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 FACIL: 50-410 Nine Mile Point Nuclear Station, Unit 2, Niagara Moho      05000410  
 AUTH. NAME      AUTHOR AFFILIATION  
 RHODE, G. K.      Niagara Mohawk Power Corp.  
 RECIP. NAME      RECIPIENT AFFILIATION  
 CARLSON, R. T.      Region 1, Philadelphia, Office of the Director

SUBJECT: Interim deficiency rept re spent fuel pool heat exchanges, initially reported on 800902. Vendor requested to either perform addl impact testing or show that stresses in mats are lower than required to initiate brittle fracture.

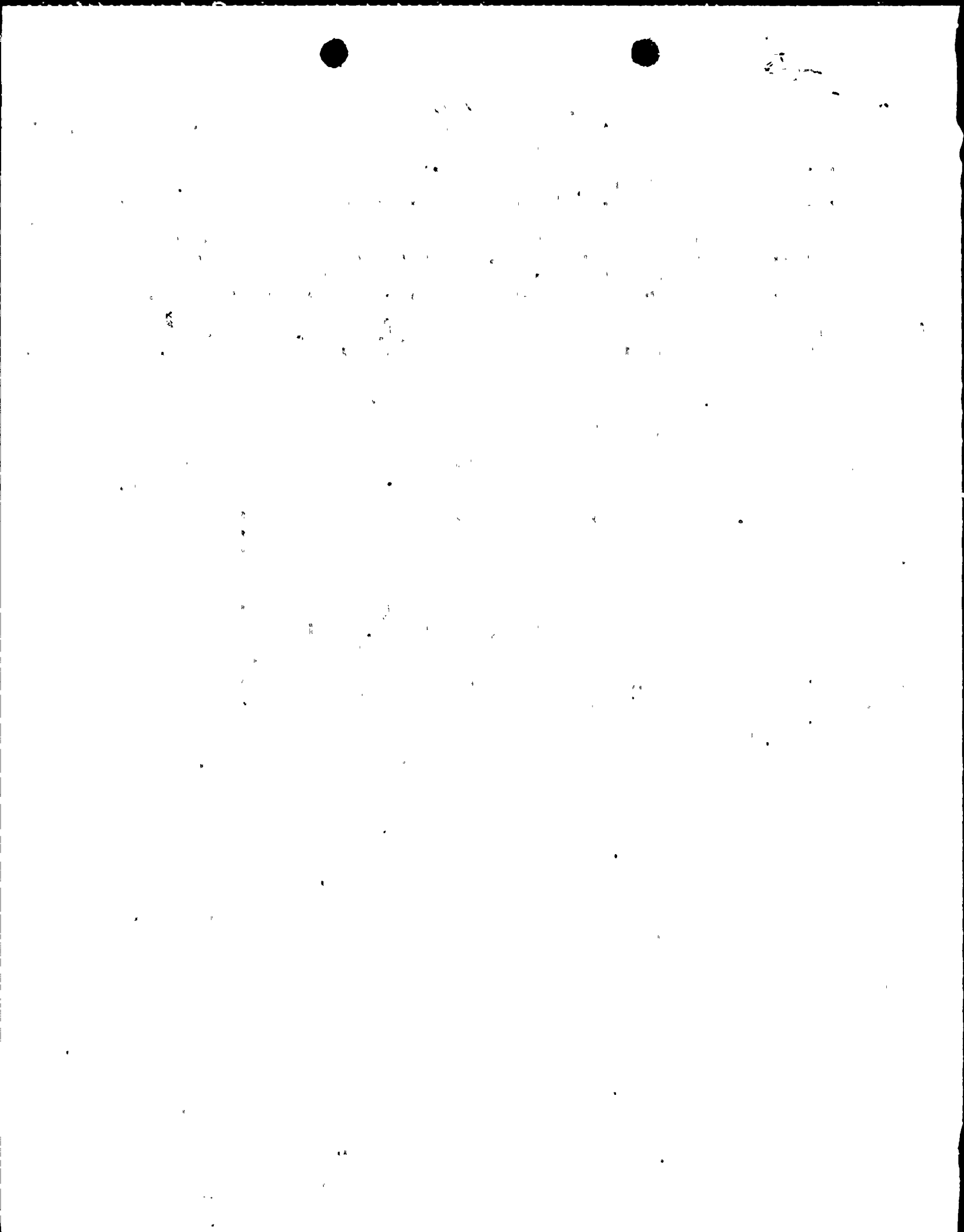
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 TITLE: Construction Deficiency Report (10CFR50.55E)

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INTERNAL:	ASLBP/J. HARD	1	1	D/DIR HUM FAC15	1	1
	EDO & STAFF 19	1	1	EQUIP QUAL BR11	1	1
	HYD/GEO BR 22	1	1	I&E 09	1	1
	IE/EES	1	1	LIC QUAL BR 12	1	1
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	OELD 21	1	1	PROG/TST REV 13	1	1
	QA BR 14	1	1	<u>REG FILE</u> 01	1	1
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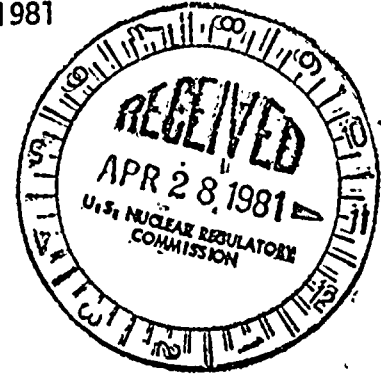
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April 23, 1981

Office of Inspection and Enforcement  
Region I  
Attn: Mr. R.T. Carlson, Chief  
Reactor Construction and Engineering Support Branch  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, Pennsylvania 19406



Dear Mr. Carlson:

Re: Nine Mile Point Unit 2  
Docket No. 50-410

Enclosed is an interim 10CFR50.55(e) report regarding a potentially reportable deficiency involving the spent fuel pool heat exchangers for Nine Mile Point Unit 2. A final report will be provided by July 1, 1981.

Very truly yours,

NIAGARA MOHAWK POWER CORPORATION

A handwritten signature in cursive script, appearing to read "Gerald K. Rhode".

Gerald K. Rhode  
Vice President

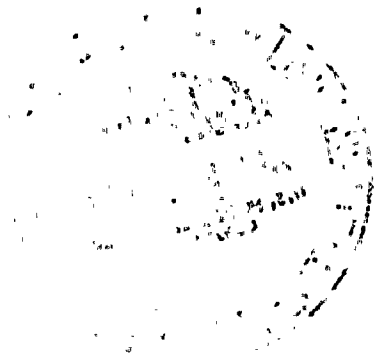
System Project Management

PEF:bd

~~cc: Director of Inspection and Enforcement~~  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

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Niagara Mohawk Power Corporation

Nine Mile Point Unit 2

Docket No. 50-410

Interim Report Spent Fuel Pool  
Cooling Water Heat Exchangers  
Potentially Reportable 10CFR50.55(e) Deficiency

Description of Deficiency

A potentially reportable deficiency involving the spent fuel pool cooling water heat exchangers was discovered during a review of documentation. This was reported to Mr. R. Feil of the Nuclear Regulatory Commission Region I staff on September 2, 1980. Interim reports were submitted to Mr. R. Carlson on October 2, 1980 and December 1, 1980.

The impact test data for several heat exchanger shell materials does not contain sufficient information to determine whether the impact test acceptance criteria of the specification have been met.

Description of Findings and Corrective Action

The heat exchanger vendor was requested to furnish additional information to evaluate the problem. The additional information furnished by the heat exchanger vendor was not sufficient to determine the acceptability of the heat exchanger material in question.

The vendor has been requested to either perform additional impact testing of specimen of the suspect materials or show that the stresses in the suspect materials are lower than that required to initiate brittle fracture using the available test data. This information will be evaluated and its safety implications analyzed. Final corrective action will be made after reviewing and analyzing this information.

