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 LEMPGES, T.E. Niagara Mohawk Power Corp.
 RECIP. NAME: RECIPIENT AFFILIATION
 VASSALO, D.B. Operating Reactors Branch 2

SUBJECT: Forwards revised sys/component evaluation work sheets & addl justification for continued operation per 820302 transmittal re environ qualifications of safety-related electrical equipment.

SEE REPTS.

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 TITLE: Equipment Qualification (OR & PRE-OL)

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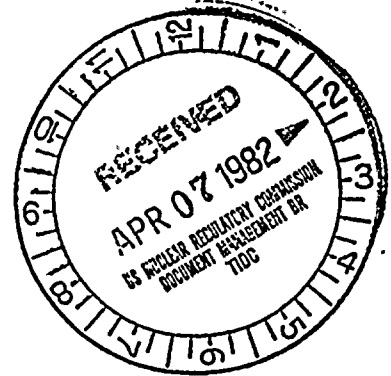
THE UNITED STATES OF AMERICA
DEPARTMENT OF THE INTERIOR
BUREAU OF LAND MANAGEMENT

WYOMING
COUNTY OF TETON
TETON NATIONAL MONUMENT

BY THE SECRETARY OF THE INTERIOR
IN RESPONSE TO A PETITION
FILED FOR RECORD IN THE
OFFICE OF THE SECRETARY OF THE INTERIOR
AT WASHINGTON, D. C.
ON APRIL 15, 1964
BY THE TETON NATIONAL MONUMENT
COMMISSIONERS

April 2, 1982

Mr. Dominic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing
Office of Nuclear Regulatory Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555



Re: Nine Mile Point Unit 1
Docket No. 50-220
DPR-63

Dear Mr. Vassallo:

Our letter of March 2, 1982 transmitted information regarding environmental qualifications of safety related electrical equipment. In that letter, we indicated revised system/component evaluation work sheets and additional justification for continued operation, if required, would be submitted. Contained herein are those revised work sheets and additional justification for continued operation.

Very truly yours,

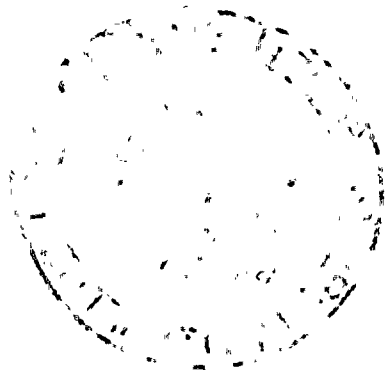
NIAGARA MOHAWK POWER CORPORATION

A handwritten signature in cursive script that reads "Thomas E. Lempges".

Thomas E. Lempges
Vice President - Nuclear Generation

RJP:ja

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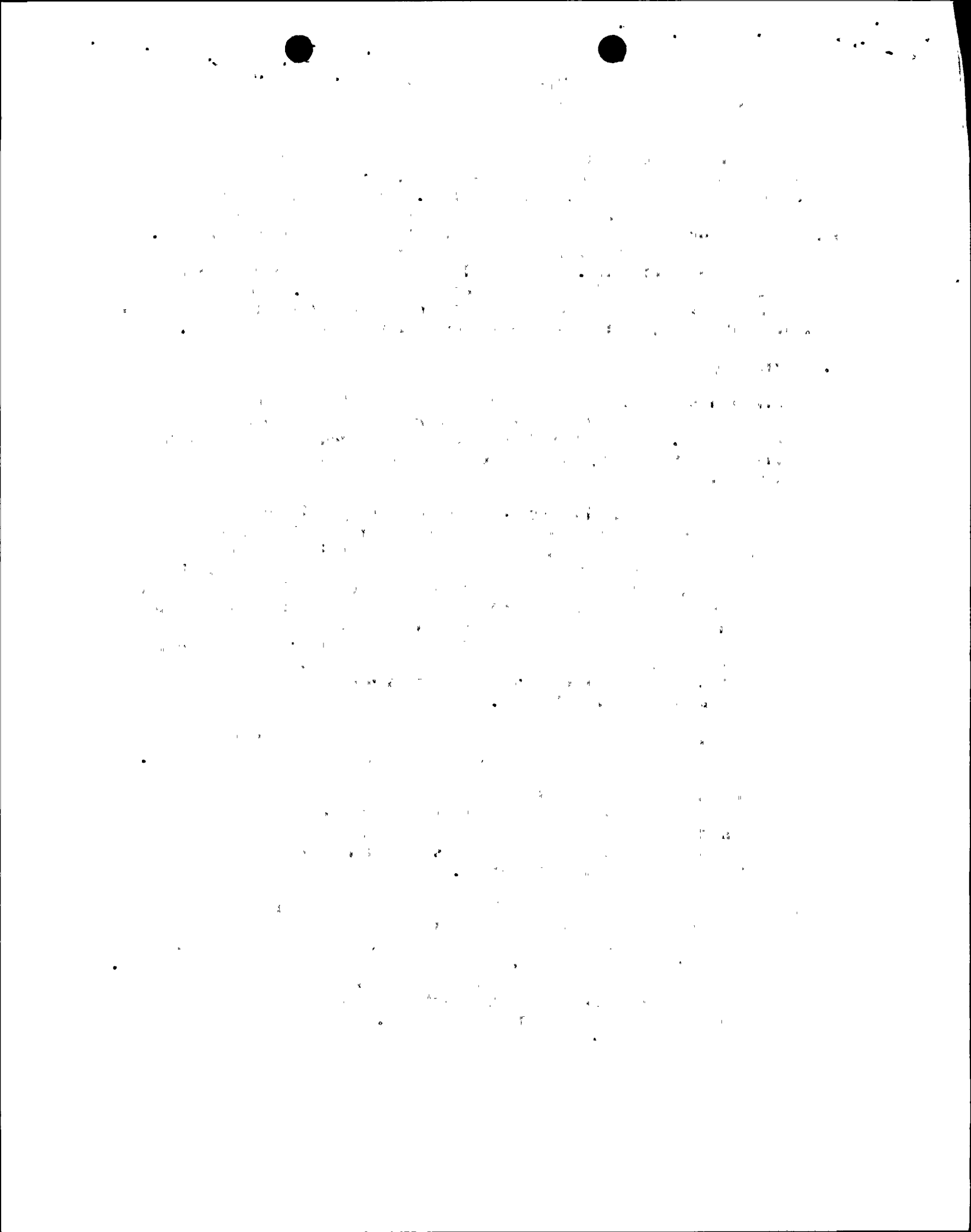
Additional Information
Nine Mile Point Unit 1
Equipment Qualification

The NRC staff's Safety Evaluation Report (SER) on the environmental qualification of safety related electrical equipment at Nine Mile Point Unit 1 was transmitted by letter dated June 8, 1981. Niagara Mohawk provided by letter dated September 9, 1981 a response to the SER which included additional information, corrective action and justification for continued operation. Subsequent to this response, analysis has continued with respect to environmental qualification. As a result of this additional analysis, revisions to our previously submitted data have been made. The purpose of this submittal is to provide revised system component evaluation work sheets and, if appropriate, additional justification for continued operation.

1.0 Revised System Component Evaluation Work Sheets (SCEW sheets)

Individual system component evaluation work sheets have been revised to reflect data included in evaluation reports provided by our letter of March 2, 1982. The previously submitted evaluations and the enclosed revised work sheets may be subject to further revisions based on the following:

- (a) Integrated Radiation Dose - Values currently being specified on system components evaluation work sheets for radiation dose qualifications are based on integrated doses in excess of the dose the equipment may receive for its required operating time. For example, equipment specified on the SCEW sheets with operating times of one hour are being evaluated for radiation qualification based on an integrated dose for 100 days or 180 days, depending where the equipment is located outside or inside the primary containment. Further analysis using integrated doses associated with the required operating times will reduce the radiation qualification limits by several orders of magnitude.
- (b) Normal Drywell Operating Temperature - Aging analyses performed to date have been based on a maximum operating temperature of 103°F. While this value is conservative with respect to equipment located within the reactor building, it may not accurately reflect the temperatures existing at various levels within the primary containment. Preliminary analysis at an assumed normal ambient drywell temperature of 135°F has shown minimal impact on the evaluation previously submitted.
- (c) Specific Area Dose Rates - For equipment located in the Reactor Building, general area radiation levels were specified. In some cases, a more detailed review with respect to equipment location may raise or lower the dose rates to which the equipment may be subject. This, coupled with the integrated doses assumed as discussed above, may result in revision to our existing evaluation and associated system component evaluation work sheets.

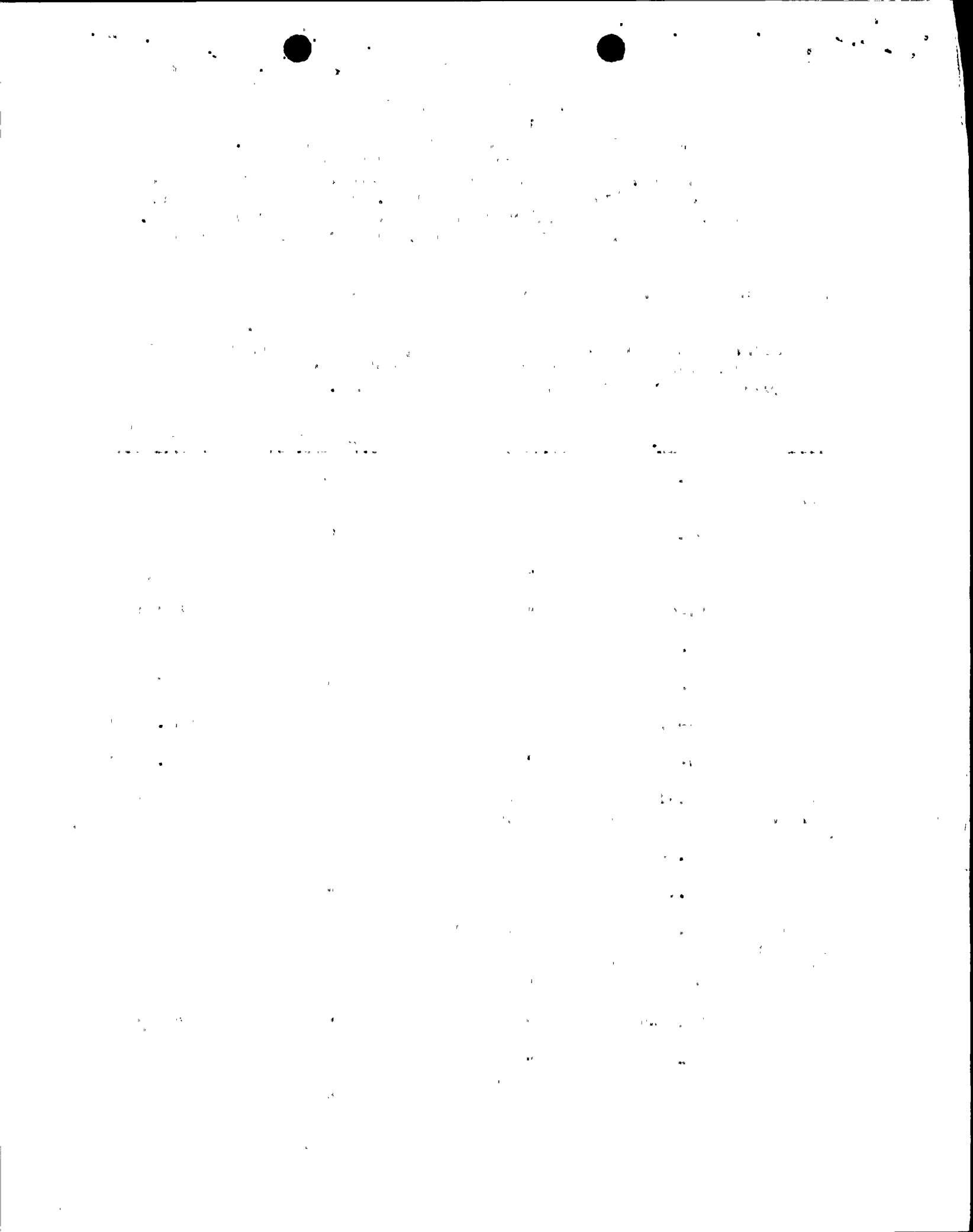


- (d) Thermal Aging - Analysis for thermal aging involved use of Arrhenius techniques assuming either the maximum harsh environment temperature for the specified life of the equipment or the ambient temperature for the specified life in combination with the specified maximum temperature for the duration of the design basis event. Where ambient temperatures were used, the maximum expected temperature was used the entire time rather than an average temperature based on seasonal variations or plant operations. The results presented, therefore, are conservative with respect to aging considerations. Revisions to our existing evaluations and associated SCEW sheets may be warranted.

2.0 Additional Justification for Continued Operation

The following is a list of those components that have changed in qualification status from our September 9, 1981 submittal. Only those components that have been changed from completely qualified to assessment ongoing due to an outstanding item are identified.

<u>System</u>	<u>ID No.</u>	<u>Component</u>	<u>Manufacturer</u>	<u>Outstanding Item</u>
Reactor Isolation	01-03	Position Switch	Namco	Aging
	01-04	"	"	Aging
	01-05	"	"	Aging
	01-06	"	"	Aging
	39-05	"	"	Aging
	39-06	"	"	Aging
	01-01	"	"	Rad., Aging
	01-02	"	"	Rad., Aging
Containment Isolation - Drywell	201-10	Position Switch	Namco	Aging
	83.1-10	"	"	Aging
	83.1-12	"	"	Aging
Containment Isolation - Torus	201-08	Position Switch	Namco	Aging
	201-16	"	"	Aging
	201.2-06	"	"	Aging
	68-08	"	"	Aging
	68-09	"	"	Aging



<u>System</u>	<u>ID No.</u>	<u>Component</u>	<u>Manufacturer</u>	<u>Outstanding Item</u>
	68-10	"	"	Aging
Reactor Vessel	36-03 A-D	Level XMTR	Rosemount	All
Instrumentation	36-04 A-D	" "	"	All
	36-05 A-D	" "	"	All
	36-07 A-D	Press. XMTR	"	All
	36-08 A-D	" "	"	All
Containment Spray	201.2-476 A-D	Press. XMTR	Rosemount	All
Core Spray	RV-26 A-B	Flow XMTR	Rosemount	All
	58-05	Level XMTR	Rosemount	All
	58-06	Level SMTR	Rosemount	All
Automatic Depressurization System	ID 33 A	Flow XMTR	Rosemount	All
	ID 33 B	Flow XMTR	Rosemount	All
Reactor Bldg. Closed Loop Cooling	70-137	E/P XMTR	Fisher	Aging
Emergency Condensers	36-06 A-D	Press. XMTR	Rosemount	All
Common Electrical Equipment	N/A	5kV Quad Assembly	Kerite	All
Additional Instrumentation	01-26 A-H	D/P XMTR	Rosemount	All

Additional justification for continued operation are provided for the above identified components. These components were not addressed by our September 9, 1981 due to their fully qualified status at that time.



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