

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-220

UNIT Nine Mile Point Unit

DATE _____

COMPLETED BY T.J. Perkins

TELEPHONE 315-343-2110, Ex. 1312

MONTH February

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>455</u>	17	<u>422</u>
2	<u>452</u>	18	<u>419</u>
3	<u>447</u>	19	<u>364</u>
4	<u>444</u>	20	<u>-</u>
5	<u>452</u>	21	<u>-</u>
6	<u>442</u>	22	<u>293</u>
7	<u>443</u>	23	<u>407</u>
8	<u>440</u>	24	<u>367</u>
9	<u>436</u>	25	<u>392</u>
10	<u>437</u>	26	<u>422</u>
11	<u>436</u>	27	<u>413</u>
12	<u>432</u>	28	<u>411</u>
13	<u>430</u>	29	<u>_____</u>
14	<u>429</u>	30	<u>_____</u>
15	<u>427</u>	31	<u>_____</u>
16	<u>424</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

7903100240

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-220
 UNIT NAME Nine Mile Point Unit #1
 DATE _____
 COMPLETED BY T.J. Perkins
 TELEPHONE 315-343-2110, Ext. 1312

REPORT MONTH February

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
2	2/19/79	F	53.	A	1				Reactor Drain Valve Packing Leak Repacked Valve

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT NUCLEAR STATION UNIT #1

NARRATIVE OF OPERATING EXPERIENCE

February 1979

The unit operated at 64% electrical capacity factor with an availability of 92.1% for the month. The monthly operational history is as follows:

- February 1-18 All rods withdrawn, 100% recirculation flow. End of cycle coastdown from approximately 467 MWe to approximately 440 MWe.
- February 19 At 2000 started reactor shutdown due to high drywell leakage. Generator off line at 2136.
- February 20 Reactor temperature at 185°F at 1300. Maintenance repacked reactor drain valve which was cause of leakage. At 1858 reactor startup commenced. At 2029 the reactor scrambled by neutron monitoring system during startup.
- February 21 Reactor startup commenced at 1010.
- February 22 Generator on line at 0210. All rods withdrawn at 0620. Increasing load with recirculation flow according to PCIOMR limits.
- February 23 Reached full recirculation flow but had to reduce load approximately 80 MWe to remove 12 reactor recirculation pump from service. The M-G set pump was rebrushed and collector ring turned down and #12 recirculation pump remained out of service through Feb. 24.
- February 25 #12 Reactor recirculation pump M-G set cleared by maintenance. #12 recirculation pump back in service. Load increased.
- February 26-28 Coastdown continued.

Maintenance was performed on #11 and #12 Reactor Feedwater pumps. #12 feedwater pump was out of service for overhaul from Feb. 5 through Feb. 10. #11 feedwater pump was out of service for overhaul from Feb. 12 through Feb. 16. 121 Core Spray Topping Pump was removed from service on Feb. 20-28 for mechanical seal replacement. #11 feedwater pump removed from service Feb. 27 for oil relief valve calibration. Returned to service on Feb. 28. #11 Liquid Poison Pump was out of service on Feb. 27 for repacking. It was returned to service the same day.

Narrative (continued)

The following change to the station was completed on February 2, 1979. The modification was reviewed by the Site Operation Review Committee and the Safety Review and Audit Board. It was determined that the modification does not constitute a change in the Technical Specifications or an unreviewed safety question in accordance with 10 CFR 50.59:

The Refuel Bridge wiring was changed on the Bridge and Trolley Auxilliary Hoists to make the 400 lb load limit effective under all hoisting conditions. Formerly this feature was, in effect, only when the bridge was over the Reactor Core with one rod out.