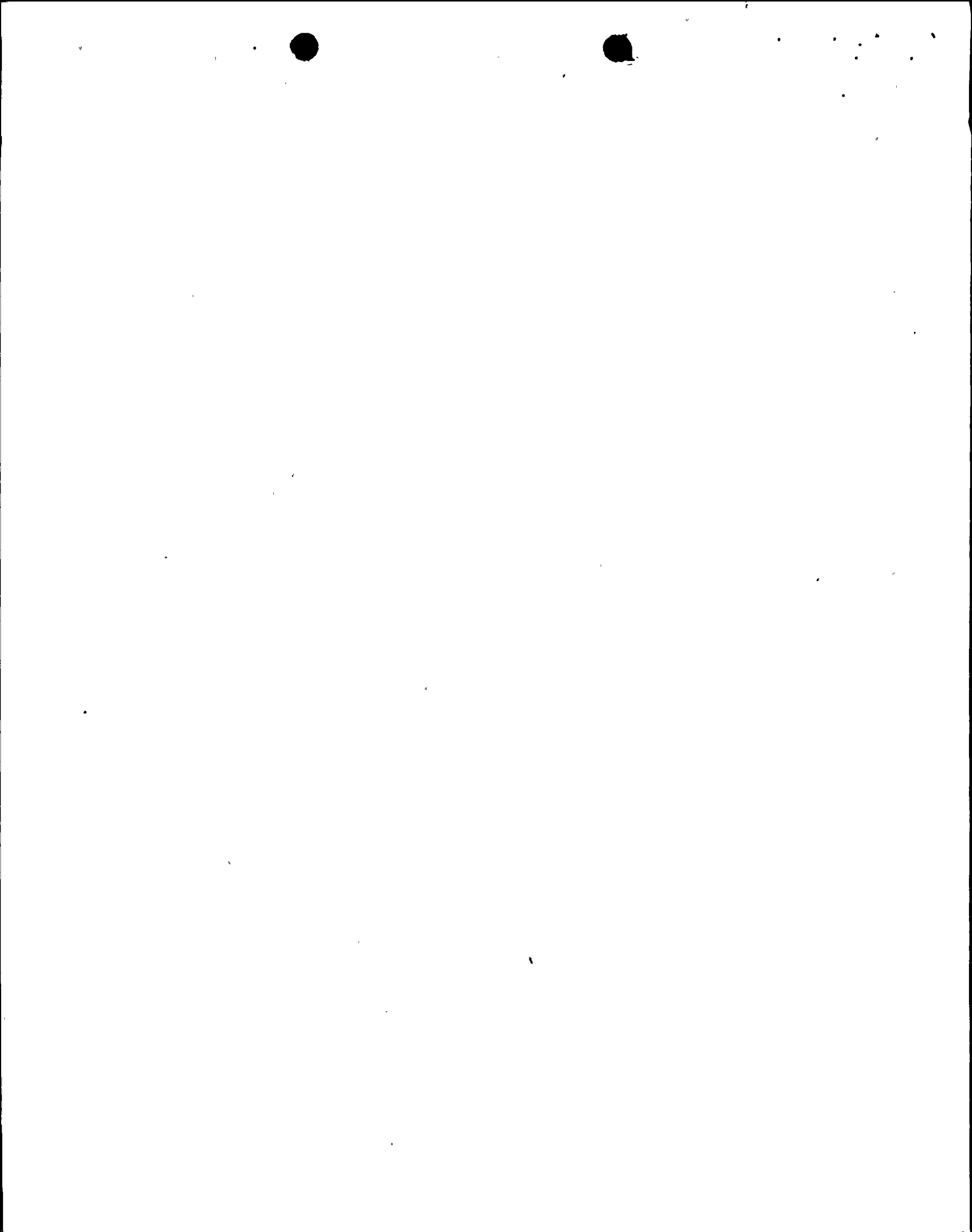
b. Inservice Testing

1. To be considered operable, Quality Group A, B and C pumps and valves shall satisfy the requirements contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda for continued service of ASME Code Class 1, 2 and 3 components, respectively, except where relief has been granted by the Commission pursuant to 10CFR50, Section 50.55a(g)(6)(i).(2)

2. An augmented inservice inspection program shall be performed in accordance with the schedules contained in NUREG 0313 Revision 1. The augmented inservice inspection program shall be performed on service sensitive components. The following systems contain service sensitive components: core spray, shutdown cooling, emergency condensers, liquid poison, reactor head spray and control rod drive return.

b. Inservice Testing

1. Inservice testing of Quality Group A, B and C pumps and valves shall be performed in accordance with the requirements for ASME Code Class 1, 2 and 3 components contained in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10CFR50, Section 50.55a(g), except where relief has been granted by the Commission pursuant to 10 CFR Part 50 Section 50.55a(g)(6)(i).(2)



BASES FOR 3.2.6 AND 4.2.6 INSERVICE INSPECTION AND TESTING

The inservice inspection and testing program for the Nine Mile Point Unit 1 plant conforms to the requirements of 10 CFR 50, Section 50.55a(g). Where practical, the inspection of components, pumps and valves classified into NRC Quality Groups A, B and C conforms to the requirements of ASME Code Class 1, 2 and 3 components, pumps and valves, respectively, contained in Section XI of the ASME Boiler and Pressure Vessel Code. If a Code required inspection is impractical for the Nine Mile Point Unit 1 facility, a request for a deviation from that requirement is submitted to the Commission in accordance with 10 CFR 50, Section 50.55a(g)(6)(i).

Deviations which are needed from the procedures prescribed in Section XI of the ASME Code and applicable Addenda will be reported to the Commission prior to the beginning of each 10-year inspection period if they are known to be required at that time. Deviations which are identified during the course of inspection will be reported quarterly throughout the inspection period.

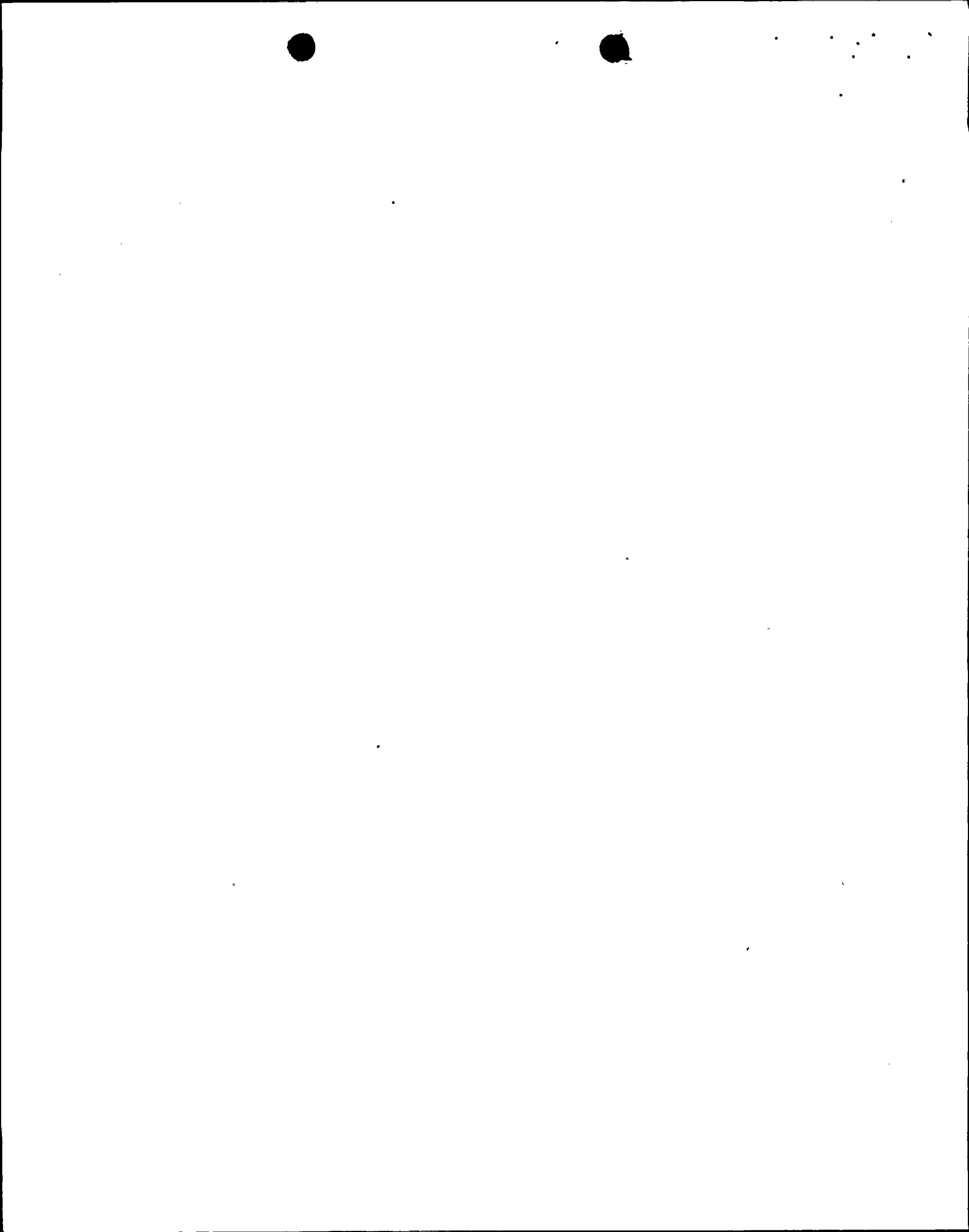
The augmented inservice inspection program for the Nine Mile Point Unit 1 plant conforms to the schedules contained in NUREG 0313 Revision 1. It is performed in order to detect and survey intergranular stress corrosion cracking of ASME Code Class 1, 2 and 3 pressure boundary piping. Inspections shall be performed by individuals qualified to (1) the ASME Boiler and Pressure Vessel Code, Section XI, as specified to the NRC, and (2) Ultrasonic Testing Operator Training for Intergranular Stress Corrosion Cracking developed by the EPRI Non-Destructive Examination Center, as specified to the NRC.

References

- (1) Letter from the Nuclear Regulatory Commission (D. B. Vassallo) to Niagara Mohawk Power Corporation (G. K. Rhode), dated September 19, 1983.
- (2) Letter from Niagara Mohawk Power Corporation (D. P. Dise) to the Nuclear Regulatory Commission (T. A. Ippolito), dated August 7, 1981.



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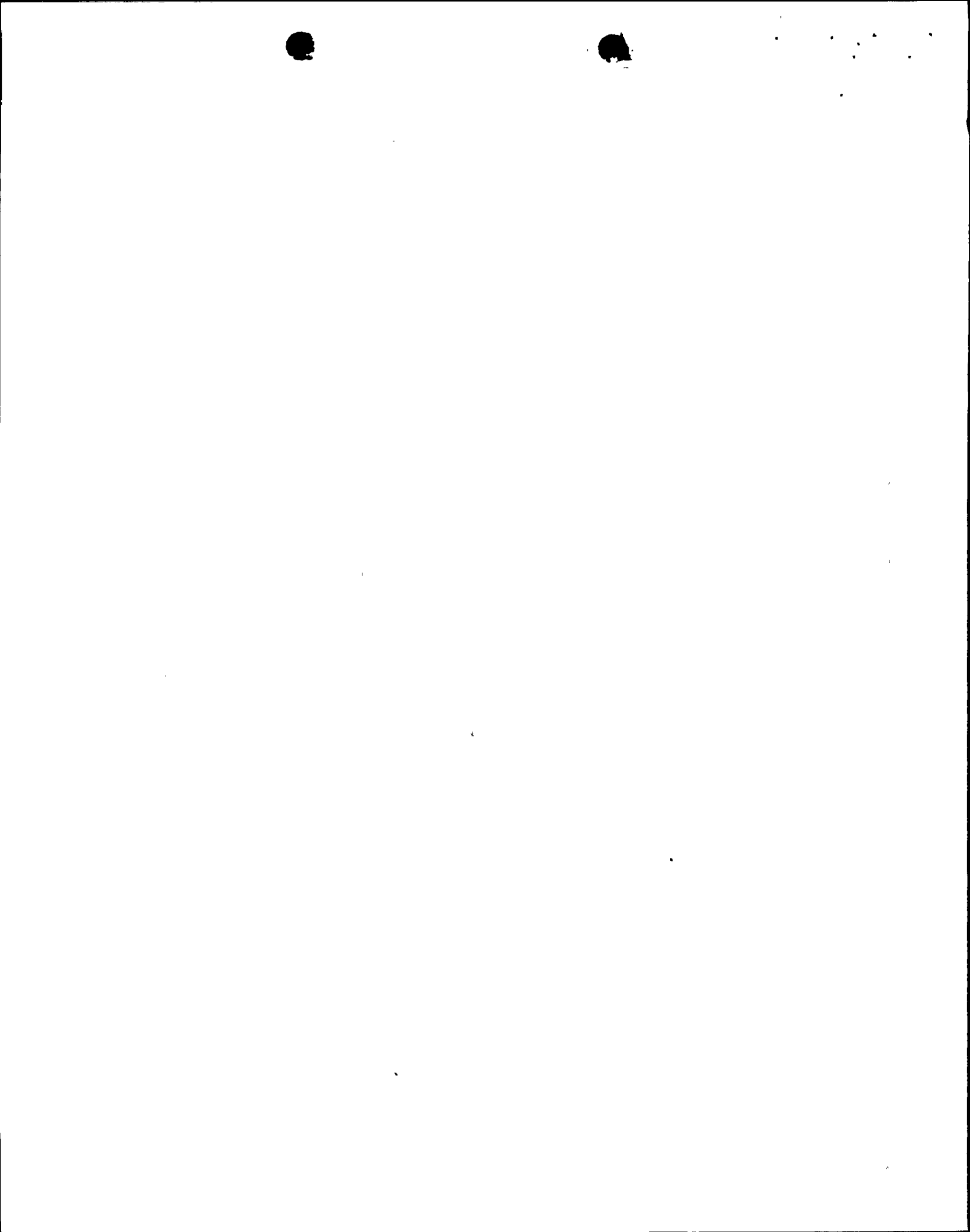


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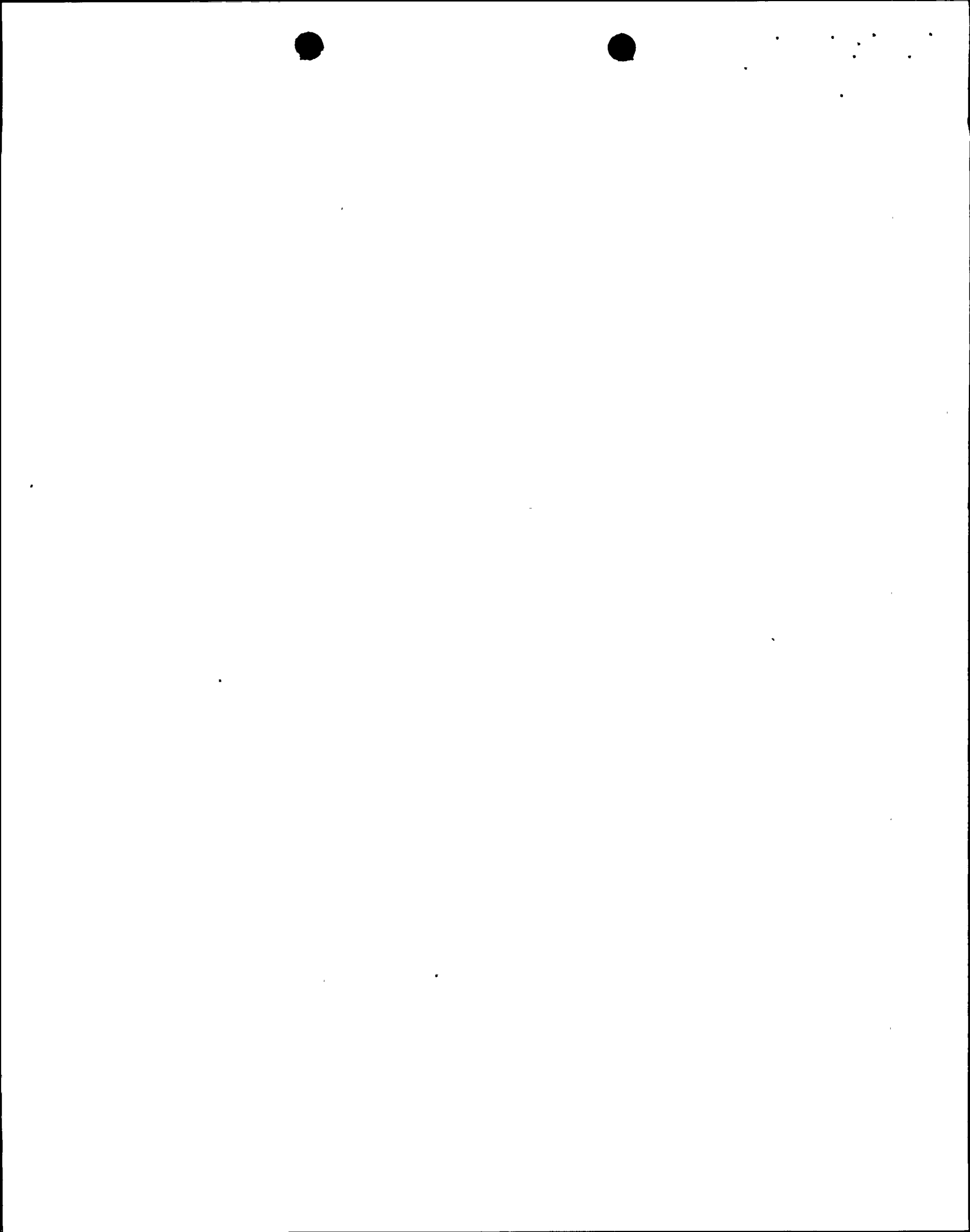


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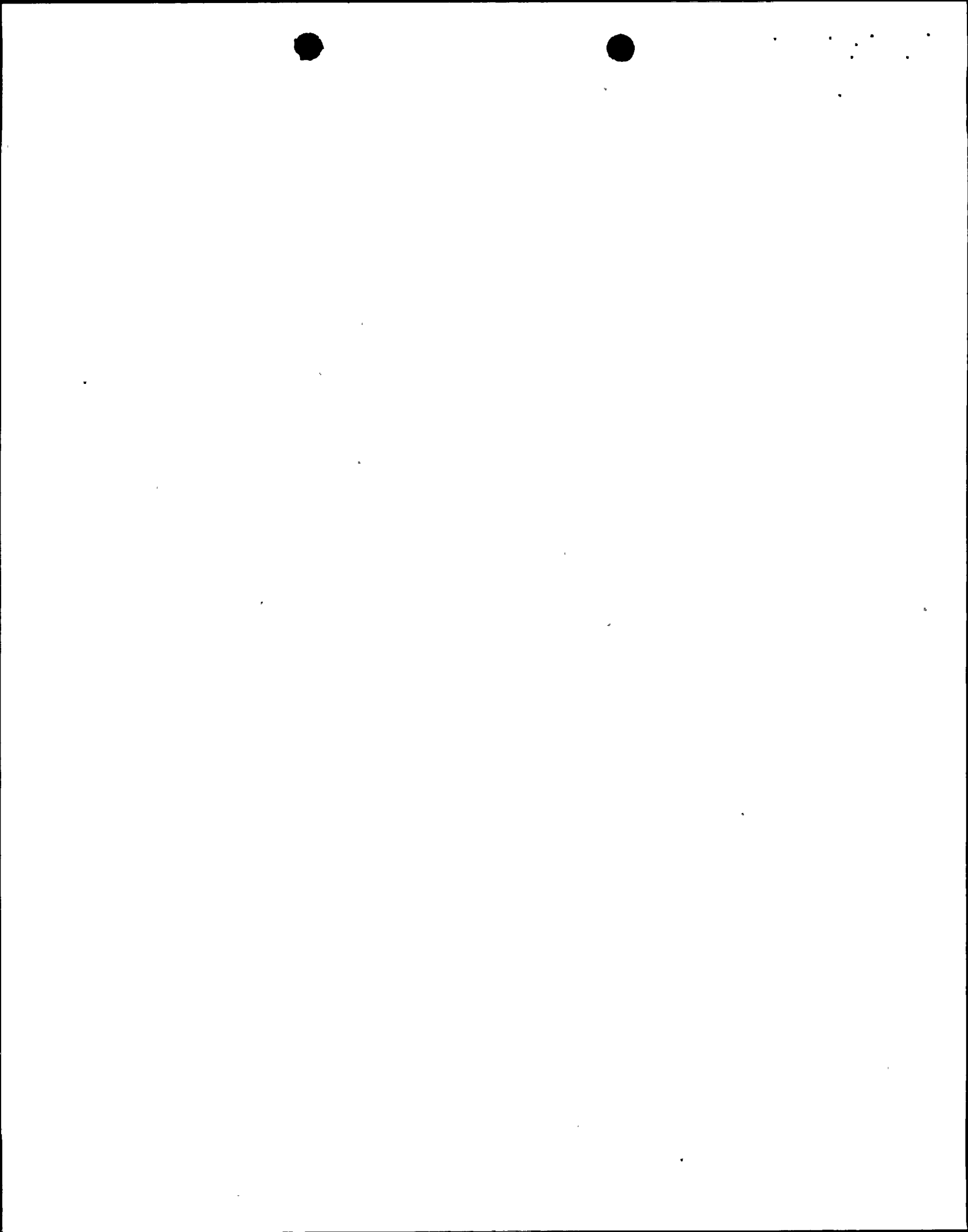
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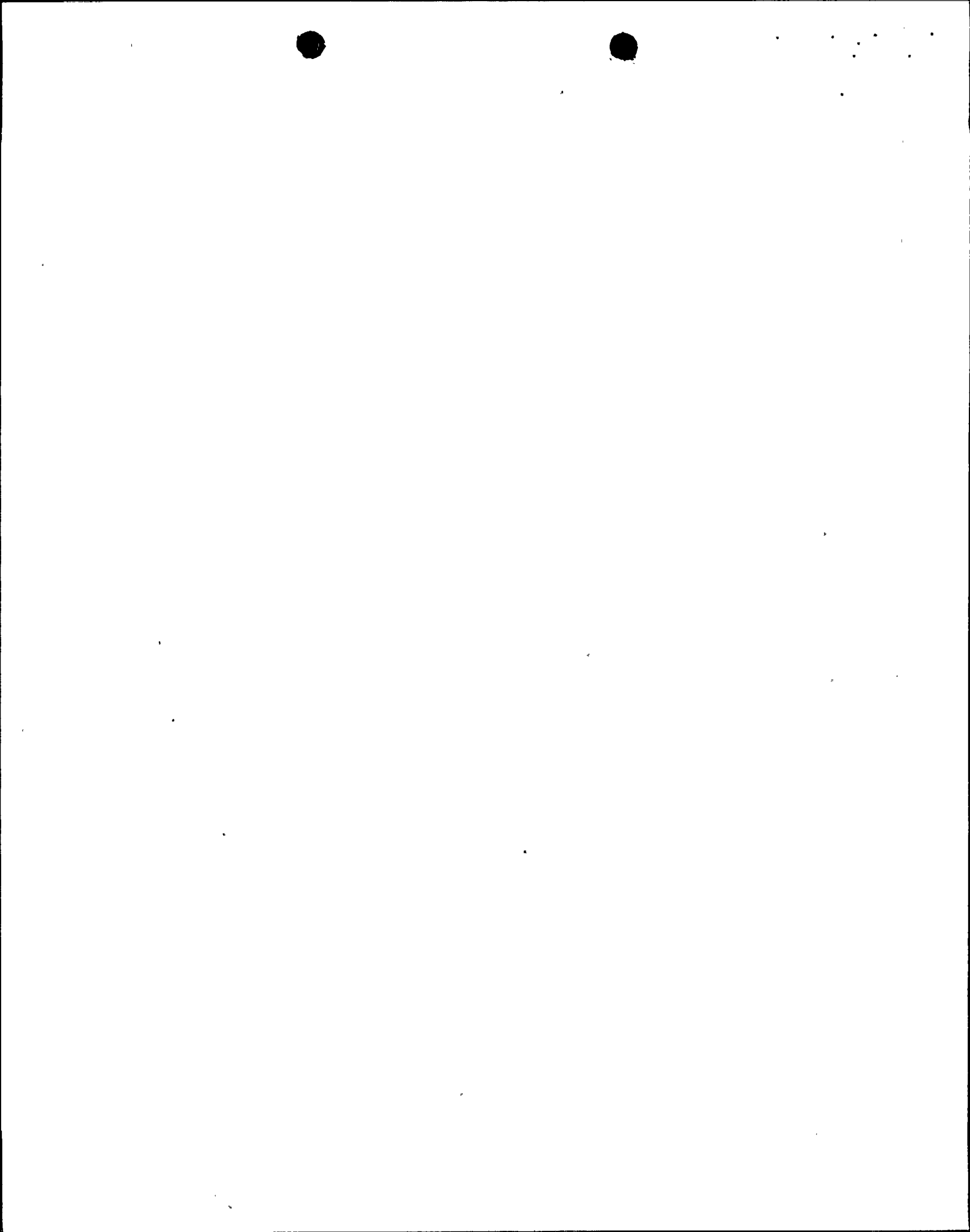
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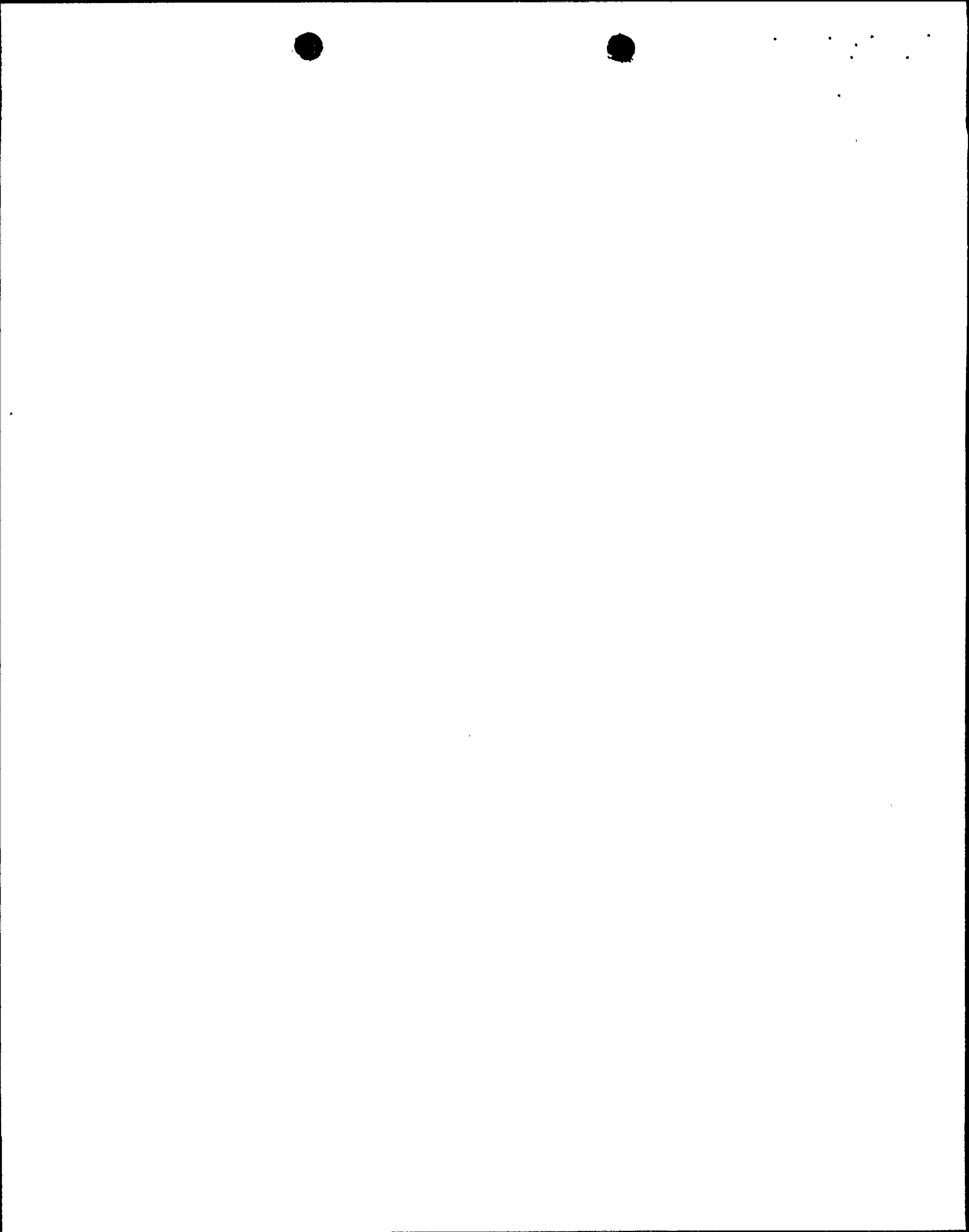
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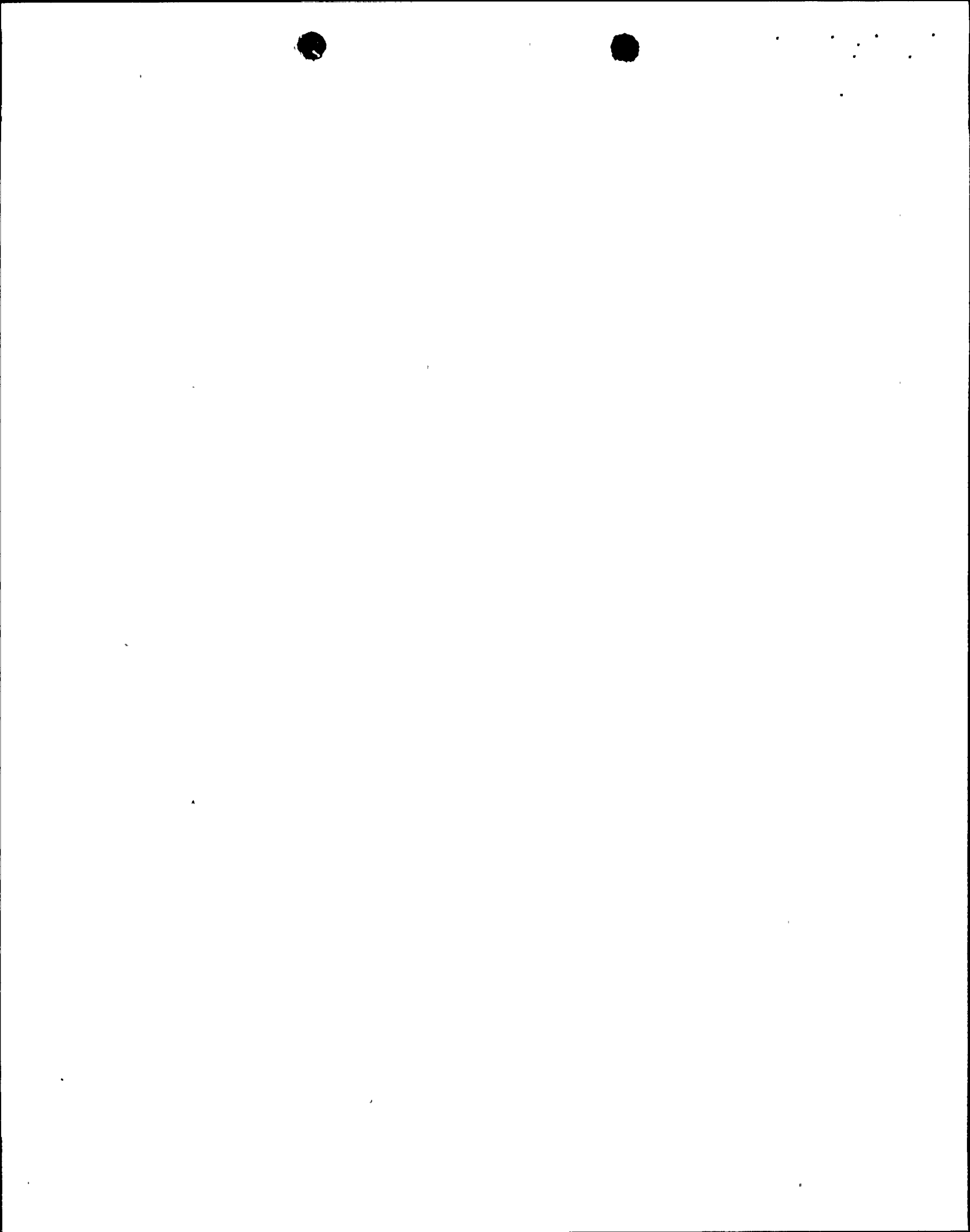
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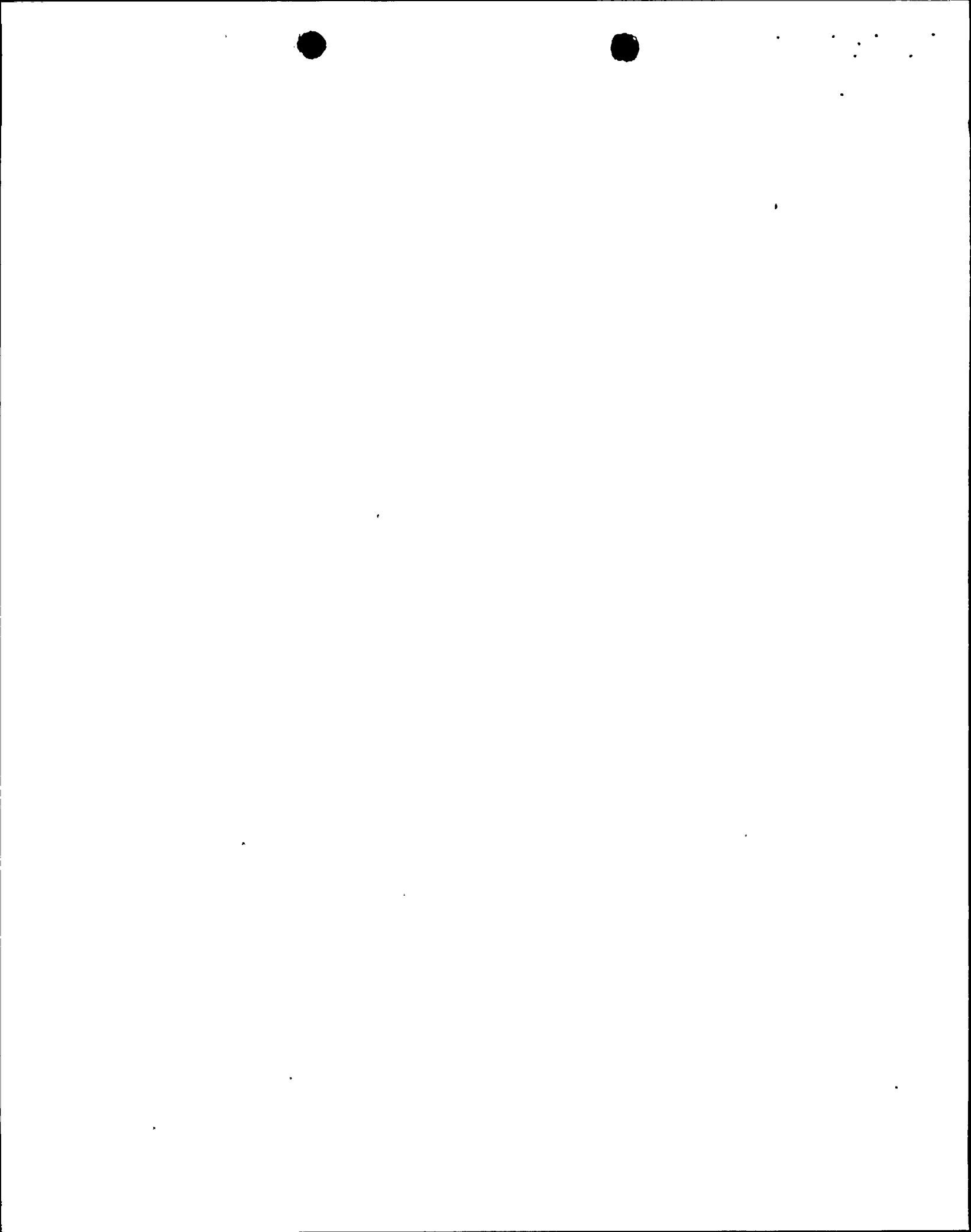
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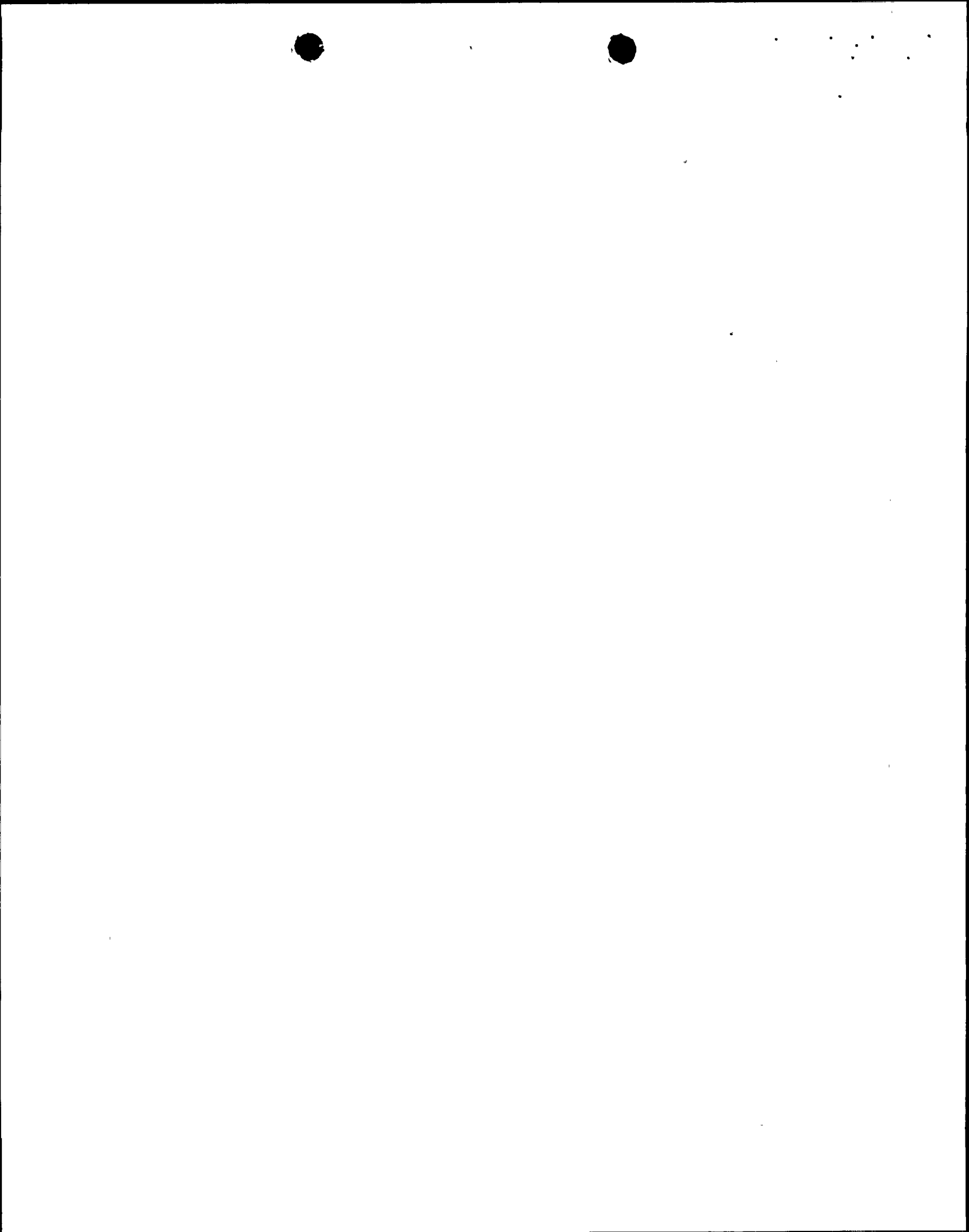
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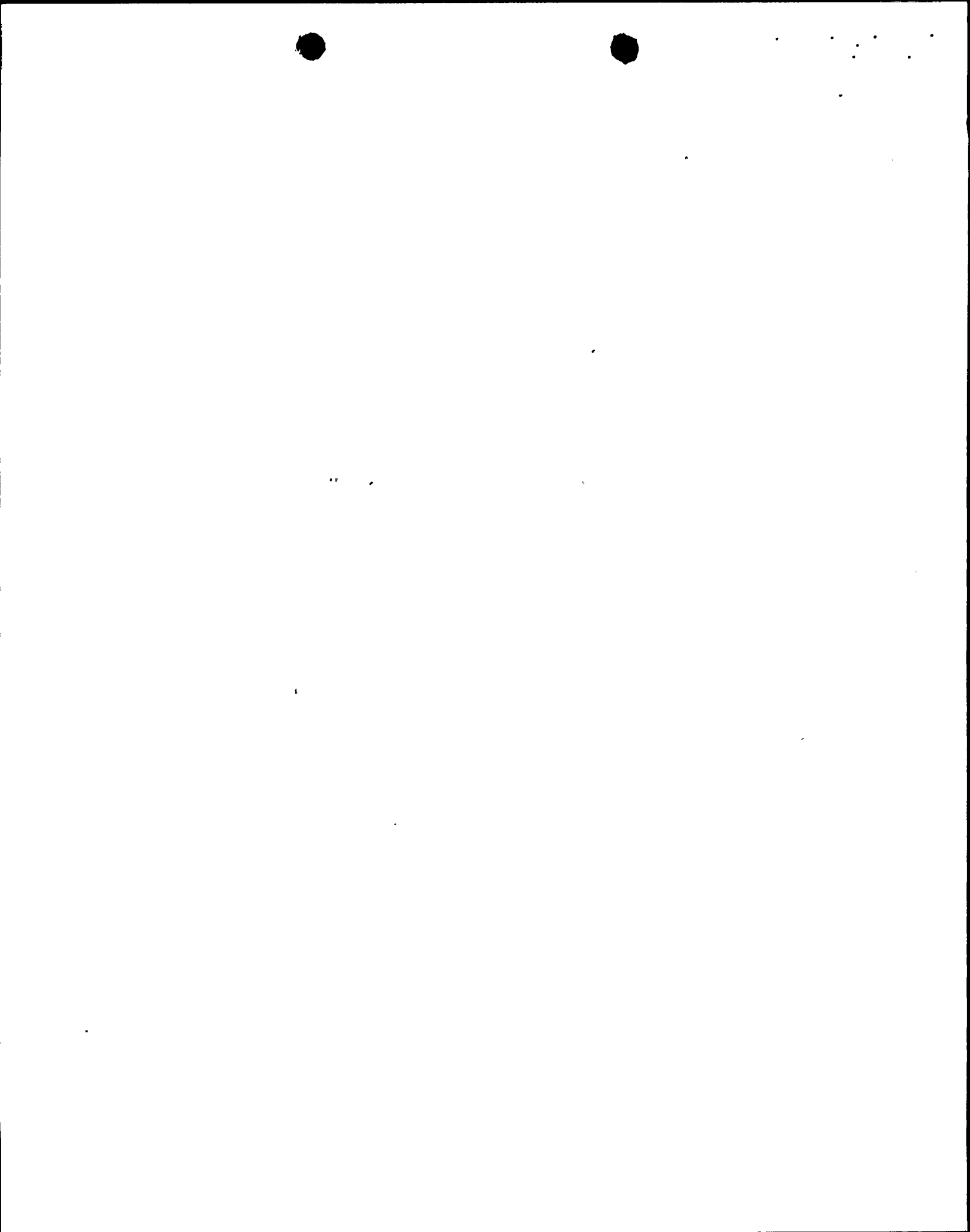
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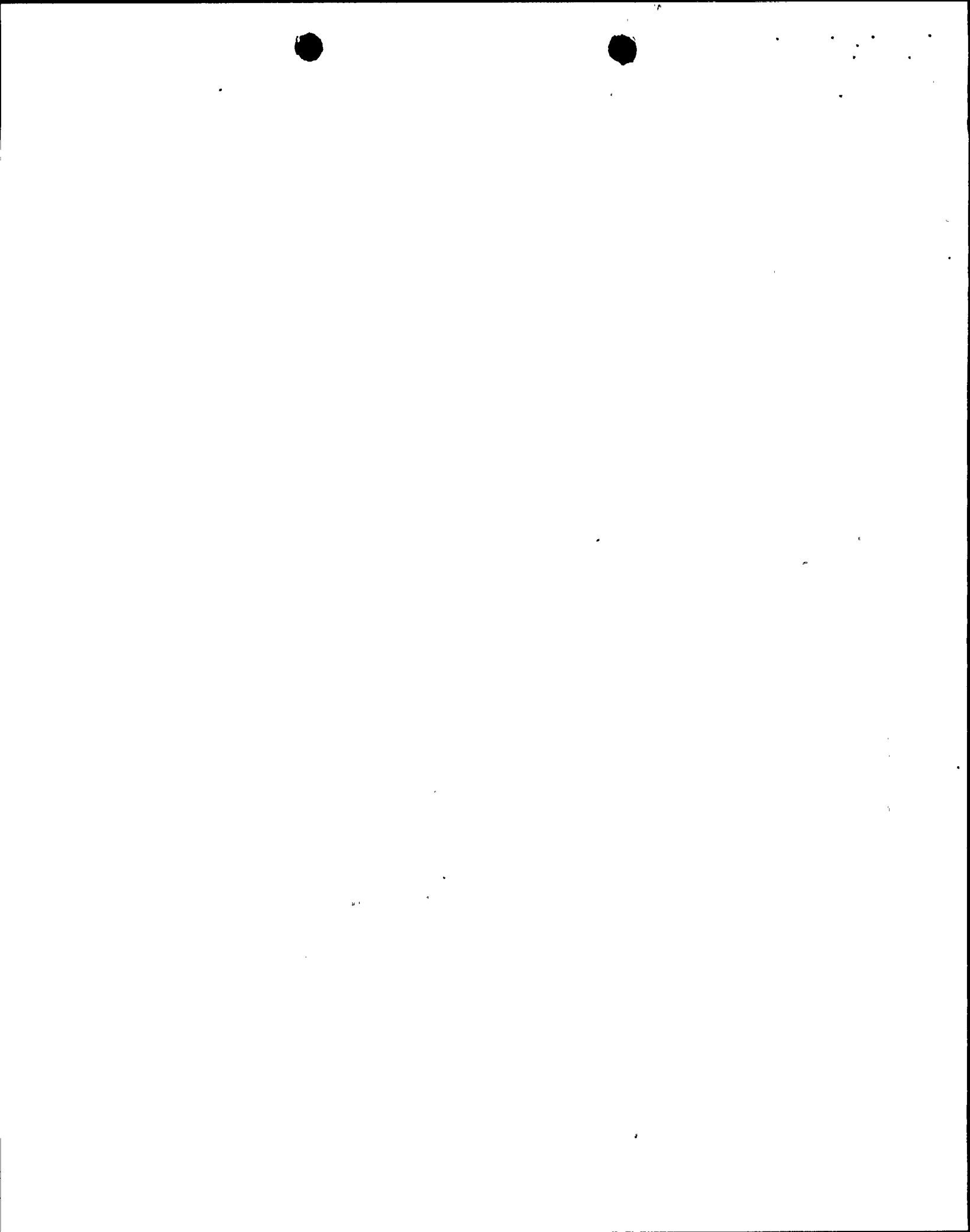
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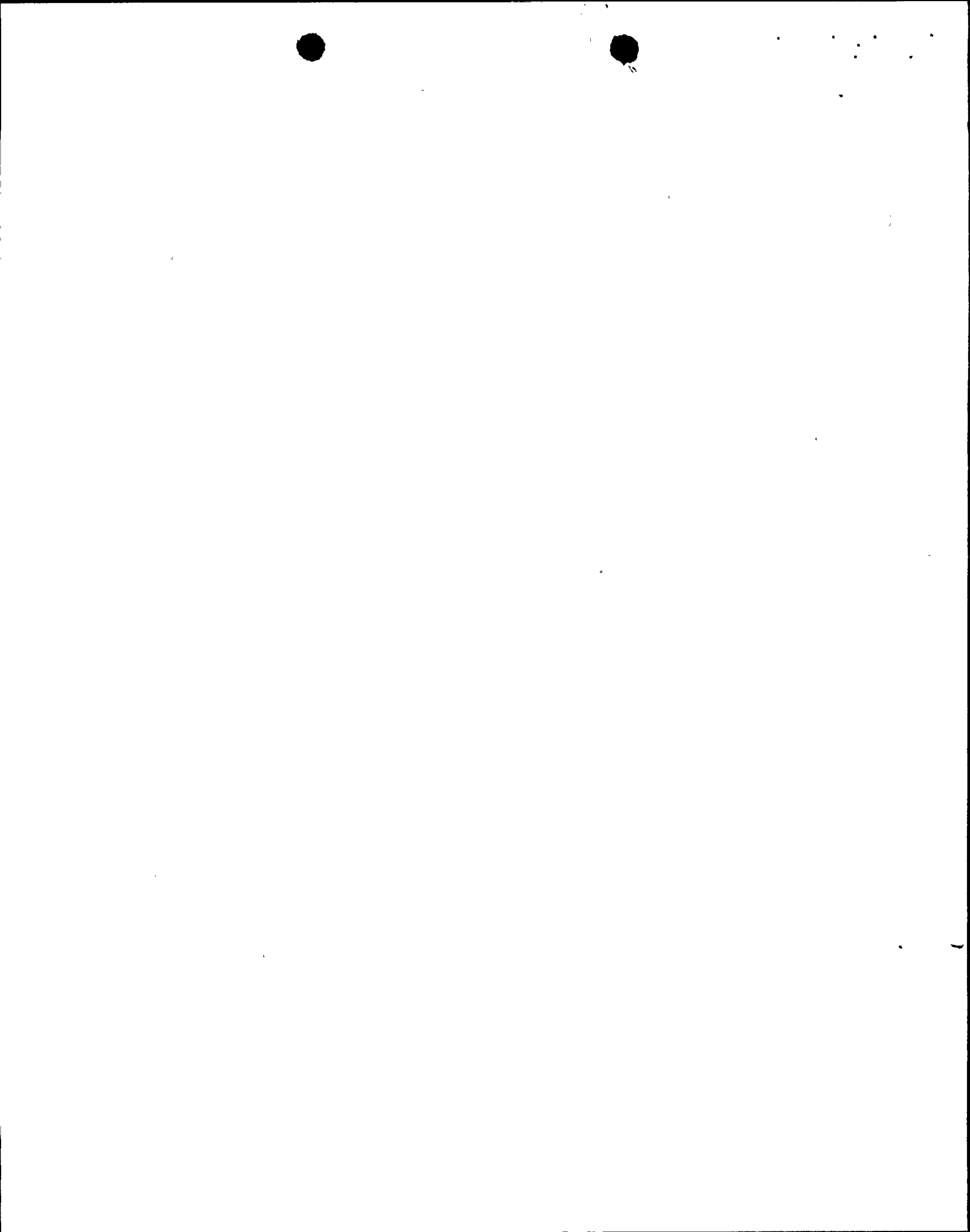
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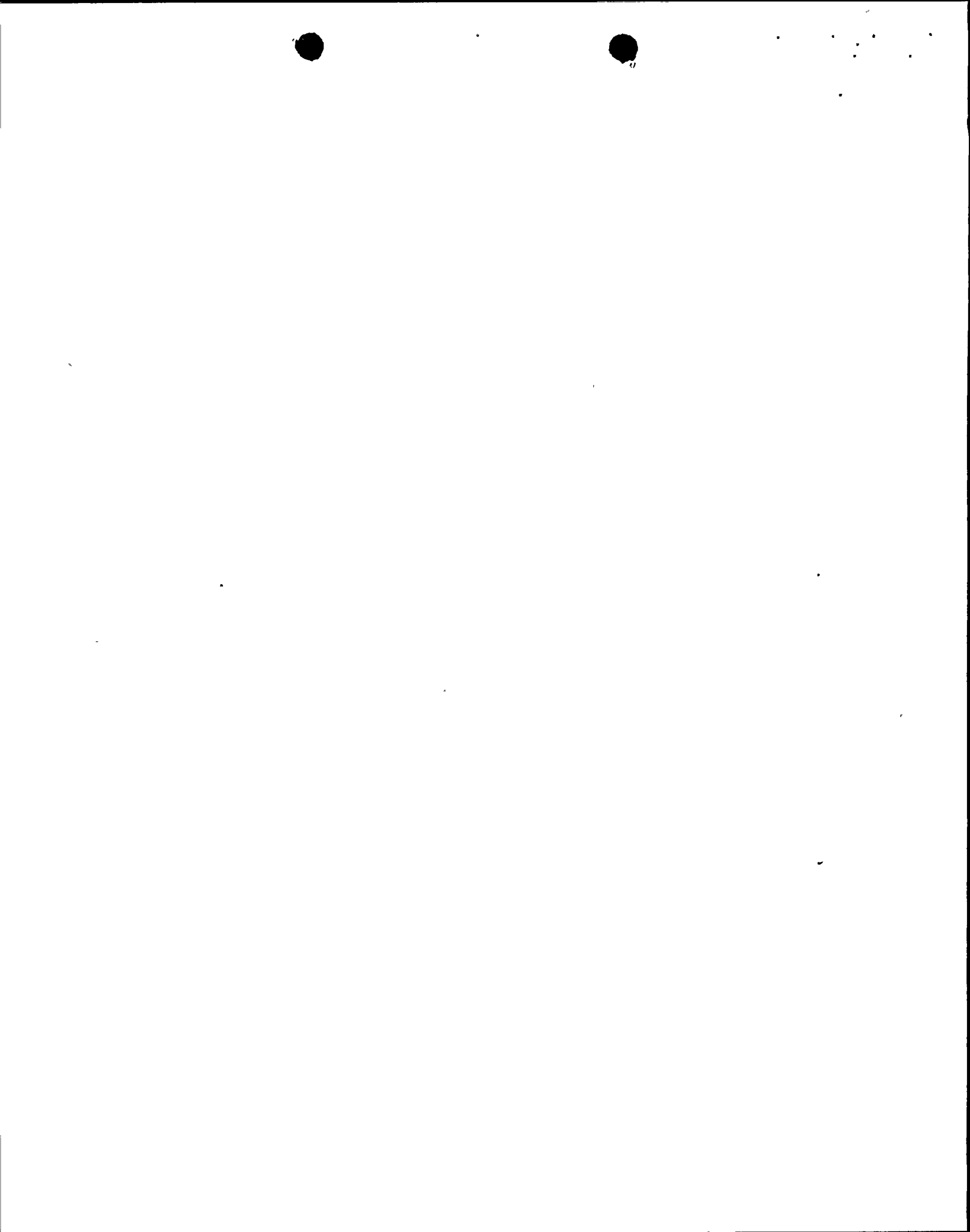
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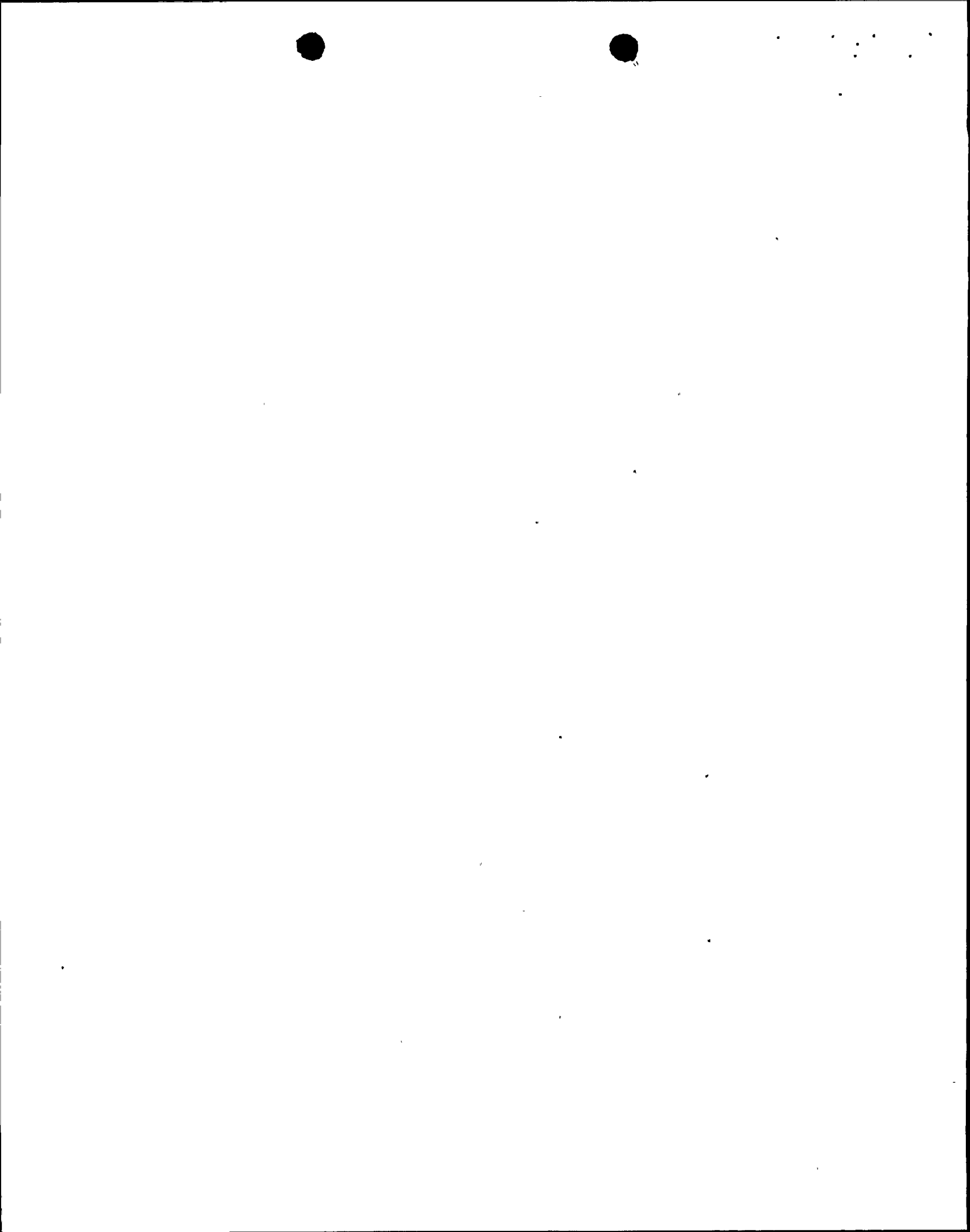
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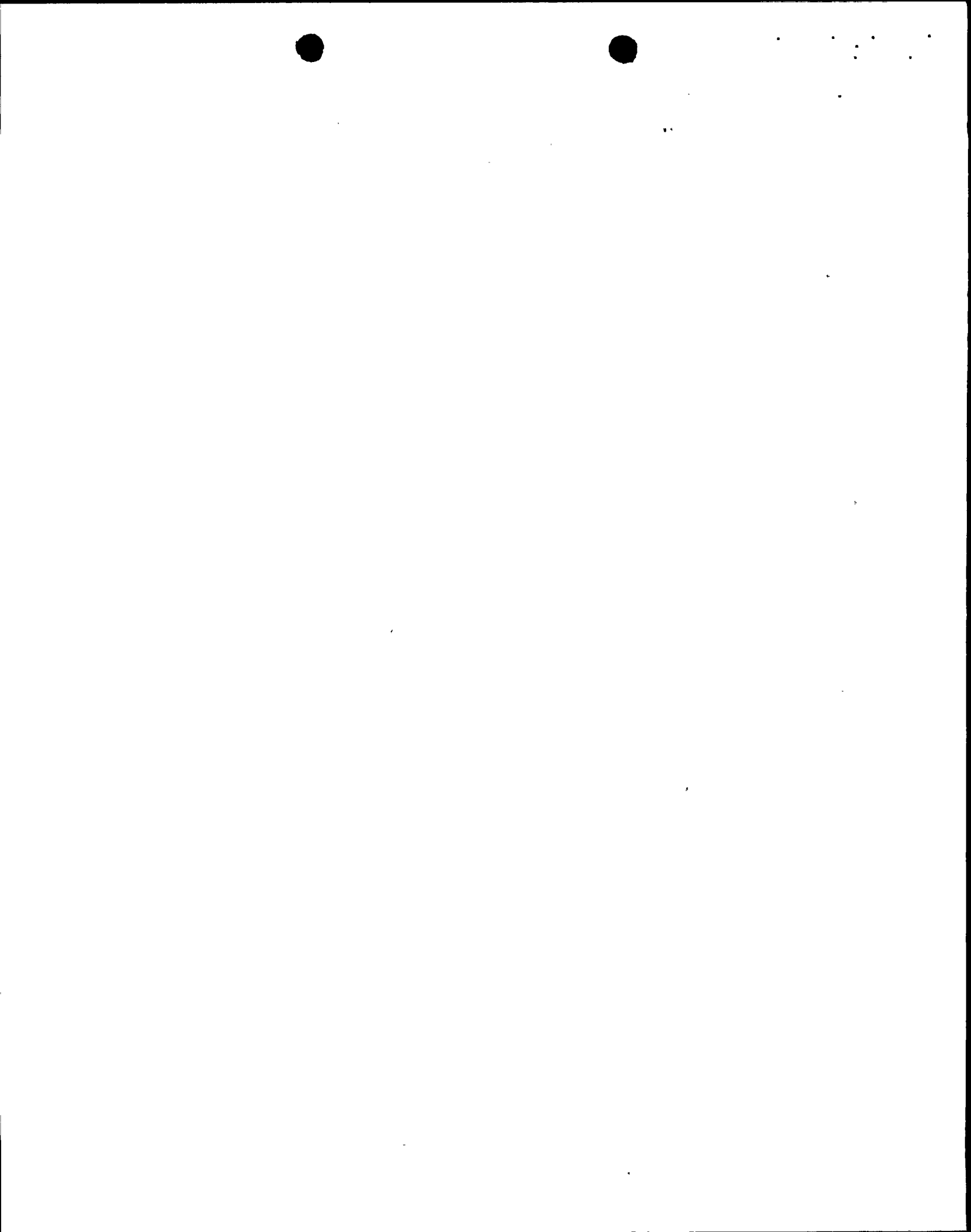
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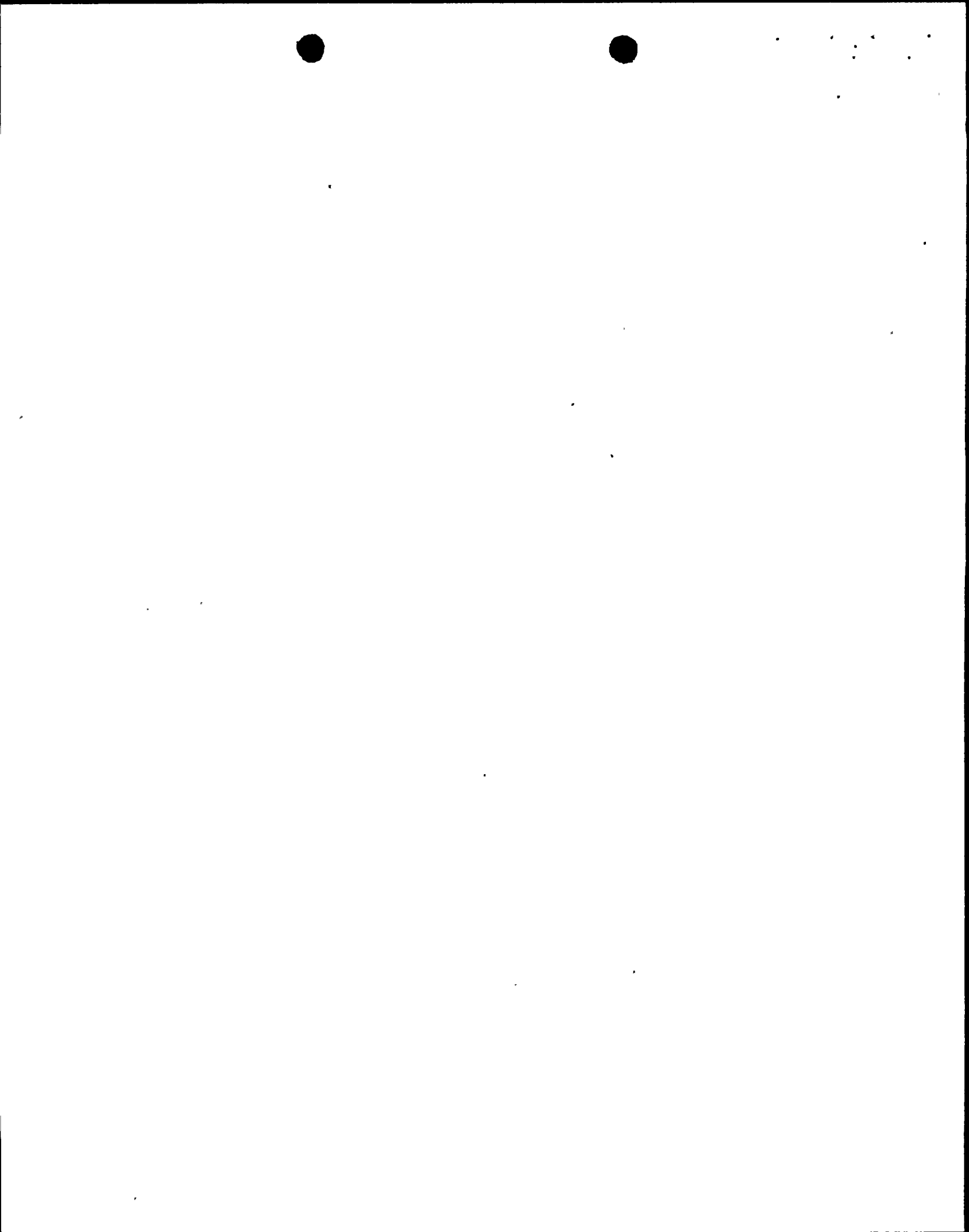
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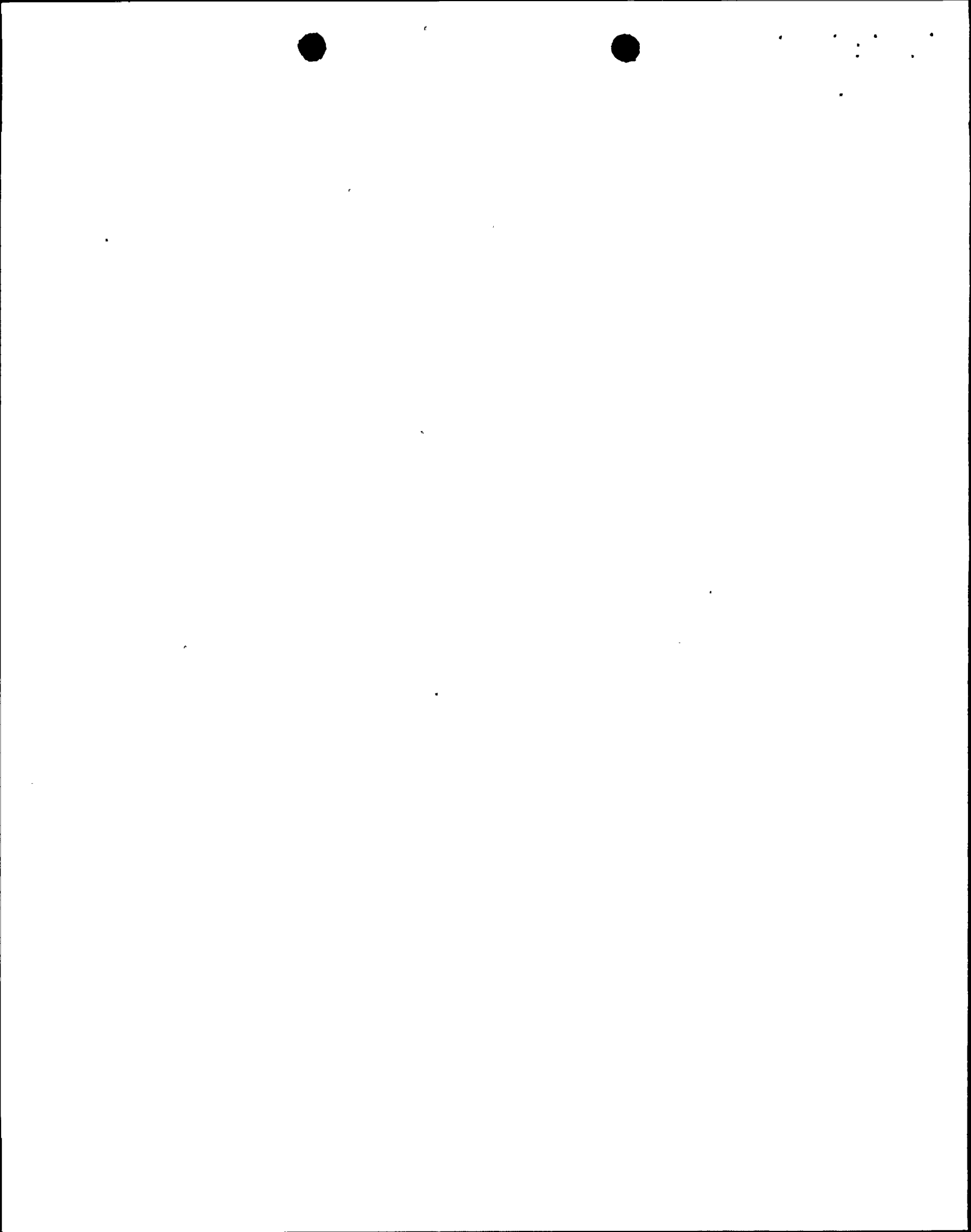
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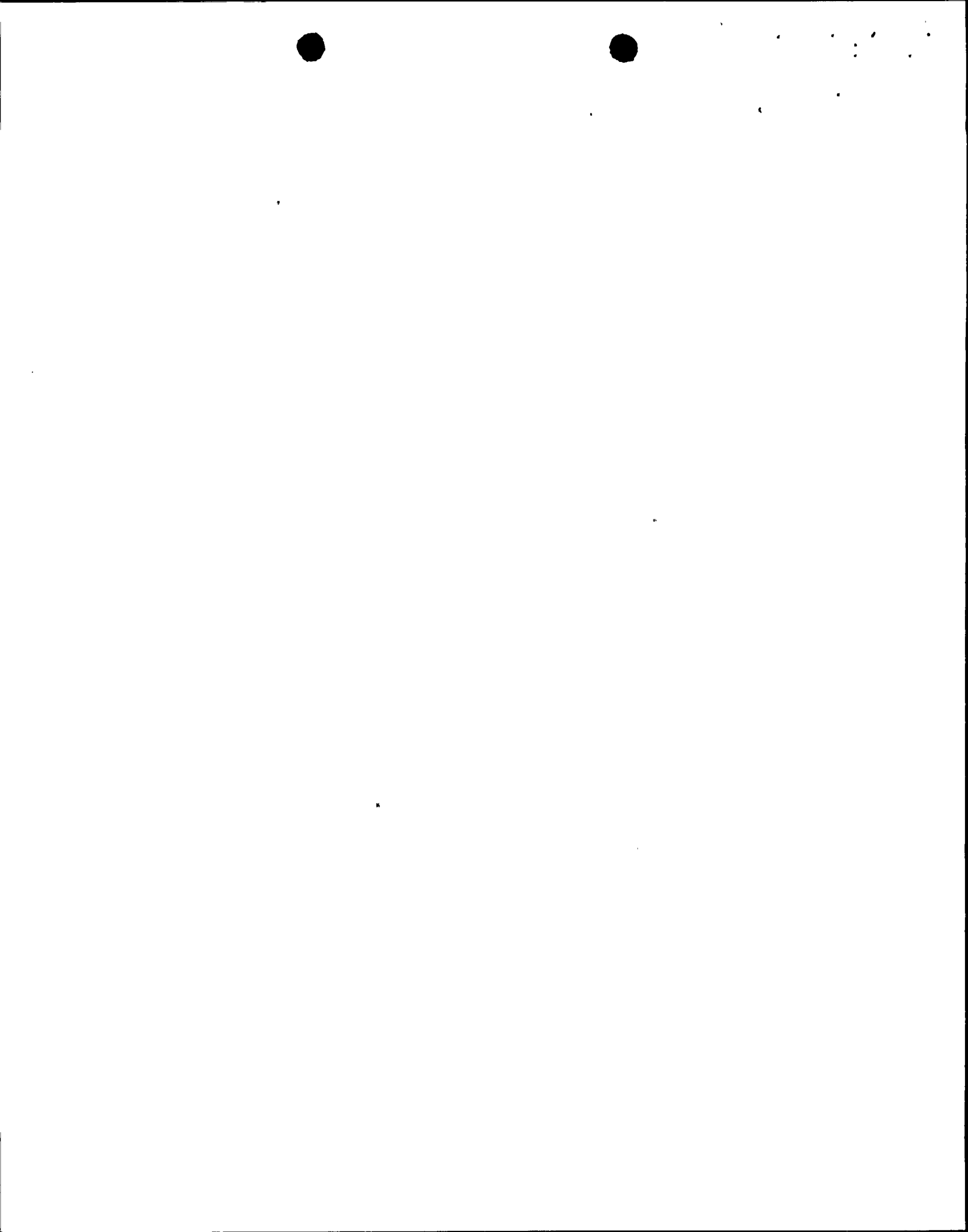
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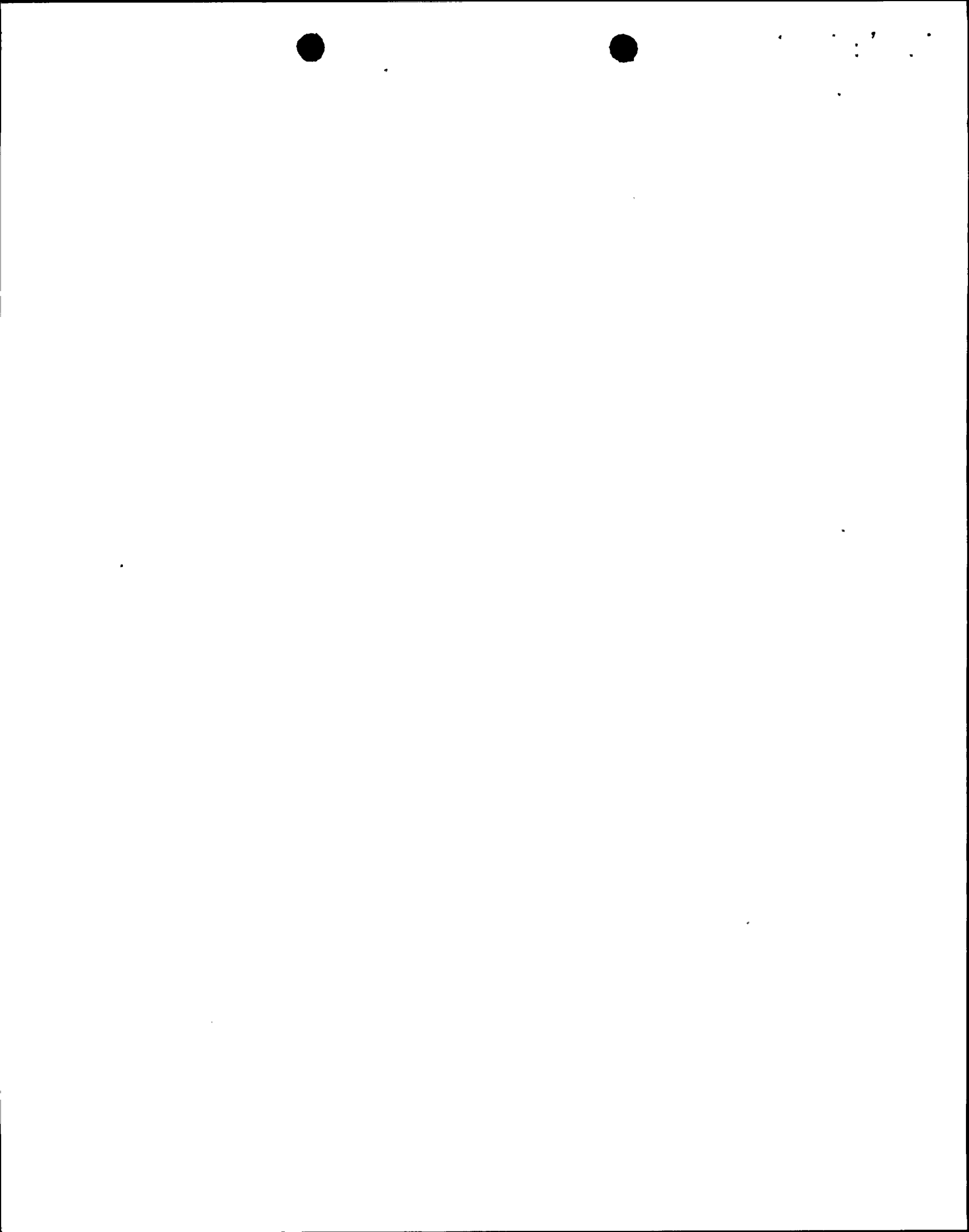
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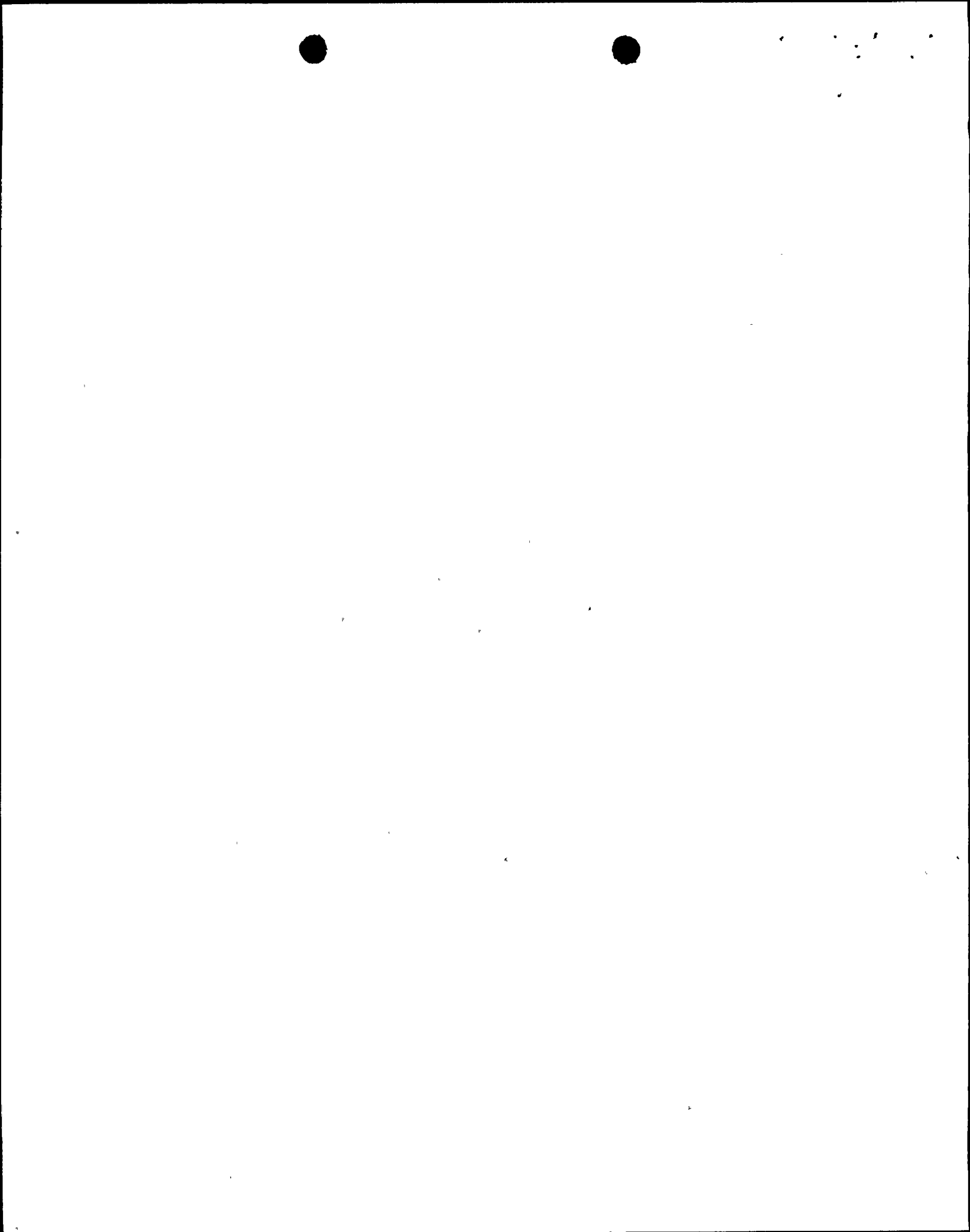


- b. Safety Class 1 Inservice Inspection, Specification 4.2.6 (Three months)
- c. Safety Class 2 Inservice Inspections, Specification 4.2.6 (Three months)
- d. Safety Class 3 Inservice Inspections; Specification 4.2.6 (Three months)
- e. Primary Containment Leakage Testing, Specification 3.3.3 (Three months)
- f. Secondary Containment Leakage Testing, Specification 3.4.1 (Three months)
- g. Sealed Source Leakage In Excess Of Limits, Specification 3.6.5.2 (Three months)

6.10 Record Retention

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. REPORTABLE OCCURRENCE Reports.



ATTACHMENT C

NIAGARA MOHAWK POWER CORPORATION

LICENSE NO. DPR-63

DOCKET NO. 50-220

Amendment Classification

This proposed amendment to the Operating License has been evaluated and determined to fall within the definition of Class II of 10CFR170.22 requiring a fee of \$1,200.



ATTACHMENT B

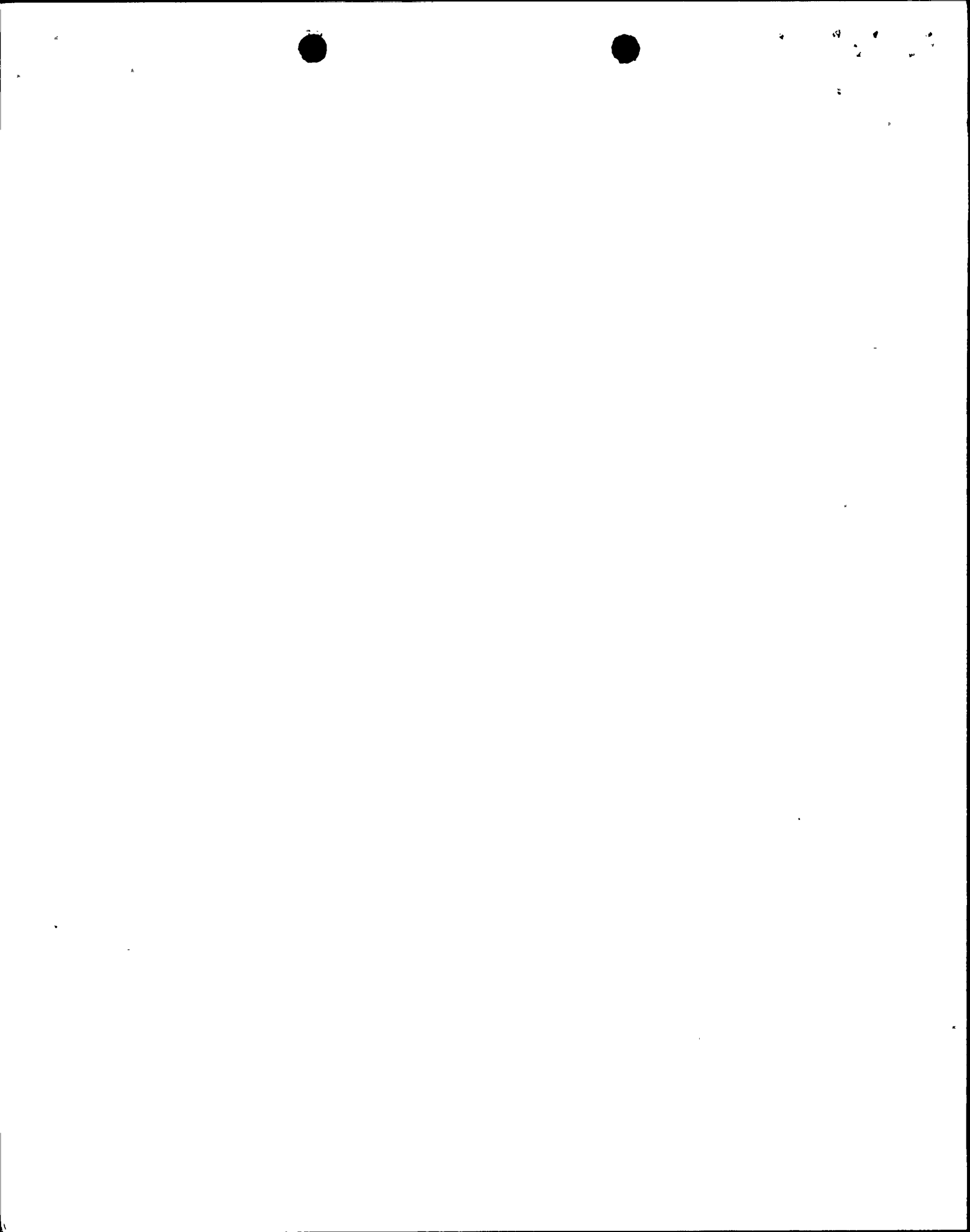
NIAGARA MOHAWK POWER CORPORATION

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Supporting Information

This revision changes the section of the Technical Specifications pertaining to the inservice inspection program and incorporates information not formerly included on the augmented inservice inspection and inservice testing programs. The enclosed Technical Specifications conform with 10CFR55(a),(g) and more closely matches the Standard Technical Specifications. The proposed Technical Specifications are general enough to permit changes in accordance with Section XI of the ASME Boiler and Pressure Vessel Code without a corresponding Technical Specification change. This submittal does not affect, or change, the present program being implemented by Niagara Mohawk. Changes to the inservice inspection or inservice testing programs must receive prior approval from the Nuclear Regulatory Commission.



ATTACHMENT D

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No Significant Hazards Considerations Analysis

The proposed Technical Specification change for the inservice inspection program involves no significant hazard considerations. Therefore, the operation of the Nine Mile Point Unit 1 in accordance with the proposed amendment will not 1) involve a significant increase in the probability or consequences of an accident previously evaluated, 2) create the possibility of a new or different kind of accident from any accident previously evaluated, or 3) involve a significant reduction in a margin of safety. This determination is based on the following analysis.

The proposed amendment changes the section of the Technical Specifications pertaining to the inservice inspection and incorporates information not formerly included on the augmented inservice inspection and inservice testing programs. The enclosed Technical Specifications conform with 10CFR55(a)(g) and more closely matches the Standard Technical Specifications. The Technical Specifications changes are general enough to permit changes in accordance with Section XI of the ASME Boiler and Pressure Vessel Code without a corresponding Technical Specification change. This submittal does not affect, or change, the present programs being implemented by Niagara Mohawk. Changes to the inservice inspection or inservice testing programs must receive prior approval from the Nuclear Regulatory Commission.

The Commission has provided guidance concerning the application of these standards by providing certain examples (48FR14870) of amendments considered not likely to involve significant hazards considerations. One of the examples (vii) relates to a change made to a license to conform to changes in the regulation where the license change results in very minor changes to facility operations clearly in keeping with the regulations. The proposed Technical Specifications change is similar to this example because the change conforms to 10CFR55(a)(g) and does not change the inservice inspection or inservice testing programs now in effect. Another example (ii), site changes that constitute an additional limitation, restriction or control not presently included in the Technical Specifications. The proposed Technical Specifications includes information on the augmented inservice inspection and inservice testing programs which are not presently described in the Technical Specifications.

