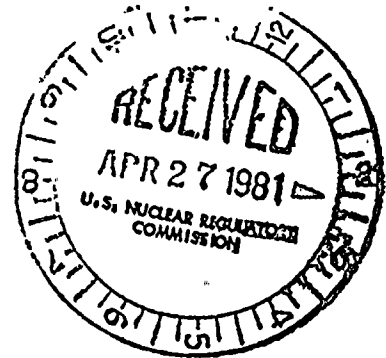


April 16, 1981

Mr. Boyce H. Grier, Director
 United States Nuclear Regulatory
 Commission - Region I
 631 Park Avenue
 King of Prussia, Pennsylvania 19406



RE: Docket No. 50-220
 LER 81-09

Dear Mr. Grier:

In accordance with Nine Mile Point Nuclear Station Unit #1 Technical Specifications, we hereby submit the following Licensee Event Report:

81-09 which is being submitted in accordance with Section 6.9.2(b)1, Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.

This report was completed in the format designated in NUREG-0161, dated July 1977.

Very truly yours,

Thomas E. Lempges
 Thomas E. Lempges
 Vice President
 Nuclear Generation

TEL/PH: jll
 Attachments (3 copies)
 xc: Director, Office of I&E (30 copies)
 Director, Office of MIPC (3 copies)

A002
 3/11

810.4280 424

5

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

011 | NYINMP1 | 00-0000000-00 | 34111111 | 3 | 5

7 8 9
14 15
25 25
33 33
37 37
53 53

011 | L | 05000220 | 70312181 | 3042081 | 9

7 8 9
30 31
68 69
74 75
80 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

012 | During a refueling outage, while performing outage frequency isolation valve pneumatic
013 | leak rate testing, a 3/4" torus oxygen sampling isolation check valve #201.2-71 did not
014 | meet the Tech Spec. minimum leak rate requirement for one isolation valve. The accept-
015 | able limit is <12.9 SCFH. Testing indicated a leak rate of >20.0 SCFH. This had min-
016 | imal safety implications since the redundant check valve tested satisfactorily and the
017 | overall primary containment leakage values were acceptable.

019 | SE | E | X | VALVEX | C | A

7 8 9
9 10
11 12
13 13
18 18
19 20

17 | 81 | 009 | 03 | L | 0

7 8 9
21 22
23 23
24 25
27 27
28 29
30 31
32 32

AX | Z | Z | 0000 | N | N | L | C3319

7 8 9
33 34
35 35
36 36
37 40
41 41
42 42
43 43
44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

110 | Upon valve disassembly and examination, the poppet seating O ring exhibited a worn con-
111 | dition. This seating O ring was readily replaced and the valve was retested with accept-
112 | able results. Further testing following flow through the line for a 48-hour period pro-
113 | duced repeatable satisfactory results. Further action involves a history file for fur-
114 | ther trend analysis.

15 | H | 000 | N/A | B | REFUELING SURVEILLANCE

7 8 9
10 10
12 13
44 44
45 46
80 80

16 | Z | Z | N/A | N/A

7 8 9
10 10
44 44
45 45
80 80

17 | 000 | Z | N/A

7 8 9
11 12
13 13
80 80

18 | 000 | N/A

7 8 9
11 12
80 80

19 | Z | N/A

7 8 9
11 12
80 80

20 | Z | N/A

7 8 9
10 10
80 80

8104280430