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 AUTH. NAME AUTHOR AFFILIATION
 DISE, D.P. Niagara Mohawk Power Corp.
 RECIP. NAME RECIPIENT AFFILIATION
 EISENHUT, D.G. Division of Licensing

SUBJECT: Forwards info providing status & clarification of NUREG-0737
 Action Plan Items II.B.3, "Post & Accident Sampling," II.F.1.3,
 "Containment High Radiation Monitor" & II.F.2,
 "Instrumentation for Detection of Inadequate Core Cooling."

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 TITLE: Response to NUREG -0737/NUREG-0660 TMI Action Plan Rgmts (OL's)

NOTES:

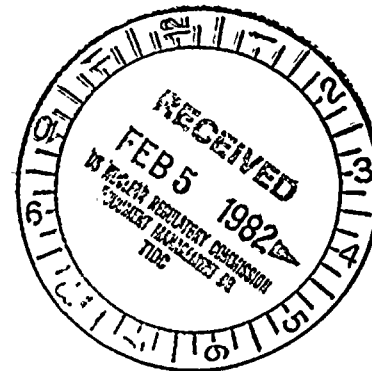
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INTERNAL:	ELD		1	0	IE	12	2	2
	IE/DEP DIR	33	1	1	IE/DEP/EPDB		1	1
	IE/DEP/EPLB		3	3	NRR/DE DIR	21	1	1
	NRR/DE/ADCSE	22	1	1	NRR/DE/ADMGE	23	1	1
	NRR/DE/ADSA	17	1	1	NRR/DHFS DIR	28	1	1
	NRR/DHFS/DEPY29		1	1	NRR/DL DIR	14	1	1
	NRR/DL/ADL	16	1	1	NRR/DL/ADOR	15	1	1
	NRR/DL/ORAB	18	3	3	NRR/DSI ADRS	27	1	1
	NRR/DSI DIR	24	1	1	NRR/DSI/ADGP	31	1	1
	NRR/DSI/ADPS	25	1	1	NRR/DSI/ADRP	26	1	1
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	NRR/DST/ADT	32	1	1	<u>REG FILE</u>	04	1	1
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	INPO, J. STARNES		1	1	LPDR	03	1	1
	NRC PDR	02	1	1	NSIC	05	1	1
	NTIS		1	1				

THE UNIVERSITY OF CHICAGO
DEPARTMENT OF CHEMISTRY
CHICAGO, ILLINOIS 60637

RECEIVED
MAY 15 1964
FROM
DR. J. H. GOLDSTEIN
AND
DR. R. A. FORTMAYOR

TO
DR. J. H. GOLDSTEIN
AND
DR. R. A. FORTMAYOR
DEPARTMENT OF CHEMISTRY
UNIVERSITY OF CHICAGO
CHICAGO, ILLINOIS 60637

February 1, 1982



Mr. Darrell G. Eisenhut, Director
Division of Licensing
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Re: Nine Mile Point Unit 1
Docket No. 50-220
DPR-63

Dear Mr. Eisenhut:

Our letter of December 31, 1981 provided the status of NUREG 0737 action plan items and indicated that several items were anticipated to be completed by January 31, 1982. The attached information provides a status of those items and outlines further clarification regarding their implementation.

Very truly yours,

NIAGARA MOHAWK POWER CORPORATION

D. P. Dise
Vice President Engineering

RJP:ja

Handwritten: A046
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PDR

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THE UNIVERSITY OF CHICAGO
DIVISION OF THE PHYSICAL SCIENCES
DEPARTMENT OF CHEMISTRY
5708 SOUTH CAMPUS DRIVE
CHICAGO, ILLINOIS

RECEIVED
JAN 10 1950

1950

TO THE DIRECTOR OF THE DIVISION OF THE PHYSICAL SCIENCES
FROM THE DEPARTMENT OF CHEMISTRY
RE: [Illegible]

[Illegible text]

[Illegible text]

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT UNIT 1

DOCKET NO. 50-220

DPR-63

STATUS AND CLARIFICATIONS

FOR

NUREG 0737 ACTION PLAN ITEMS

- II.B.3 - Post Accident Sampling
- II.F.1.3 - Containment High Radiation Monitor
- II.F.2 - Instrumentation for Detection of Inadequate Core Cooling



The first part of the document
 describes the general situation
 and the objectives of the study.
 The second part contains the
 results of the investigation.
 The third part discusses the
 implications of the findings.

The results of the study
 show that there is a significant
 difference between the two groups.
 This difference is statistically
 significant at the 5% level.

NUREG 0737 ACTION PLAN ITEMS
STATUS AND CLARIFICATION

II.B.3 Post Accident Sampling

The post accident sampling system has been installed and tested in accordance with our December 31, 1981 letter. However, the results of the pre-operational testing and the system operating procedures have not received final review and approval. It is anticipated that these reviews and approvals will be completed by February 15, 1982.

II.F.1.3 Containment High Radiation Monitor

Our letter of December 31, 1980 provided a description of the modifications to be performed to meet the requirements of NUREG 0737, Item II.F.1.3. These modifications have been completed in accordance with our previous letter, with the following clarifications.

Penetration Wall Thickness - We previously indicated that the monitors would be installed in spare penetration sleeves having a maximum wall thickness of approximately 1/4 inch. However, the monitors were installed in spare penetration sleeves which utilize 8 inch schedule 80 pipe with a wall thickness of 0.500 inch and 10 inch schedule 80 pipe with a wall thickness of 0.593 inch. As indicated in our letter of December 31, 1980, the in sleeve arrangement with the wall thickness noted above will perform their intended function since early indicators of fuel damage have high energy gammas and are not significantly affected by this attenuation.

In Situ Calibration - We previously indicated that the in situ calibration required by Table II.F.1-3 of Action Plan Item II.F.1.3 would be performed. However, since the plant was on line and drywell entry was not possible, in situ calibration was not possible. Therefore, mockup detector calibration sleeves were utilized to simulate in situ calibration. The mockup detector calibration sleeves were fabricated from 8 inch pipe with a nominal wall thickness of 0.510 inch and a 10 inch pipe with a nominal wall thickness of 0.540 inch.

Calibration - We previously indicated that calibration checks required by Table II.F.1-3 of Action Plan Item II.F.1.3 would be performed. Calibration testing was performed at 5, 13 and 21 R/hr by utilizing a known Co-60 source. No data point was taken in the range of 100-1000 R/hr due to ALARA considerations. Sleeve attenuation factors were also determined using the mockup calibration sleeves. The vendor also performed a calibration check of the detectors at 83 R/hr.

Based on the aforementioned, Niagara Mohawk has met its requirements regarding NUREG 0737, Item II.F.1.3.

II.F.2 Instrumentation for Detection of Inadequate Core Cooling

Our letter of February 9, 1981 provided a description of the modifications to be performed to meet the requirements of NUREG 0737, Item II.F.2. These modifications have been completed in accordance with our previous letter with the following clarification.

Our submittal indicated that a readout from the newly installed instrumentation would be available in the Technical Support Center via the process computer.

MEMORANDUM FOR THE RECORD

DATE: 10/15/54

RE: [Illegible text]

1. [Illegible text]

2. [Illegible text]

3. [Illegible text]

4. [Illegible text]

5. [Illegible text]

6. [Illegible text]

7. [Illegible text]

8. [Illegible text]

9. [Illegible text]

Necessary components to accomplish this function have had to be returned to the vendor for modifications. Upon receipt and installation of the modified components, this modification will be completed. In the interim, the existing Control Room color camera and Technical Support Center color camera monitor will be utilized to provide a readout of this information in the Technical Support Center.

Dear Mr. [Name],
I have your letter of the 10th and am sorry that I cannot
reply to you more quickly. I am sure that you will
understand that I am very busy at the moment.
I will try to get back to you as soon as possible.
Yours faithfully,
[Signature]