CENTRAL FILES

NIAGARA MOHAWK

NIAGARA MOHAWK POWER CORPORATION/300 ERIE BOULEVARD WEST, SYRACUSE, N.Y. 13202/TELEPHONE (315) 474-1511

April 30, 1980

Office of Inspection & Enforcement
Region I
Attn: Mr. Boyce H. Grier, Director
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Dear Mr. Grier:

Re: Nine Mile Point Unit 1
Docket No. 50-220
DPR-63

All inspections required by Inspection and Enforcement Bulletin 79-14, including those inside of the drywell, have been completed. Our letters of August 17, 1979, November 7, 1979, December 7 and 31, 1979, February 22, 1980 and March 31, 1980 provided interim reports regarding these required inspections. The information provided below supplements that previously provided and completes our response.

Due to high radiation fields and physical locations, inspections were limited on certain systems as follows:

1. The reactor recirculating pump supports were inspected. However, all the frames were not inspected. These frames are a part of the Inservice Inspection program and inspected accordingly.

2. Seismic restraints and piping geometry for a portion of the 4 inch reactor feedwater bypass to the main condenser were visually inspected only. These restraints were previously inspected for general arrangement and member sizes under Inspection and Enforcement Bulletin 79-02.

3. Portions of the Instrument air, Reactor Building Closed Loop Cooling and Emergency Condenser Makeup systems were visually inspected for piping geometry only. These systems however, are of low temperature and pressure design.

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As a result of drywell inspections and subsequent evaluations, the following additional discrepancies have been identified.

1. One snubber on the west side of the reactor feedwater system was mislocated by approximately four feet. This system has been re-analyzed in the as built condition and found to be in compliance with allowable code stress levels.

2. A vertical run of pipe on the west side of the core spray system was found to be a few feet longer than that modeled in the computer analysis. This system has been re-analyzed in the as built condition and found to be in compliance with allowable code stress levels.

3. A 2 inch diameter section of the reactor drain to the cleanup system was found to differ from that modeled in the computer analysis. This system will be re-analyzed by June 1, 1980. If modifications are necessary, plans will be reported at that time.

In addition, the following discrepancies on systems outside the drywell remain outstanding.

1. The Reactor Containment Purge and Fill N_2 and the Drywell Vent and Purge Air and N_2 systems lacked as built information. This has been gathered during the 79-14 inspection. The systems are bing evaluated based on original plant design criteria. This evaluation and any modifications necessary to satisfy this criteria will be completed by June 1, 1980.

2. As built information for the H_2-O_2 monitoring system gathered under 79-14 showed some differences in geometry and restraint function from the computer analysis. All piping in this system is 1 inch and less. Those portions of the system found different from the design drawings will be re-analyzed by June 1, 1980. If modifications are necessary, plans will be reported at that time.

3. Two restraints in containment spray raw water system were not in accordance with the original design. Rather than re-analyze this system, these restraints will be modified by June 1, 1980.

4. Two restraints on a drain line off the feedwater system were not in accordance with original design. The effects of this discrepancy will be analyzed by June 1, 1980. If any modifications are necessary, plans will be reported at that time.

In summary, our evaluation and review of safety related piping systems per Inspection and Enforcement Bulletin 79-14 resulted in approximately 140 restraint modifications. Typical modifications included increasing weld sizes, adding U-bolts, stiffener plates, and new members to existing restraints as well as modifying or adding new restraints.

Very truly yours,

NIAGARA MOHAWK POWER CORPORATION

Thomas E. Lempges

Thomas E. Lempges
Vice President - Nuclear Generation

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