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RECIP.NAME RECIPIENT AFFILIATION
EISENHUT,D.G. Division of Operating Reactors

SUBJECT: Forwards info re primary coolant pressure isolation capability as per 800223 request.Info demonstrates that continued operation of facility does not present undue safety hazard to public..

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NIAGARA MOHAWK

NIAGARA MOHAWK POWER CORPORATION/300 ERIE BOULEVARD WEST, SYRACUSE, N.Y. 13202/TELEPHONE (315) 474-1511

March 19, 1980

Director of Nuclear Reactor Regulation
Attn: Mr. Darrell G. Eisenhut, Acting Director
Division of Operating Reactors
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Re: Nine Mile Point Unit 1
Docket No. 50-220
DPR-63

Gentlemen:

Your letter of February 23, 1980 requested information regarding primary coolant pressure isolation capability (WASH-1400 EVENT V). The attachment to this letter addresses your concerns.

The information contained in the attachment to this letter demonstrates that continued operation of Nine Mile Point Unit 1 does not present an undue safety hazard to the public.

Very truly yours,

NIAGARA MOHAWK POWER CORPORATION

Donald P. Dise
Vice President - Engineering

MGM:jk
attachment

A001
S
1/1
ADD: LTR ENC
P. Polk 1 1
E. Butcher 1 1

8004010330

STATE OF NEW YORK)
COUNTY OF ONONDAGA) ss:

DONALD P. DISE, being duly sworn, says:

I am Vice President, Engineering of Niagara Mohawk Power Corporation. I have read the foregoing letter and the facts contained in the letter and attachment are true to the best of my knowledge, information and belief.

Donald P. Dise

Donald P. Dise

Sworn to before me on
this 26th day of MARCH , 1980

Phyllis D. Voytko

Notary Public

PHYLLIS D. VOYTKO
Notary Public in the State of New York
Qualified in Onon. Co. No. 34-9458535
My Commission Expires March 30, 1982

NUCLEAR REGULATORY COMMISSION
REQUEST FOR INFORMATION

WASH-1400, EVENT V
(LOSS OF PRESSURE ISOLATION)

Introduction and Summary

Wash-1400 describes an Event V configuration as two in series check valves that isolate a low pressure system from the reactor coolant system. There are no configurations at Nine Mile Point Unit 1 which conform with this definition.

The following systems interface with the primary coolant system. A one-line diagram is attached for each.

1. Feedwater (Figure OP-16-1)
2. Control Rod Drive (Figure OP-5-1)
3. Shutdown Cooling (Figure OP-4-1)
4. Cleanup (Figure OP-3-1)
5. Core spray (Figure OP-2-1)
6. Emergency Condenser (Figure OP-13-1)
7. Liquid Poison (Figure OP-12-1)
8. Head spray (Figure OP-4-2)
9. Reactor instrumentation (Figure VIII-3)
10. Main steam (Figure OP-1-1)

Description

The following is a description of the valve configuration on all systems which interface with the primary coolant system at Nine Mile Point Unit 1.

1. Feedwater (Figure OP-16-1)

   The feedwater system is designed for reactor pressure from the reactor vessel back to and including the feedwater pumps. This system does not rely upon isolation between high and low pressure by two check valves in series.

2. Control Rod Drive (Figure OP-5-1)

   The control rod drive system has two check valves in series. However, this system is a high pressure system from the reactor vessel back to and including the control rod drive pumps. In addition, there are motor operated valves between the check valves and the pumps which could be used for isolation.

3. Shutdown Cooling (Figure OP-4-1)

   The entire shutdown cooling system is designed for reactor pressure. Therefore, there is no isolation between high and low pressure.

4. Reactor Cleanup (Figure OP-3-1)

The reactor cleanup system is designed for reactor pressure from the reactor recirculation line on the outlet side to CV-BYPASS PCV and CV-PCV-1. It is also high pressure on the inlet side from the clean-up pumps back to the reactor recirculation line. There are two check valves in series on the cleanup inlet. However, they are not relied upon to isolate the high pressure from low pressure. There is a motor operated valve at the discharge of the cleanup pumps and there is a motor operated isolation valve inside the drywell.

5. Reactor Core Spray (Figure OP-2-1)

The core spray system is designed for reactor pressure from the reactor vessel to check valves CRS #19 and 20. This system does not rely on two check valves in series for pressure isolation. Normal isolation is by motor operated valve inside containment.

6. Emergency Cooling (Figure OP-13-1)

The emergency cooling system steam supply and condensate return are designed for reactor pressure. There are no high to low pressure boundaries.

7. Liquid Poison (Figure OP-12-1)

The liquid poison system is designed for reactor pressure from the reactor vessel back to and including the positive displacement pumps. This system contains two check valves in series. However, there is a normally closed explosive valve and another check between that configuration and the pumps.

8. Head Spray (Figure OP-4-2)

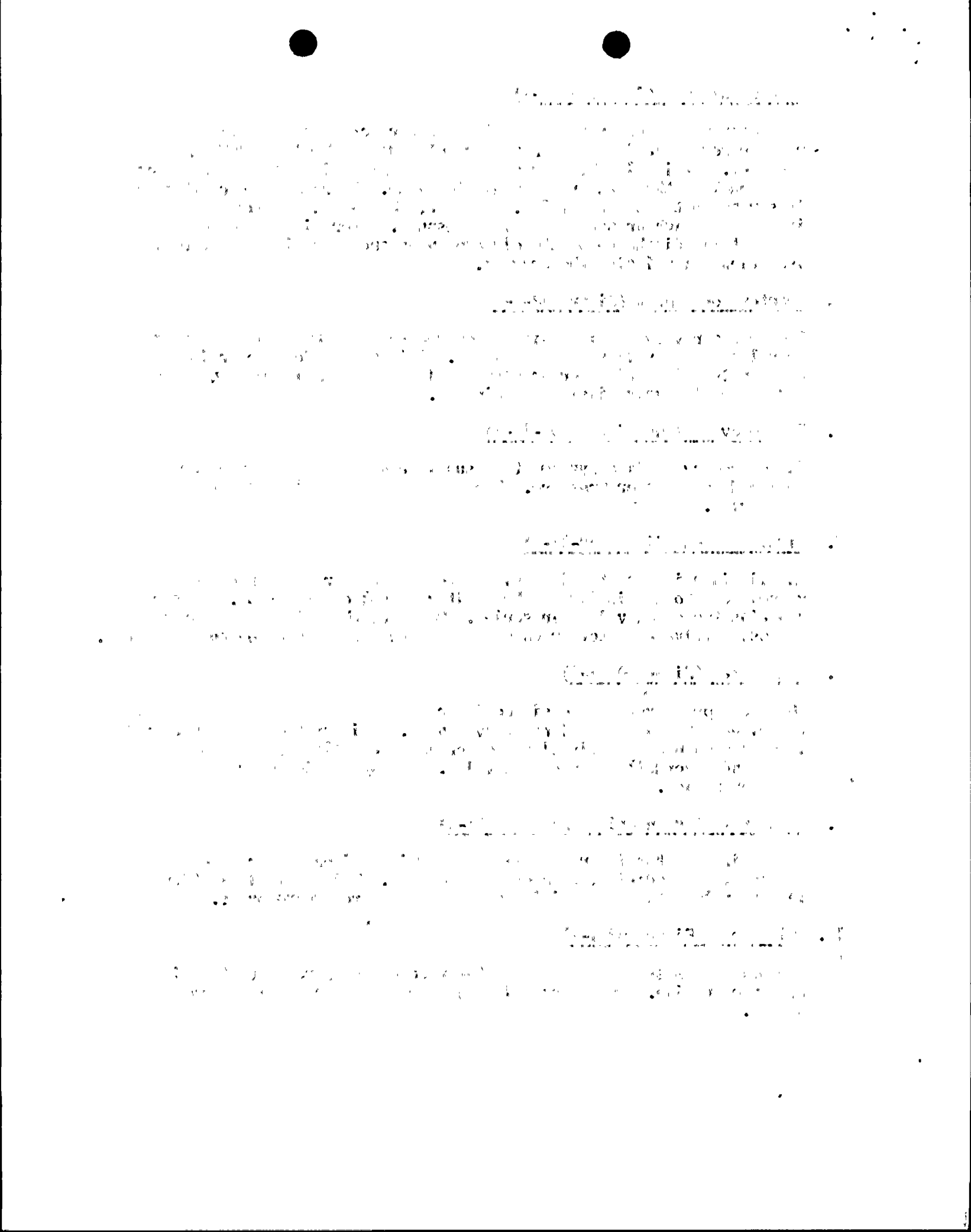
The head spray system is designed for reactor pressure from the reactor vessel back to the control rod drive pumps. This system has a check valve in series with a normally closed motor operated valve, a flow control valve and a normally closed gate valve. The system is not used during power operation.

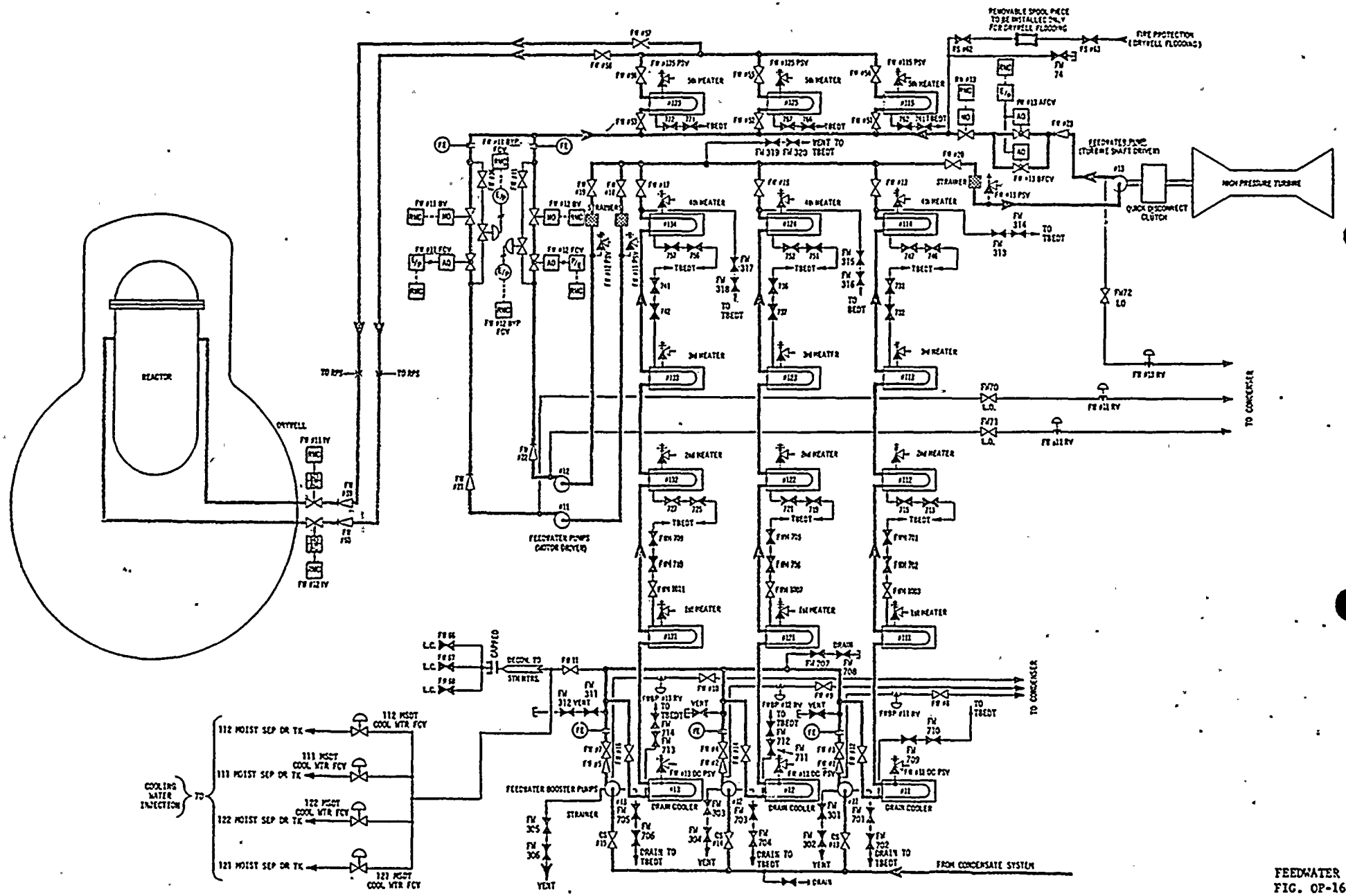
9. Reactor Instrumentation (Figure VIII-3)

The entire reactor instrumentation system is designed for reactor pressure. In addition, flow checks are used. This would limit flow to less than the makeup capability of the control rod drive pumps.

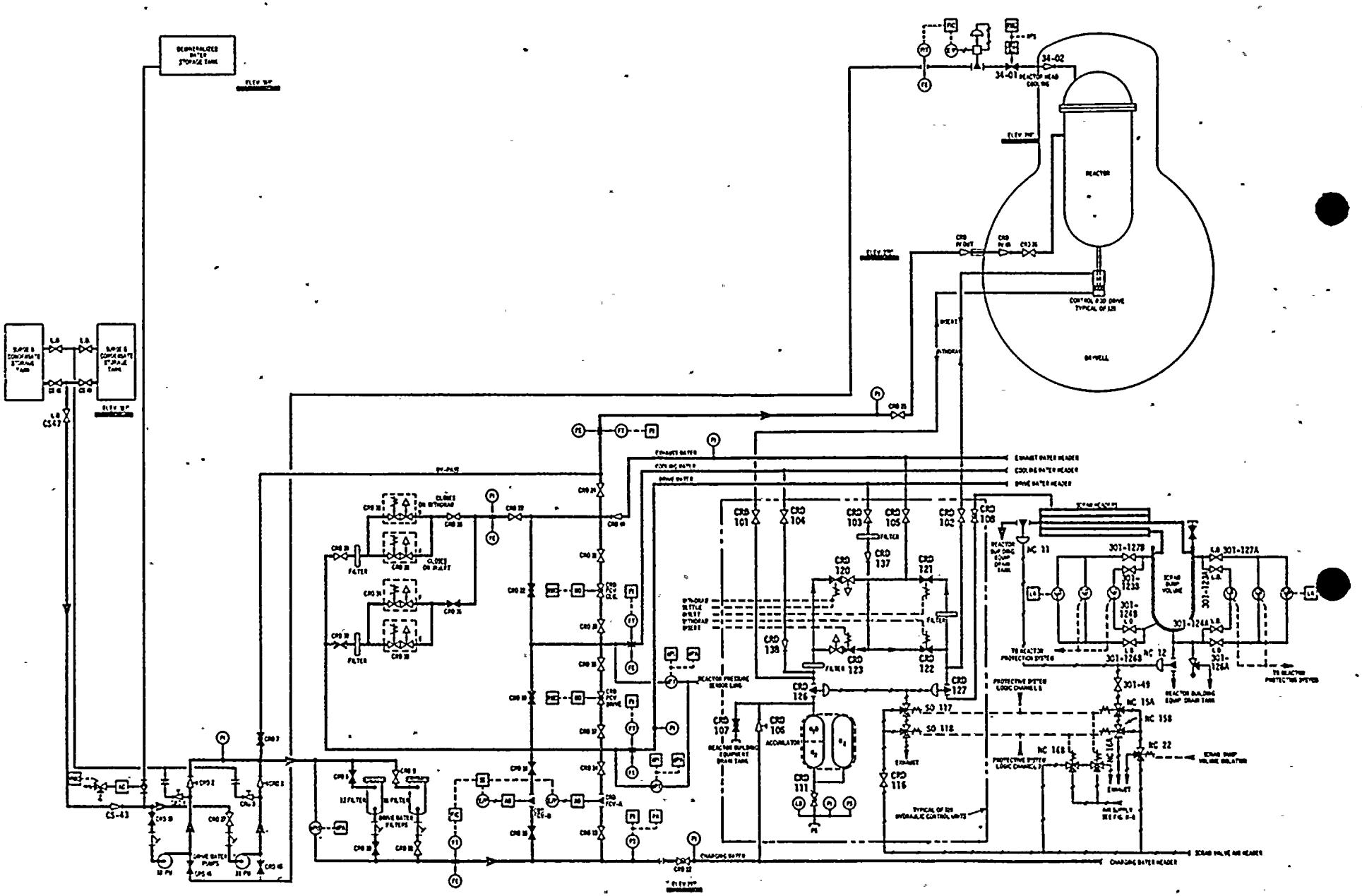
10. Main Steam (Figure OP-1-1)

The main steam system is designed for reactor pressure up to the high pressure turbine. Two check valves in series are not relied upon in this system.

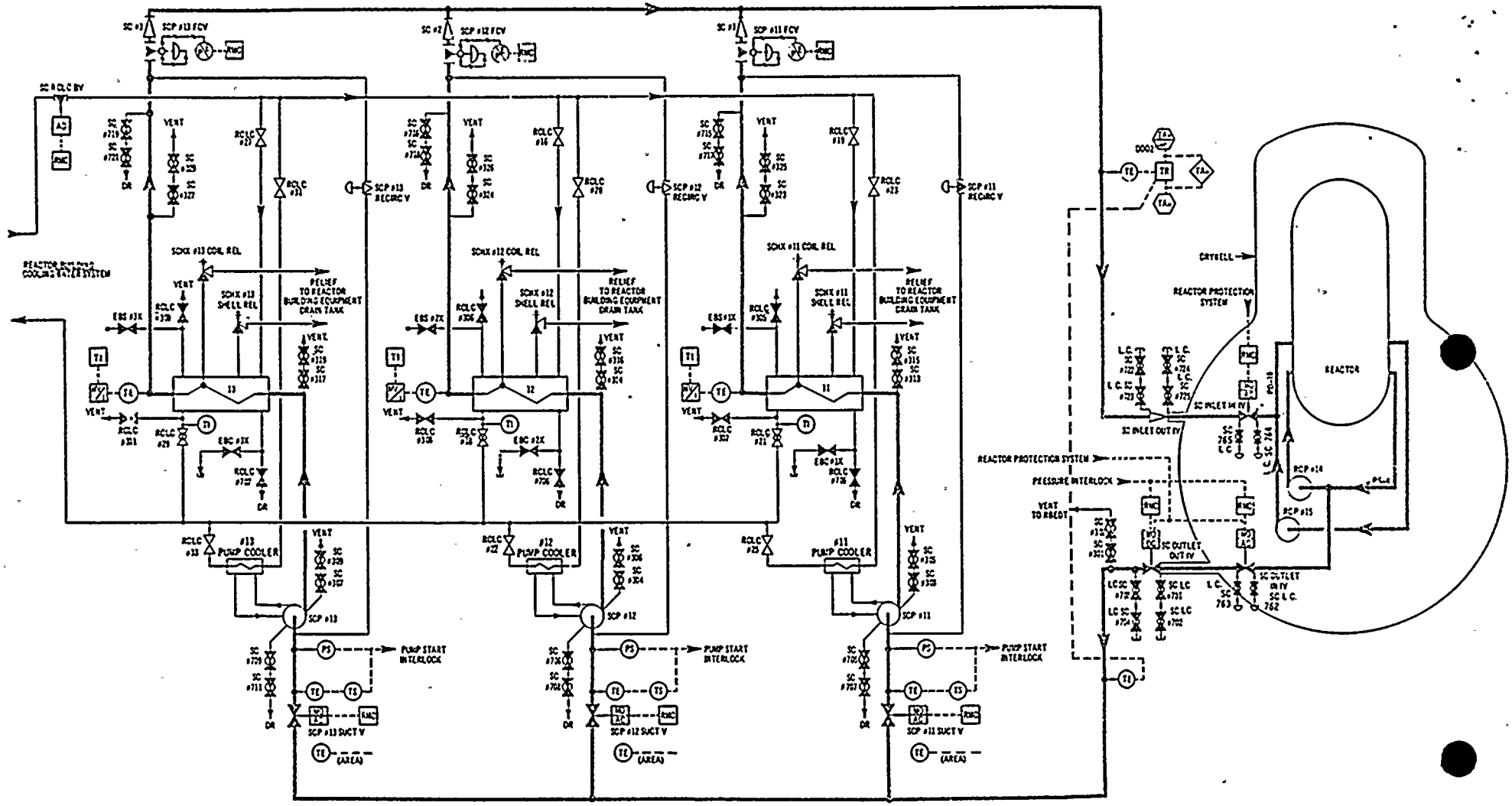




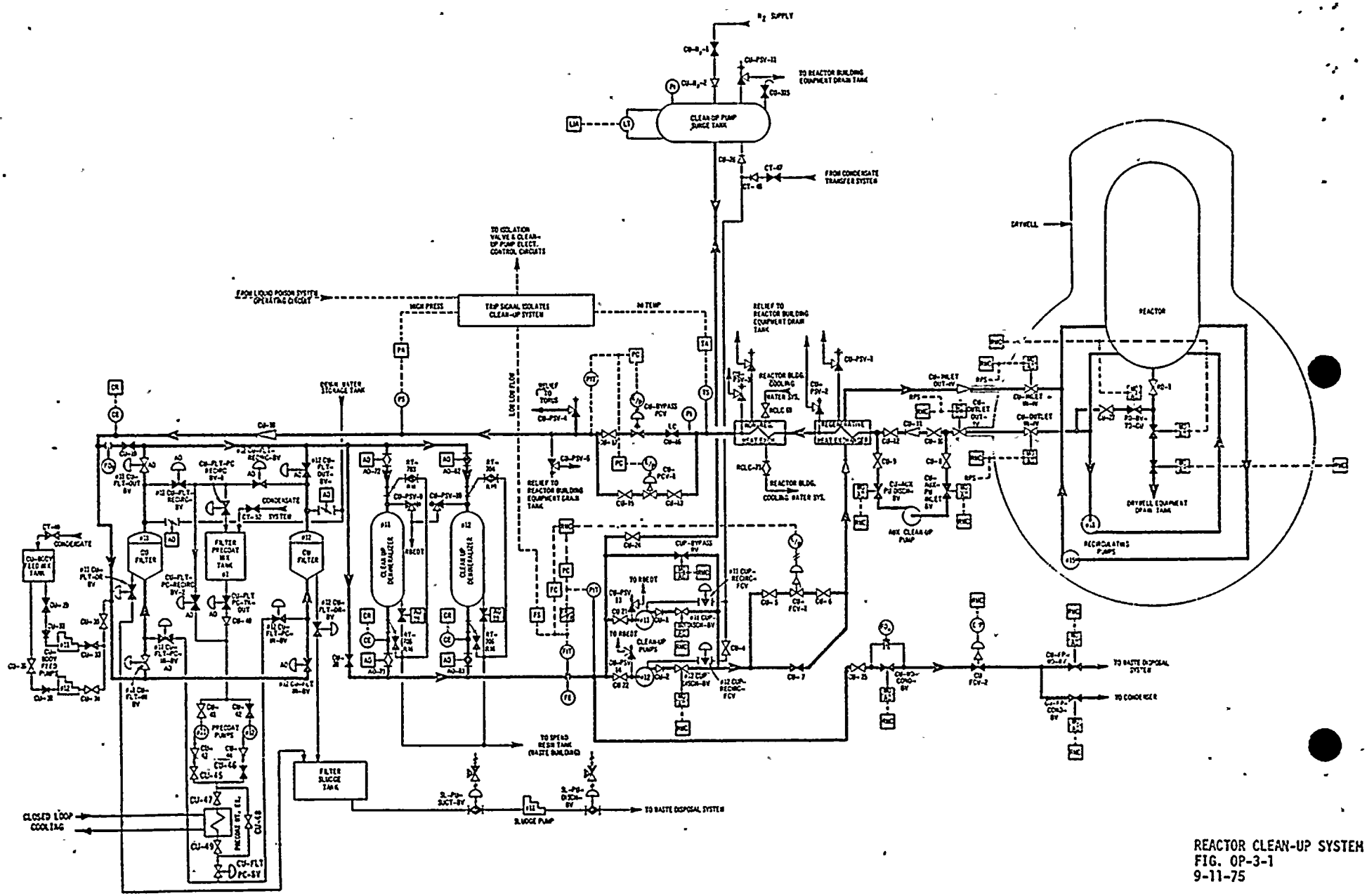
FEEDWATER SYSTEM
FIG. OP-16-1
9-30-75



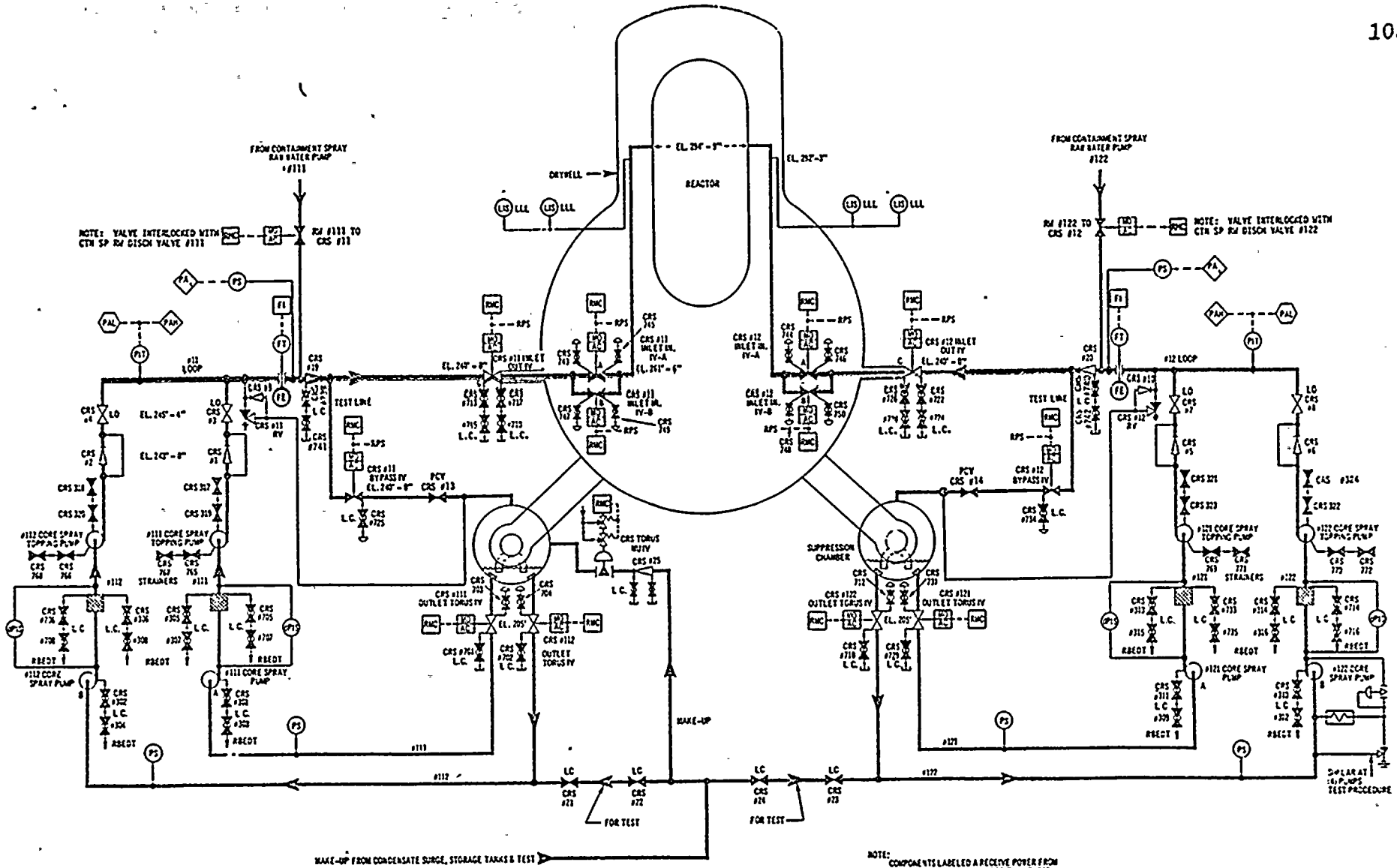
CONTROL ROD DRIVE HYDRAULIC SYSTEM
FIG. OP-5-1



SHUTDOWN COOLING SYST
FIGURE OP-4-1
8-4-75



REACTOR CLEAN-UP SYSTEM
FIG. OP-3-1
9-11-75



NOTE:
COMPONENTS LABELED A RECEIVE POWER FROM A SEPERATE DIESEL GENERATOR THAN THOSE LABELED B COMPONENTS LABELED C RECEIVE POWER FROM EITHER DIESEL GENERATOR.

REACTOR CORE SPRAY SYSTEM
FIGURE OP-2-1

REVISED 4-1-75

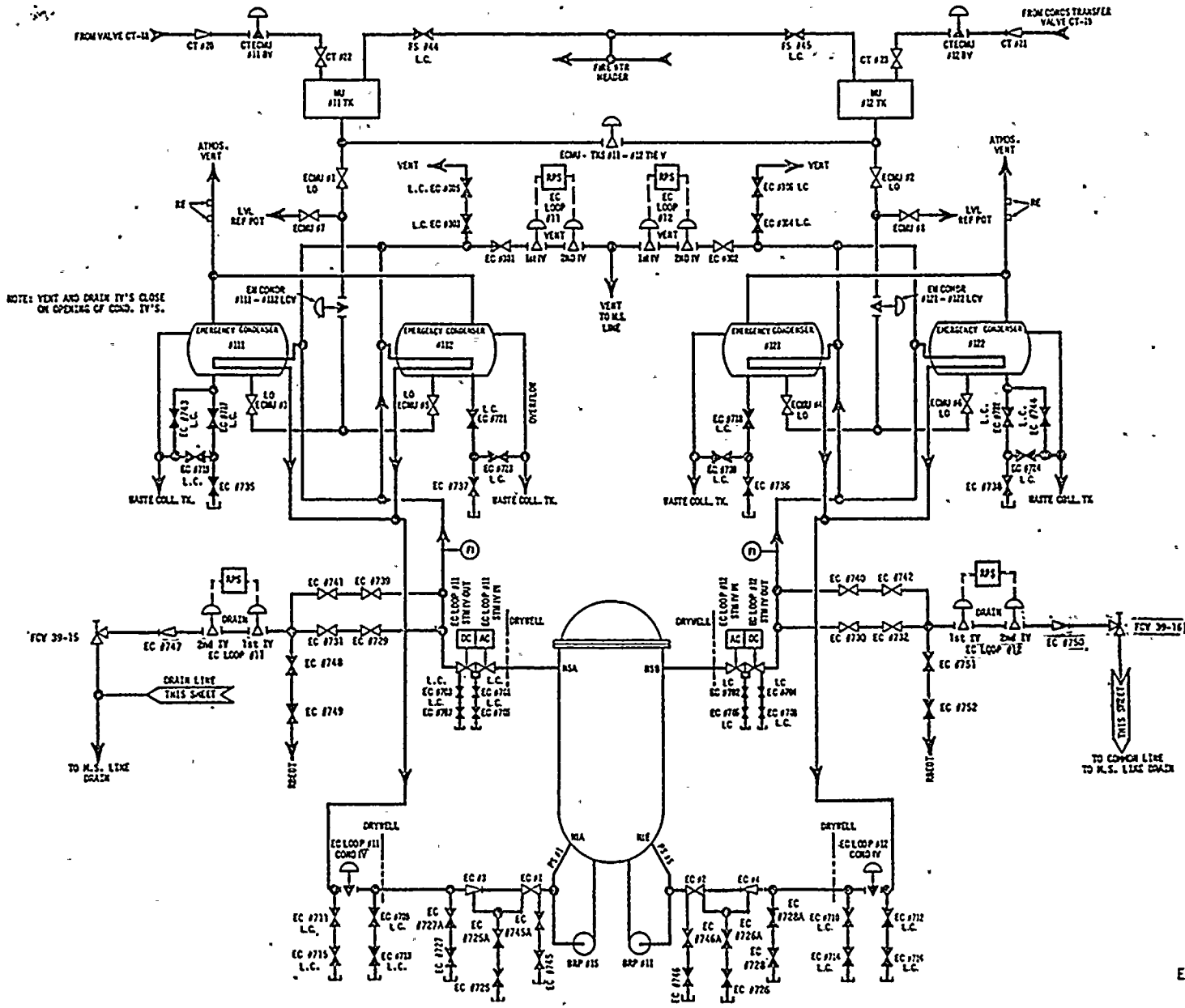
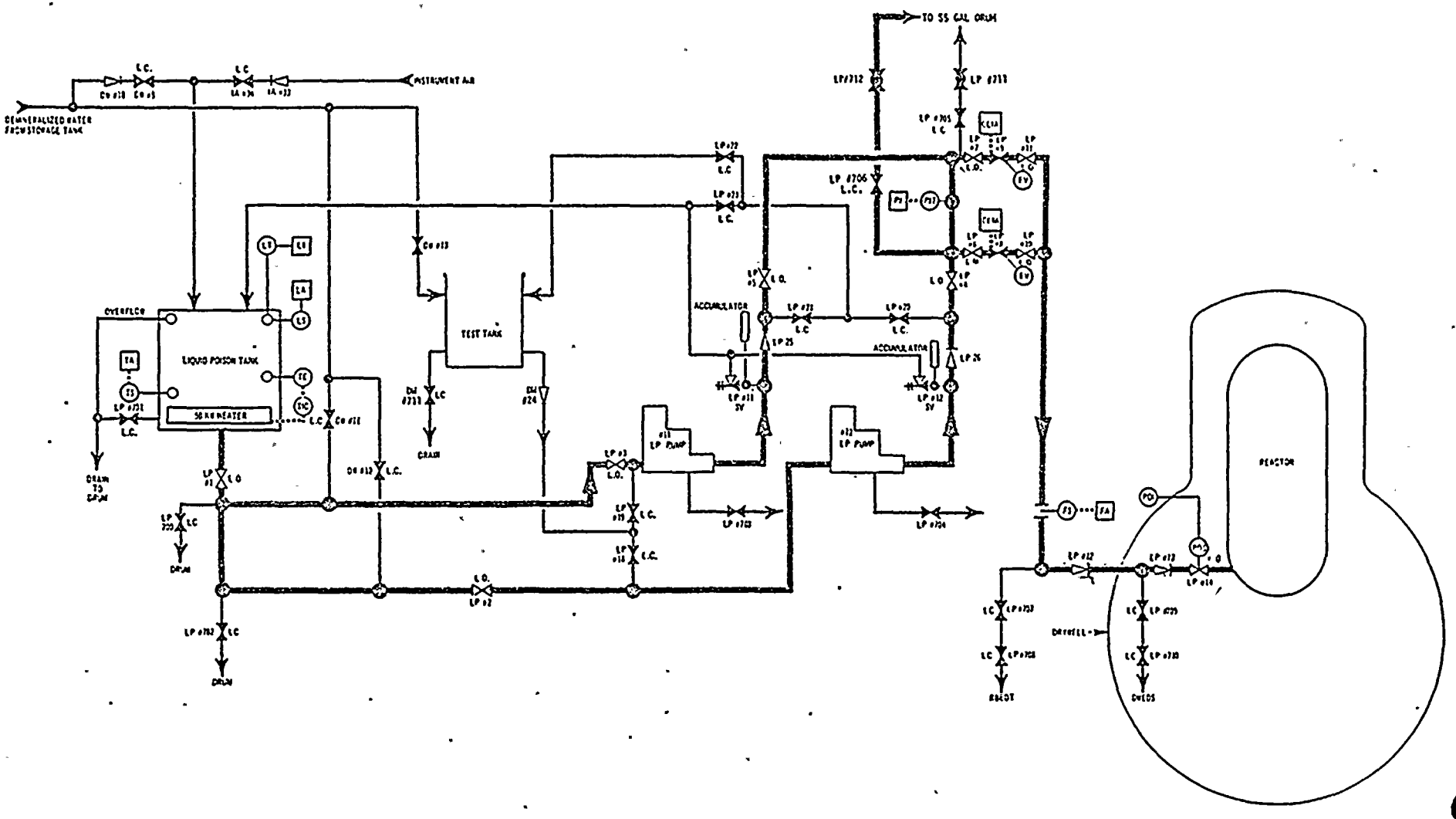


FIGURE OP-13-1
EMERGENCY COOLING SYST

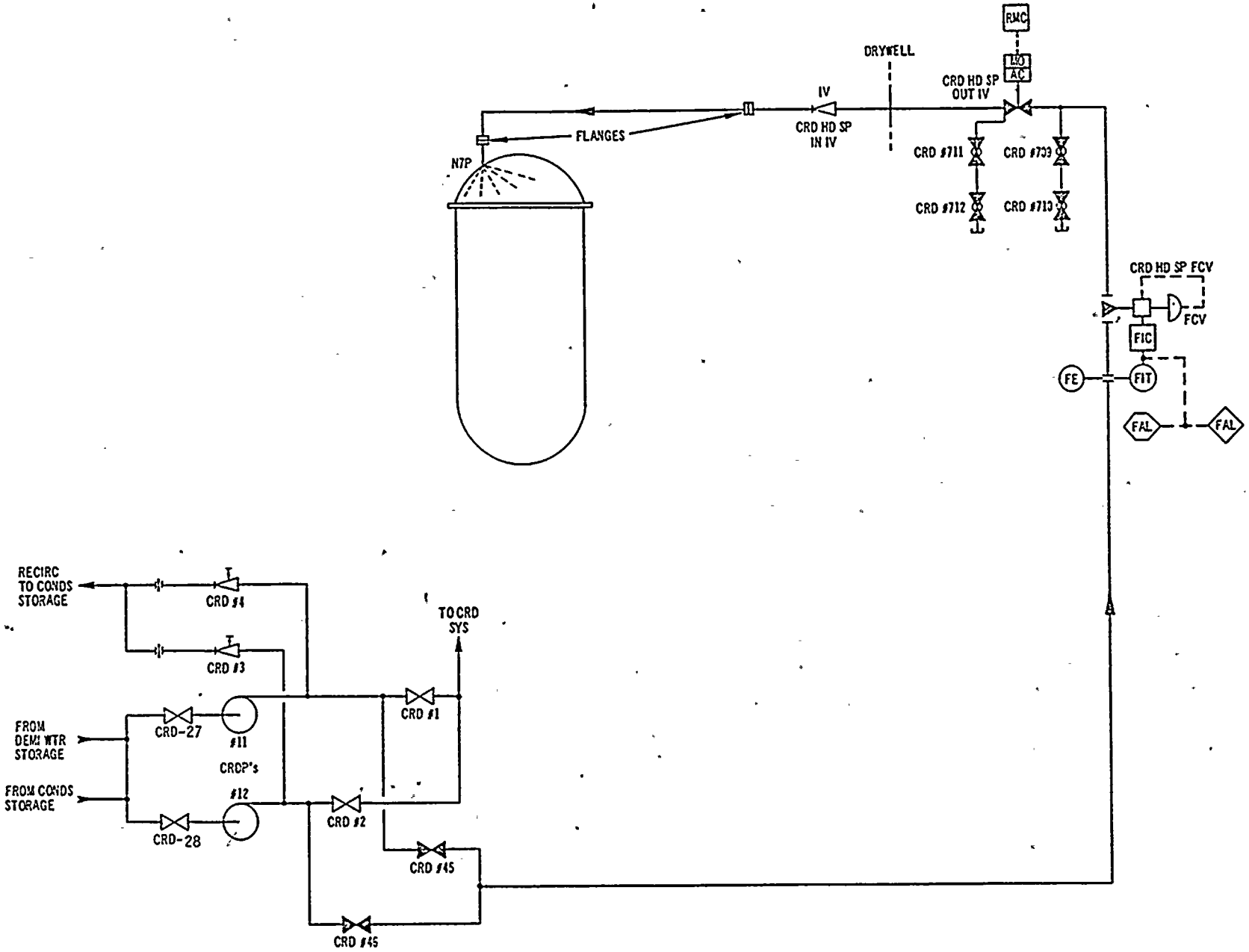
REVISED 4-1-75



LIQUID POISON SYSTEM

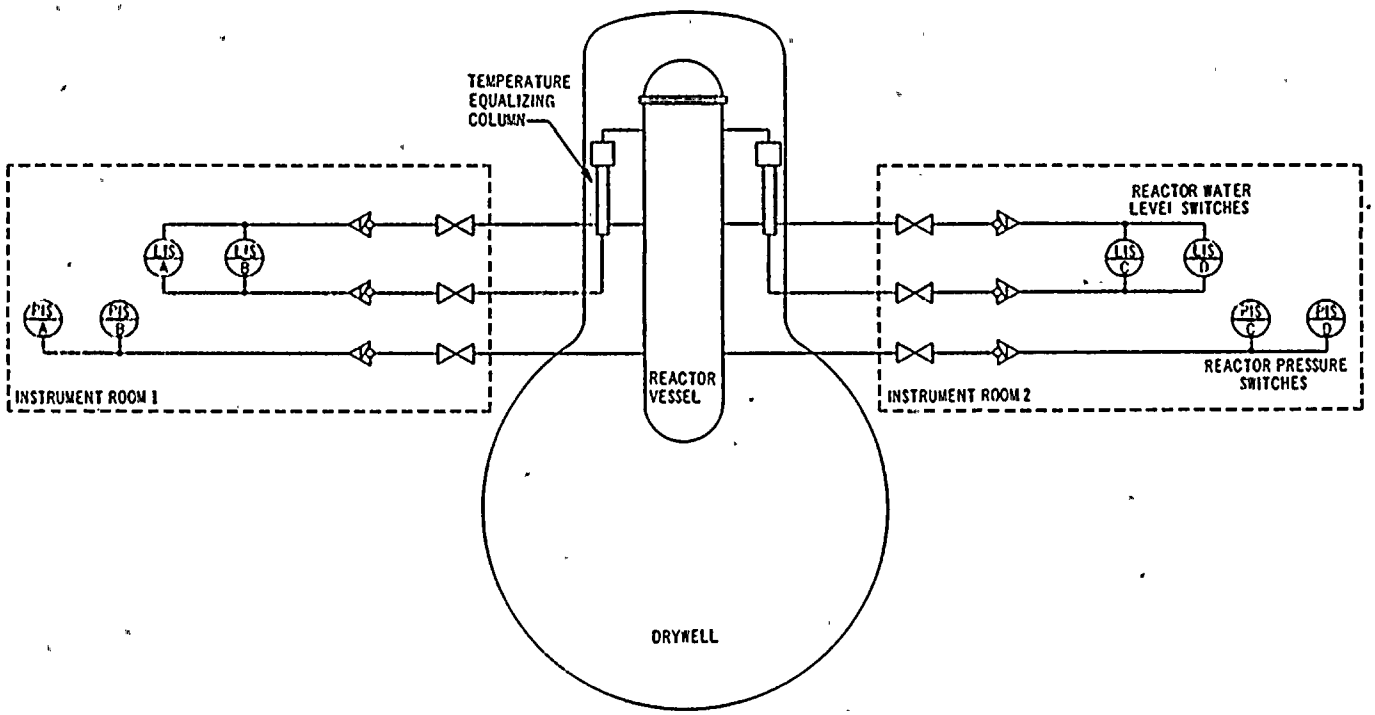
FIGURE OP-12-1

REV. 2-17-75



HEAD SPRAY SYSTEM
FIGURE OP-4-2

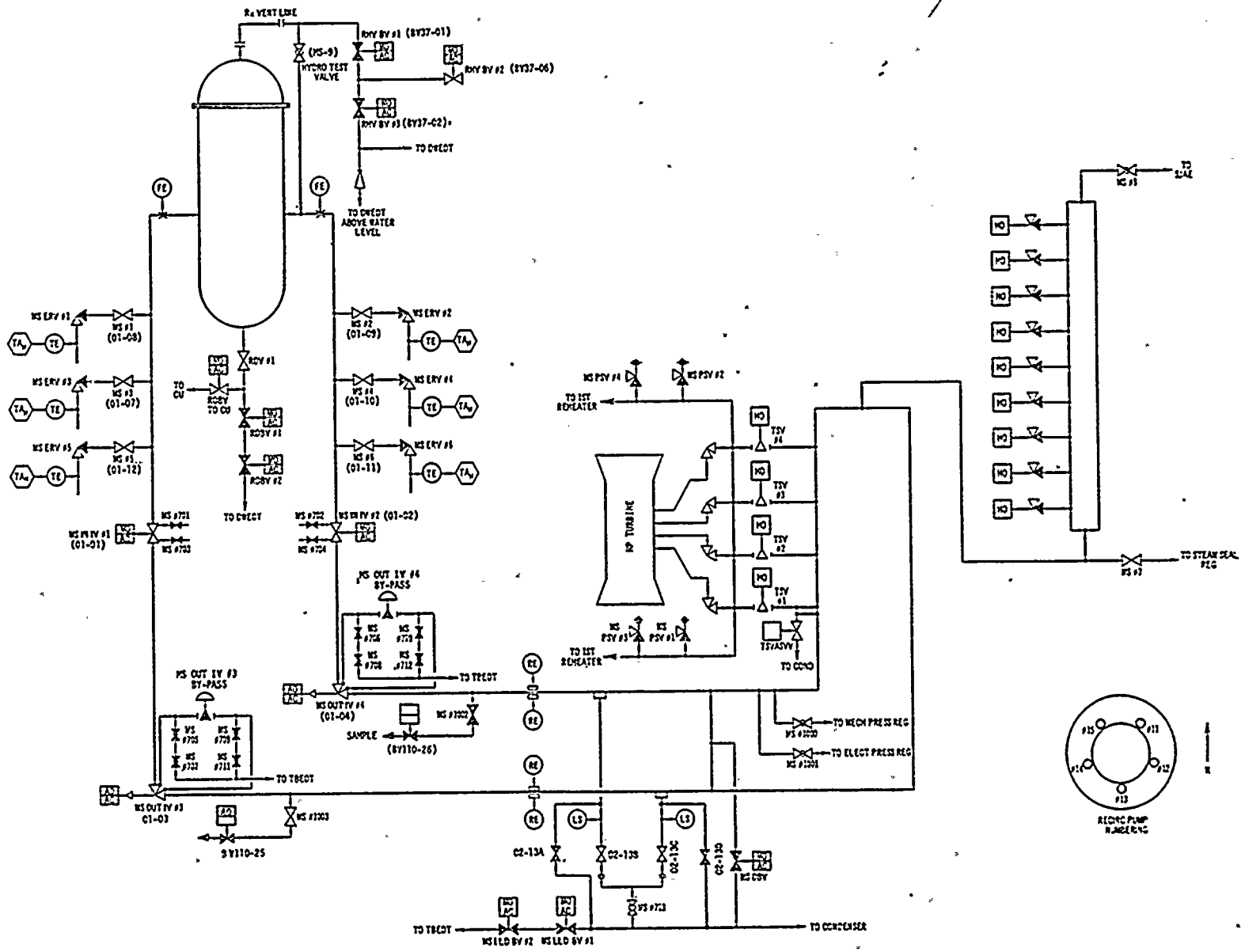
PROTECTIVE SYSTEM TYPICAL SENSOR ARRANGEMENT



TYPICAL PROTECTIVE SYSTEM
SENSOR ARRANGEMENT

LOGIC CHANNEL	1A	1B	2A	2B
PRESSURE SWITCH	A	C	B	D
LEVEL SWITCH	A	C	B	D

FIGURE VIII – 3



NUCLEAR STEAM SYSTEM
FIGURE OP-1-1
9-1-75