

U.S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT

Region I

Report No. 50-220/79-02

Docket No. 50-220

License No. DPR-63 Priority --- Category C

Licensee: Niagara Mohawk Power Corporation

300 Erie Boulevard West

Syracuse, New York 13202

Facility Name: Nine Mile Point Nuclear Station, Unit 1

Inspection at: Scriba, New York

Inspection conducted: January 23-26, 1979

Inspectors: H.B. Kister for
R. P. Zimmerman, Reactor Inspector

2/6/79
date signed

Accompanying
Personnel: H. B. Kister, Chief, Nuclear Support Section
No. 2, RO&NS Branch

_____ date signed

Approved by: H.B. Kister
H. B. Kister, Chief, Nuclear Support
Section No. 2, RO&NS Branch

_____ date signed
2/6/79
date signed

Inspection Summary:

Inspection on January 23-26, 1979 (Report No. 50-220/79-02)

Areas Inspected: Routine, unannounced inspection by regional based inspector of licensee action on previous inspection findings; administrative controls for surveillance procedures; surveillance testing; inspector witnessing of surveillance tests; technician qualification; and facility tours. The inspection involved 23.5 inspector-hours onsite by one NRC regional based inspector and one supervisor.

Results: Of the six areas inspected, no items of noncompliance were identified in five areas; one apparent item of noncompliance was found in one area (Infraction - approval of unacceptable surveillance test results - Paragraph 4.c(3)).



DETAILS

1. Persons Contacted

Principal Licensee Employees

W. Bandla, Shift Supervisor
*W. Drews, Reactor Analyst
J. Earls, Shift Supervisor
*T. Perkins, Station Superintendent
*J. Shea, Shift Supervisor
*M. Silliman, Results Superintendent
B. Taylor, Instrument & Control Supervisor

Other Accompanying NRC Personnel

*H. Kister, Chief, Nuclear Support Section No. 2, RO&NS Branch

The inspector also interviewed other licensee employees including members of the technical and engineering staff and reactor operators.

* denotes those attending the exit interview.

2. Licensee Action on Previous Inspection Findings

(Open) Unresolved item (220/78-03-01): Licensee to revise surveillance procedure N1-ST-Q1. Office of Inspection and Enforcement Report No. 50-220/78-03 documented that the Core Spray Motor Cooler lacked instrumentation to certify its operability and the applicable surveillance procedure did not have provisions for monitoring cooler performance.

The licensee representative stated that the surveillance procedure will be revised to include the use of a hand held pyrometer in order to certify cooler operability.

This item remains unresolved pending procedure revision and subsequent NRC:RI review.

3. Administrative Controls for Surveillance Procedures

The inspector performed an audit of the licensee's administrative controls by conducting a sampling review of the below listed administrative procedures with respect to the requirements of the Technical



Specifications, Section 6, "Administrative Controls," ANSI N18.7, "Administrative Controls for Nuclear Power Plants" and Regulatory Guide 1.33, "Quality Assurance Program Requirements."

- Administrative Procedure No. (APN)-1, Procedure for Administrative Controls, Revision 2, June 19, 1978.
- APN-5, Procedure for Control of Procedures, Instructions and Orders, Revision 1, January 20, 1978.
- APN-6, Preparation of Procedures, Instructions and Orders, Revision 1, July 11, 1978.
- APN-8, Test and Inspection Program, Revision 0, November 3, 1977.
- APN-15, Procedure for Control and Calibration of Equipment used in Tests and Inspections, Revision 1, March 3, 1978.

No items of noncompliance were identified.

4. Surveillance Testing

- a. The inspector reviewed surveillance tests on a sampling basis to verify the following:
 - Tests required by Technical Specifications are available and covered by properly approved procedures.
 - Test format and technical content are adequate and provide satisfactory testing of related systems or components.
 - Test results of selected tests are in conformance with Technical Specifications and procedure requirements have been reviewed by someone other than the tester or individual directing the test.
- b. The following surveillance tests were reviewed to verify the above.
 - N1-ST-Q6, January 9, 1979, Containment Spray and Raw Water Pumps Operability Test. Data reviewed for tests performed February 16, May 12, August 14 and November 11, 1978.
 - N1-PSP-3, August 31, 1978, Liquid Poison Tank Sampling and Boron Analysis. Data reviewed for tests performed October 10, November 9, November 13, December 15, 1978 and January 11, 1979.



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- N1-ST-C2, September 13, 1978, Manual Opening of the Solenoid-Actuated Pressure Relief Valves and Flow Verification. Data reviewed for test performed July 13, 1977.
- N1-ST-R8, September 13, 1978, Reactor Coolant and Primary Containment Isolation Valves Timing. Data reviewed for test performed June 9, 1977.
- N1-ST-W14-4, August 31, 1978, Fire Protection System - Weekly Operator Routine. Data reviewed for tests performed October 30, November 5, November 12 and November 19, 1978. (T.S. 4.6.8.a(1) only)
- N1-ST-C5A, May 19, 1977, Emergency Ventilation Flow Operability Test. Data reviewed for tests performed July 14 and July 15, 1977.
- N1-ST-C5, September 15, 1977, Secondary Containment Leak Rate/Reactor Building Ventilation Duct Radiation Monitor Test. Data reviewed for test performed April 11, 1977.
- N1-ST-C4, September 13, 1978, Containment Spray Air Flow for Spray Headers and Nozzles Test. Data reviewed for test performed April 10, 1977.
- N1-ST-W11, August 21, 1978, 125 VDC Battery Pilot Cell Voltage and Specific Gravity Test. Data reviewed for tests performed October 29, November 5, November 12 and November 20, 1978.
- N1-ST-Q1, January 9, 1979, Core Spray Pumps and Motor Operated Valves Operability Test. Data reviewed for tests performed January 21, February 14, May 10, August 11 and November 13, 1978.
- N1-ST-Q2, August 28, 1978, Control Rod Drive Pumps Flow Rate Test. Data reviewed for tests performed February 18, June 14, August 18 and November 18, 1978.
- N1-ST-M3, December 12, 1978, Suppression Chamber-Drywell Vacuum Relief Valves Exercising. Data reviewed for tests performed September 6, October 8, November 8 and December 8, 1978.



- NI-ST-M1, August 28, 1978, Liquid Poison Pumps Flow Rate Discharge Pressure Test. Data reviewed for tests performed September 8, October 7, November 8 and December 10, 1978.
- NI-ST-V4, Operability of Feedwater and Main Steam Line Power Operated Isolation Valves Test. Data reviewed for eight surveillances performed October 29, 1978 through November 23, 1978.

c. The following items were identified as a result of the above review:

- (1) Surveillance Procedure NI-ST-W14-4, Fire Protection System - Weekly Operator Routine, written in part to satisfy Technical Specification 4.6.8.a(1), did not include acceptance criteria covering the Cardox tank level and pressure requirements. The inspector noted, however, that for the tests reviewed, the Cardox tank level and pressure recorded values satisfied the Technical Specification requirement. A change was issued by the licensee representative prior to the exit interview to include the Technical Specification requirement in the acceptance criteria of the procedure. As the above condition was the only instance where the acceptance criteria was not provided the inspector considered this an isolated case.
- (2) During discussions with the licensee, the inspector noted that based on his review, it appeared that, generally, temporary changes were seldom used. It was stated that since the next refueling outage was close at hand and temporary changes may be more frequently used, it would be prudent to remind the appropriate personnel of the requirements for making temporary changes. The licensee acknowledged the inspector's comments. The inspector further stated that the area of temporary changes would be reviewed again during the upcoming outage.
- (3) The inspector noted that Surveillance Procedure NI-ST-Q6, Containment Spray and Raw Water Pump Operability Test, was written in part to satisfy Technical Specification 4.3.7.a(2) which requires verifying containment spray pump operability on a quarterly basis. The most recent quarterly test, conducted on November 11, 1978, did not appear to satisfy the acceptance criteria as set forth in the procedure for two of the four pumps tested. Containment Spray Pumps 111 and 121



had pump capacity values of 3100 gpm at 150 psig and 3200 gpm at 140 psig., respectively. The above capacities do not appear to satisfy the acceptance criteria in that the flow/head values are required to fall on the design analysis pump curve at a point that ensures 3000 gpm at a total head equal to or greater than 162.4 psig. The licensee stated that the above test values did not appear indicative of past results, and believed that a failure to adequately blowdown the essential transmitters as required by procedure, may be the reason for the unacceptable test results. It was further noted the test results were signed off by the performers without questioning the acceptability of the test. Also, the questionable test results were reviewed by Operations Department supervision and approved without comment or question.

The above condition represents an item of noncompliance with TS 4.3.7. a(2) in that the data as recorded in the procedure did not support pump operability and review of the data did not result in a retest or entrance into the required Limiting Condition for Operation. (220/79-02-01)

Based on a telephone discussion held with the licensee representative, operability of Containment Spray Pumps 111 and 121 was demonstrated satisfactorily by performance of Surveillance Procedure N1-ST-Q6 on January 26, 1979.

5. Inspector Witnessing of Surveillance Tests

- a. The inspector witnessed the performance of surveillance testing of selected components to verify the following.
 - Surveillance test procedure was available and in use.
 - Special test equipment, if required by procedure, was calibrated and in use.
 - Test prerequisites were met.
- b. The inspector witnessed the performance of:
 - N1-ISP-IC-68, Reactor Building to Torus Vacuum Relief Valves Instrument Channel Test, Revision 0, February 24, 1978. Test performed January 24, 1979.

No items of noncompliance were identified.



6. Technician Qualification

The inspector reviewed the qualification records of four currently assigned technicians having responsibility for surveillance testing of safety related systems and components. This review was performed to verify that the individual's experience level and training were in accordance with ANSI N18.1-1971, "Selection and Training of Nuclear Power Plant Personnel."

No items of noncompliance were identified.

7. Facility Tour

On several occasions during the inspection, tours of the facility were conducted of both the turbine and reactor buildings. During the tours, the inspector discussed plant operations and observed housekeeping, radiation control measures, fire protection equipment, monitoring equipment and controls for Technical Specification compliance. In addition, the inspector observed control room operations on both day and evening shifts for control room manning, shift turnovers, and facility operation in accordance with administrative and Technical Specification requirements.

No items of noncompliance were identified.

8. Exit Interview

The inspector met with licensee representatives (denoted in paragraph 1) at the conclusion of the inspection on January 26, 1979. The scope and findings of the inspection were summarized. The findings were also reviewed in a subsequent telephone discussion between Mr. R. Zimmerman, NRC:RI and Mr. J. Shea on January 29, 1979.

