



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NIAGARA MOHAWK POWER CORPORATION

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 42
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Niagara Mohawk Power Corporation (the licensee) dated September 17, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 42, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This license amendment is effective as of the date of its issuance.

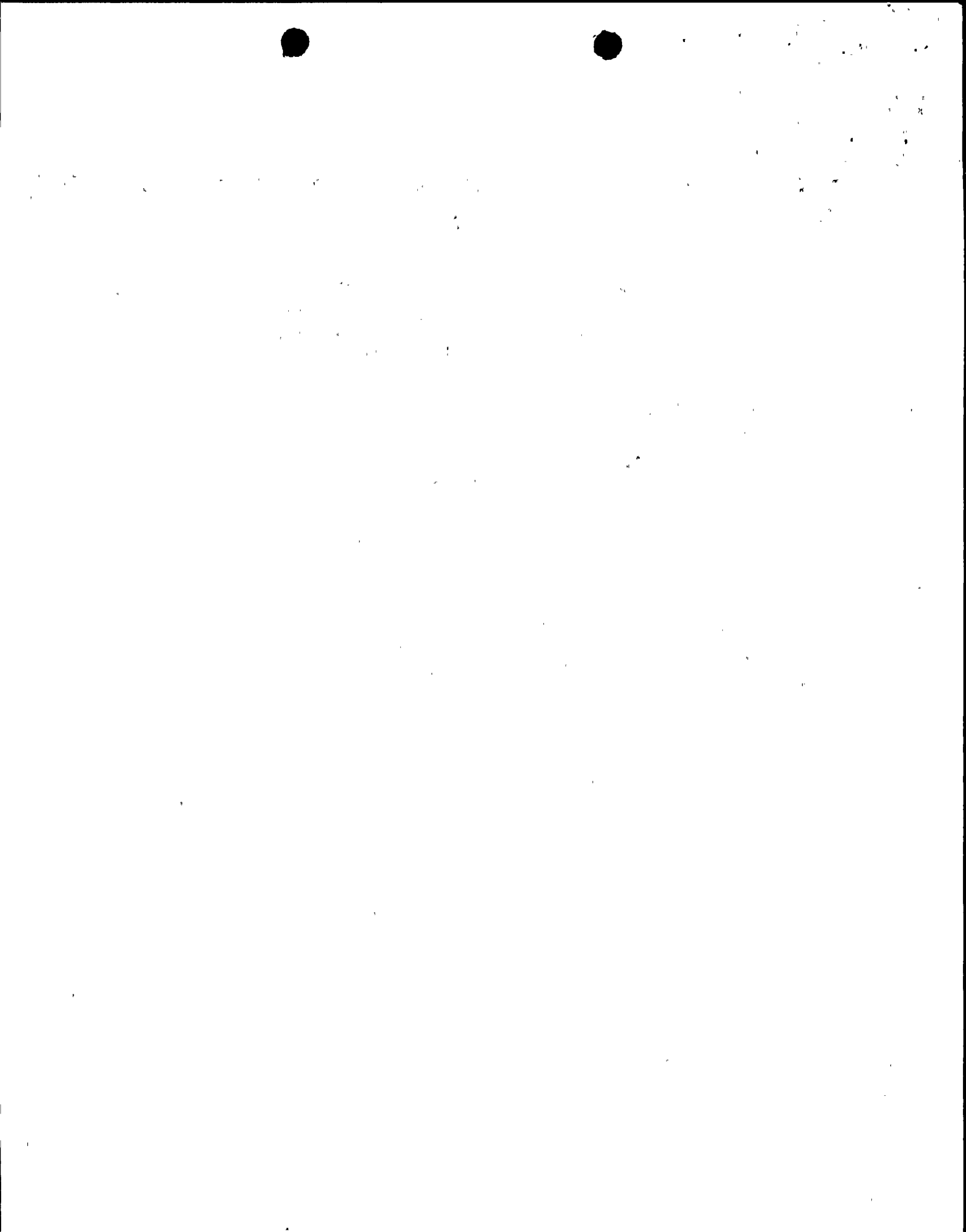
FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 13, 1981



ATTACHMENT TO LICENSE AMENDMENT NO. 42

FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Revise Appendix A as follows:

<u>Remove</u>	<u>Insert</u>
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--	24lff
--	24lgg
--	24lhh
--	24lii
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LIMITING CONDITION FOR OPERATION

3.6.11 ACCIDENT MONITORING INSTRUMENTATION

Applicability:

Applies to the operability of the plant instrumentation that performs an accident monitoring function.

Objective:

To assure high reliability of the accident monitoring instrumentation.

Specification:

- a. During the power operating condition, the accident monitoring instrumentation sensors shown in Table 3.6.11 shall be operable except as specified in 3.6.11 b or c.
- b. Safety and Relief Valves
 - (1) With the number of operable accident monitoring instrumentation sensors (for parameters 1 and 2) 1 less than the number shown in Table 3.6.11 restore to an operable status during the next cold shutdown when there is access to the drywell.
 - (2) With the number of operable accident monitoring instrumentation sensors (for parameters 1 and 2) 2 less than the number shown in Table 3.6.11 restore an inoperable sensor to an operable status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours.
 - (3) The total number of sensors shown in Table 3.6.11 will be operable prior to the beginning of each cycle.

SURVEILLANCE REQUIREMENT

4.6.11 ACCIDENT MONITORING INSTRUMENTATION

Applicability:

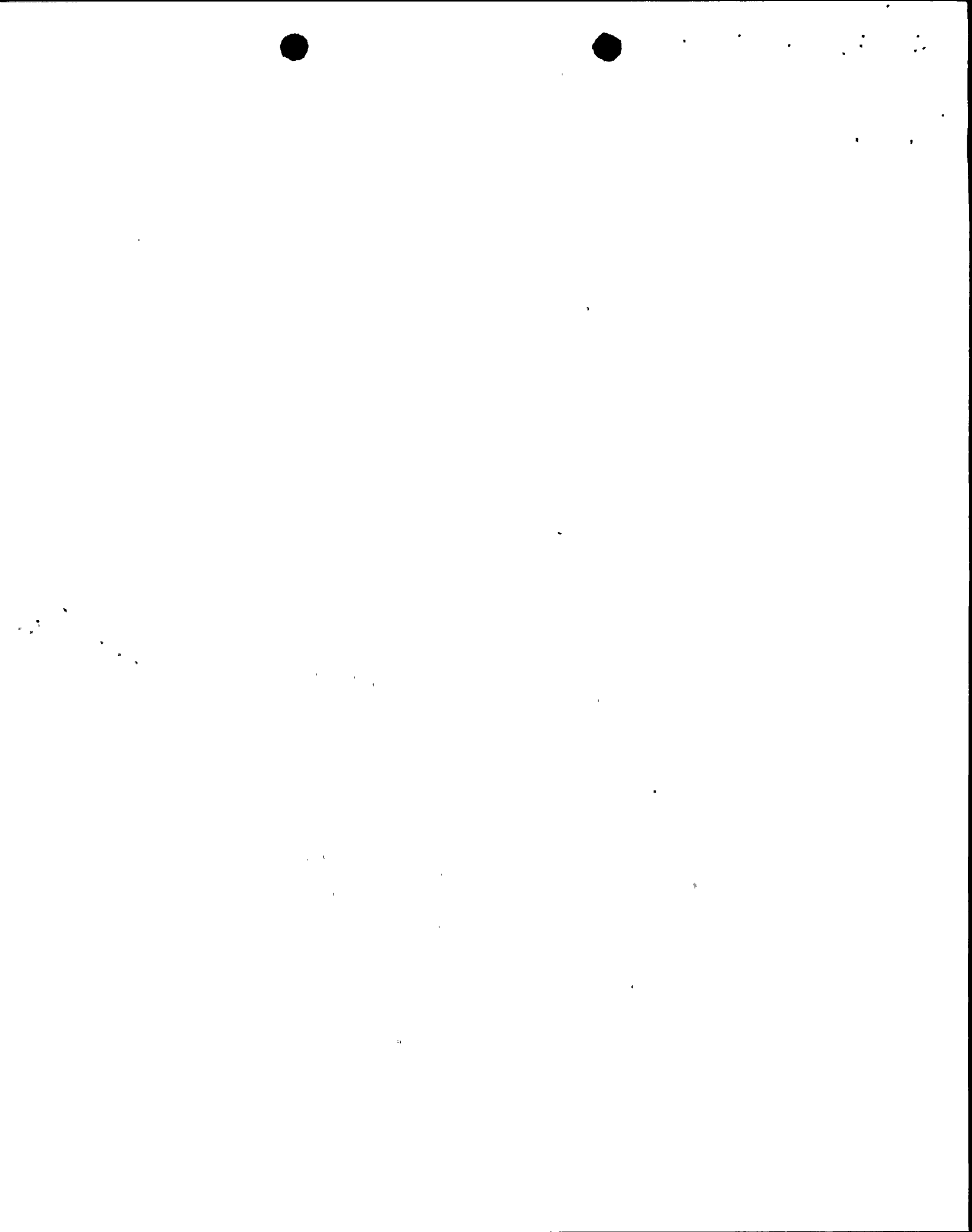
Applies to the surveillance of the instrumentation that performs an accident monitoring function.

Objective:

To verify the operability of accident monitoring instrumentation.

Specification:

Instrument channels shall be tested and calibrated at least as frequently as listed in Table 4.6.11.



LIMITING CONDITION FOR OPERATION

SURVEILLANCE REQUIREMENT

3.6.11 ACCIDENT MONITORING INSTRUMENTATION (continued)

Specification: (continued)

c. Reactor Vessel Water Level

- (1) With the number of operable accident monitoring instrumentatin sensors less than the total number of sensors (for parameter 3) shown in Table 3.6.11, either restore the inoperable sensor(s) to operable status within 7 days, or be in at least hot shutdown within the next 12 hours.
- (2) With the number of operable accident monitoring instrumentation sensors (for parameter 3) less than the minimum number of operable sensors of Table 3.6.11, either restore the inoperable sensor(s) to operable status within 48 hours or be in at least hot shutdown within the next 12 hours.



Table 3.6.11

Accident Monitoring Instrumentation

<u>Parameter</u>	<u>Total Number of Sensors</u>	<u>Minimum Number of Operable Sensors</u>
(1) Relief valve position indicator [redacted]	2/valve	1/valve
[redacted]	[redacted]	[redacted]
(2) Safety valve position indicator [redacted]	2/valve	1/valve
[redacted]	[redacted]	[redacted]
(3) Reactor vessel water level	2	1



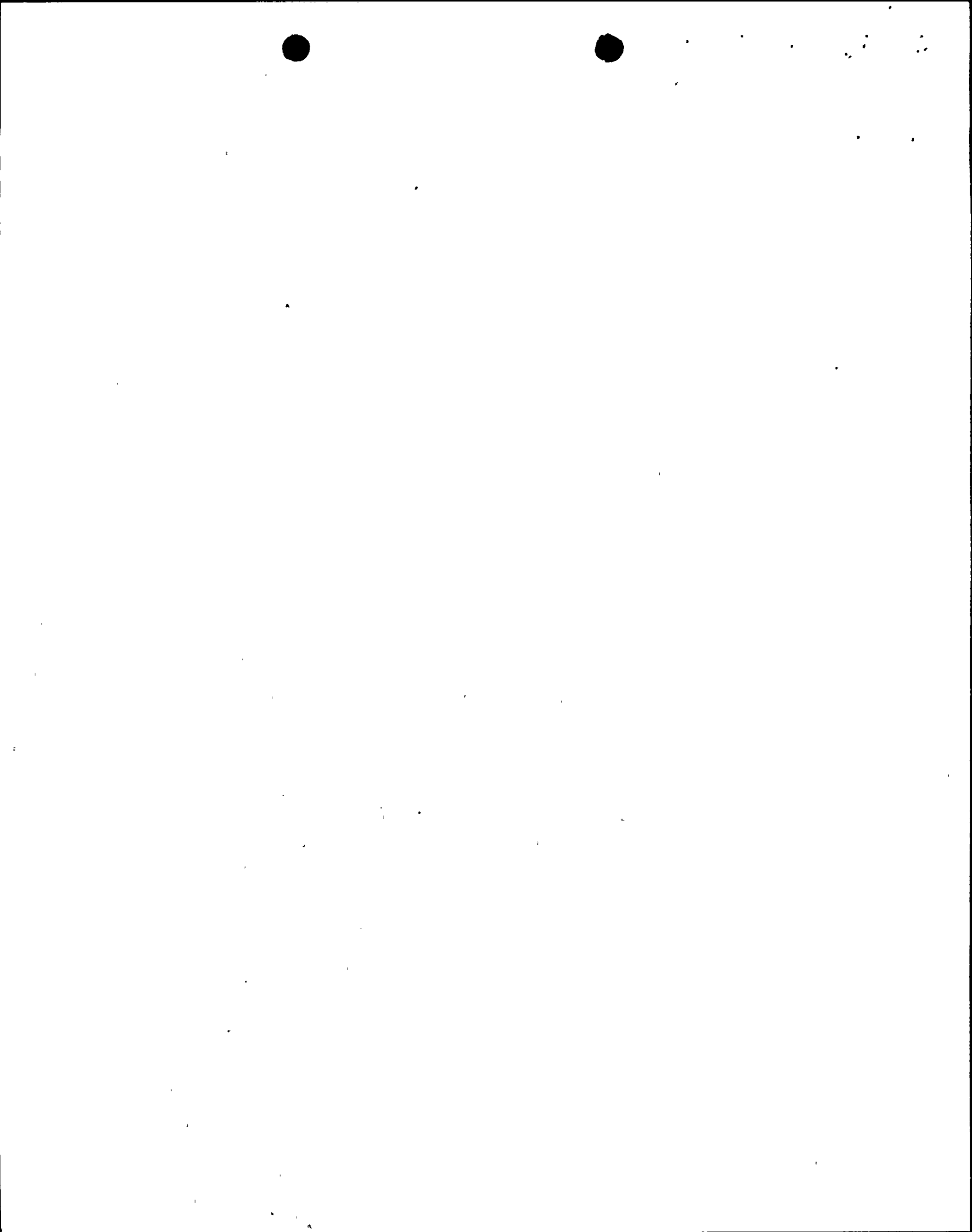
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Table 4.6.11

Accident Monitoring Instrumentation

Surveillance Requirement

<u>Parameter</u>	<u>Instrument Channel Test</u>	<u>Instrument Channel Calibration</u>
(1) Relief valve position indicator (Primary - Acoustic)	Once per month	Once during each major refueling outage
Relief valve position indicator (Backup - Thermocouple)	Once per month	Once during each major refueling outage
(2) Safety valve position indicator (Primary - Acoustic)	Once per month	Once during each major refueling outage
Safety valve position indicator (Backup - Thermocouple)	Once per month	Once during each major refueling outage
(3) Reactor vessel water level	Once per month	Once during each major refueling outage



Bases 3.6.11 and 4.6.11 Accident Monitoring Instrumentation

Accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. This capability is consistent with the recommendations of NUREG-0578, "TMI-2 Lessons Learned Task Force Status Report and Short-Term Recommendations."



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6.3 Facility Staff Qualifications

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Shift Technical Advisor who shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 Training

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975 except for Fire Brigade Training sessions which shall be held at least quarterly.

6.5 Review and Audit

6.5.1 Site Operations Review Committee (SORC)

Function

6.5.1.1 The Site Operations Review Committee shall function to advise the General Superintendent Nuclear Generation on all matters related to nuclear safety.

Composition

6.5.1.2 The Site Operations Review Committee shall be composed of the:

Chairman:	General Superintendent - Nuclear Generation
Member:	Station Superintendent - Nuclear Generation
Member:	Superintendent Results - Nuclear
Member:	Supervisor Reactor Analysis
Member:	Superintendent Maintenance - Nuclear
Member:	Supervisor Instrument & Control - Nuclear
Member:	Supervisor Radiochemical & Radiation Protection



6.14 Fire Protection Inspection

6.14.1 An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified off-site licensee personnel or an outside fire protection firm.

6.14.2 An inspection and audit by an outside qualified fire consultant shall be performed at intervals no greater than 3 years.

6.15 Environmental Qualification

A. By no later than June 30, 1982 all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification of Class IE Electrical Equipment in Operating Reactors" (DOR Guidelines); or, NUREG-0588 "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment", December 1979. Copies of these documents are attached to Order for Modification of License DPR-63, dated October 24, 1980.

B. By no later than December 1, 1980, complete and auditable records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR Guidelines or NUREG-0588. Thereafter, such records should be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified.

6.16 Systems Integrity

Procedure shall be established, implemented and maintained to meet or exceed the requirements and recommendations of section 2.1.6.a of NUREG 0578.

6.17 Iodine Monitoring

Procedures shall be established, implemented and maintained to meet or exceed the requirements and recommendations of section 2.1.8.c of NUREG 0578.

