

UNITED STATES
NUCLEAR REGULATORY COMMISSION

In the Matter of)
Niagara Mohawk Power Corporation)
(Nine Mile Point Nuclear Station)
Unit No. 1

Docket No. 50-220

APPLICATION FOR AMENDMENT
TO

OPERATING LICENSE

Pursuant to Section 50.90 of the regulations of the Nuclear Regulatory Commission, Niagara Mohawk Power Corporation, holder of Facility Operating License No. DPR-63, hereby requests that Section 6.9 of the Technical Specifications set forth in Appendix A to that License be amended. This proposed change has been reviewed by the Site Operations Review Committee and the Safety Review and Audit Board.

The proposed Technical Specification change is set forth in Attachment A to this application. Supporting information, which demonstrates that the proposed change will have no safety or environmental significance, is set forth in Attachment B. The proposed change would not authorize any changes in the types of effluents or in the authorized power level of the facility. Justification for classification of the amendment pursuant to 10CFR Section 170.22 is included as Attachment C. An analysis, which demonstrates that the proposed change involves no significant hazards considerations pursuant to 10CFR50.92, is included as Attachment D.

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1. *Chlorophyll a* and *Chlorophyll b* were determined by the method of Arar and Collins (1971).

1. The first step is to identify the problem or question that needs to be answered. This involves understanding the context and the specific requirements of the task.

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1. The first of these is the fact that the

WHEREFORE, Applicant respectfully request that Appendix A to Facility Operating License No. DPR-63 be amended in the form attached hereto as Attachment A.

NIAGARA MOHAWK POWER CORPORATION

By: *C. Managan*
Vice President
Nuclear Engineering and Licensing

Subscribed and sworn to before

me on this 6th day of October, 1983

Gail M. Ahern

GAIL M. AHERN
Notary Public in the State of New York
Qualified in Onondaga County
My Commission Expires March 30, 1985

THESE DOCUMENTS SONT EN PARTIE D'UN
PROJET DE LOI SUR LA PROTECTION
DE L'ENVIRONNEMENT

LE 15 MARS 1974

LE 15 MARS 1974
LE 15 MARS 1974

LE 15 MARS 1974

LE 15 MARS 1974

LE 15 MARS 1974

LE 15 MARS 1974

ATTACHMENT A

NIAGARA MOHAWK POWER CORPORATION

LICENSE NO. DPR-63

DOCKET NO. 50-220

Proposed Changes to Technical Specifications (Appendix A)

Replace page 257f with the attached revised page. This page has been retyped in its entirety and the marginal markings indicate changes to the text.

1. The first part of the report is a summary of the work done during the year.

2. The second part is a detailed account of the work done during the year.

3. The third part is a summary of the work done during the year.

4. The fourth part is a summary of the work done during the year.

5. The fifth part is a summary of the work done during the year.

- (1) Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.
- (2) Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

Note: Routine surveillance testing, instrument calibration, or preventative maintenance which require system configurations as described in items 2.b(1) and 2.b(2) need not be reported except where test results themselves reveal a degraded mode as described above.

- (3) Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.
- (4) Abnormal degradation of systems other than those specified in item 2.a(3) above designed to contain radioactive material resulting from the fission process.

Note: Sealed sources or calibration sources are not included under this item. Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

6.9.3 Unique Reporting Requirements

Special reports shall be submitted to the Director of Regulatory Operations Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

- a. Reactor Vessel Material Surveillance Specimen Examination, Specification 4.2.2(c) (12 months)

ATTACHMENT B

NIAGARA MOHAWK POWER CORPORATION

LICENSE NO. DPR-63

DOCKET NO. 50-220

Supporting Information

This revision incorporates the current reporting requirements for reactor vessel material surveillance specimen in accordance with the recent revision to 10CFR50 Appendix H (Federal Register, Vol. 48, No. 104, May 27, 1983).



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ATTACHMENT C

NIAGARA MOHAWK POWER CORPORATION

LICENSE NO. DPR-63

DOCKET NO. 50-220

Amendment Classification

This proposed amendment to the Operating License has been evaluated and determined to fall within the definition of Class II of 10CFR170.22 thereby requiring a fee of twelve hundred dollars (\$1,200). A check for this amount is provided with this submittal.

THE UNIVERSITY OF CHICAGO

PHYSICS DEPARTMENT

CHICAGO, ILL.

January 10, 1950

Dear Mr. Tamm: I have just received your letter of January 7, 1950, regarding the proposed experiment on the production of muons in the atmosphere. I am sorry that I cannot give you a more definite answer at this time, but I am sure that you will understand the need for further study of this problem.

ATTACHMENT D

NIAGARA MOHAWK POWER CORPORATION

LICENSE NO. DPR-63

DOCKET NO. 50-220

No Significant Hazard Considerations Analysis

The proposed Technical Specification change to the reporting requirement for vessel specimen involves no significant hazard considerations. Therefore, the operation of Nine Mile Point Unit 1 in accordance with the proposed amendment will not 1) involve a significant increase in the probability or consequences of an accident previously evaluated, 2) create the possibility of a new or different kind of accident from any accident previously evaluated or, 3) involve a significant reduction in a margin of safety. This determination is based on the following analysis.

The proposed amendment incorporates the current reporting requirements for reactor vessel material surveillance specimen in accordance with the recent revision to 10CFR50 Appendix H. This proposed determination is supported by the fact that the requested action corresponds with example (vii) of the Sholly Rule published in the Federal Register on April 6, 1983, in that the change was made to conform to changes in the regulations.

SECRET

CONFIDENTIAL - SECURITY INFORMATION

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