

U.S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT

Region I

Report No. 78-21
Docket No. 50-220
License No. DPR-63 Priority -- Category C

Licensee: Niagara Mohawk Power Corporation
300 Erie Boulevard West
Syracuse, New York

Facility Name: Nine Mile Point Nuclear Station, Unit 1

Inspection at: Scriba, New York

Inspection conducted: December 11-14, 1978

Inspectors: *R. Architzel* 1/9/79
R. Architzel, Reactor Inspector date signed
Edward J. Freeman 1/12/79
T. Stetka, Reactor Inspector date signed

Approved by: *E. C. McCabe, Jr.* 1/12/79
E. C. McCabe, Jr., Chief, Reactor Projects date signed
Section No. 2, RO&NS Branch

Inspection Summary

Inspection on December 11-14, 1978 (Report No. 50-220/78-21)

Areas Inspected: Routine, unannounced inspection of organization and administration, review committee activities, and plant operations. Licensee action on previous inspection findings was examined and NRC in office review of various licensee reports was documented. A plant tour was conducted. The inspection involved 58 inspector-hours onsite by two regional based NRC inspectors.

Results: Of the three areas inspected, no items of noncompliance were identified in two areas; one item of noncompliance was identified in the other (Deficiency, failure to review and approve procedure change, paragraph 4.a).

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DETAILS

1. Persons Contacted

The following technical and supervisory level personnel were contacted:

- J. Aldrich, Supervisor-Training
- W. Bandla, Station Shift Supervisor
- R. Coon, Assistant Supervisor, I&C
- R. Jutras, Technical Assistant - Station Superintendent
- *T. Lempges, General Superintendent - Nuclear Generation
- J. Pavel, Assistant to General Superintendent
- *T. Perkins, Station Superintendent
- R. Raymond, Fire Protection Coordinator
- M. Silliman, Results Superintendent
- *C. Stuart, Supervisor - Operations
- *B. Taylor, Supervisor - I&C

*denotes those present at the exit interview.

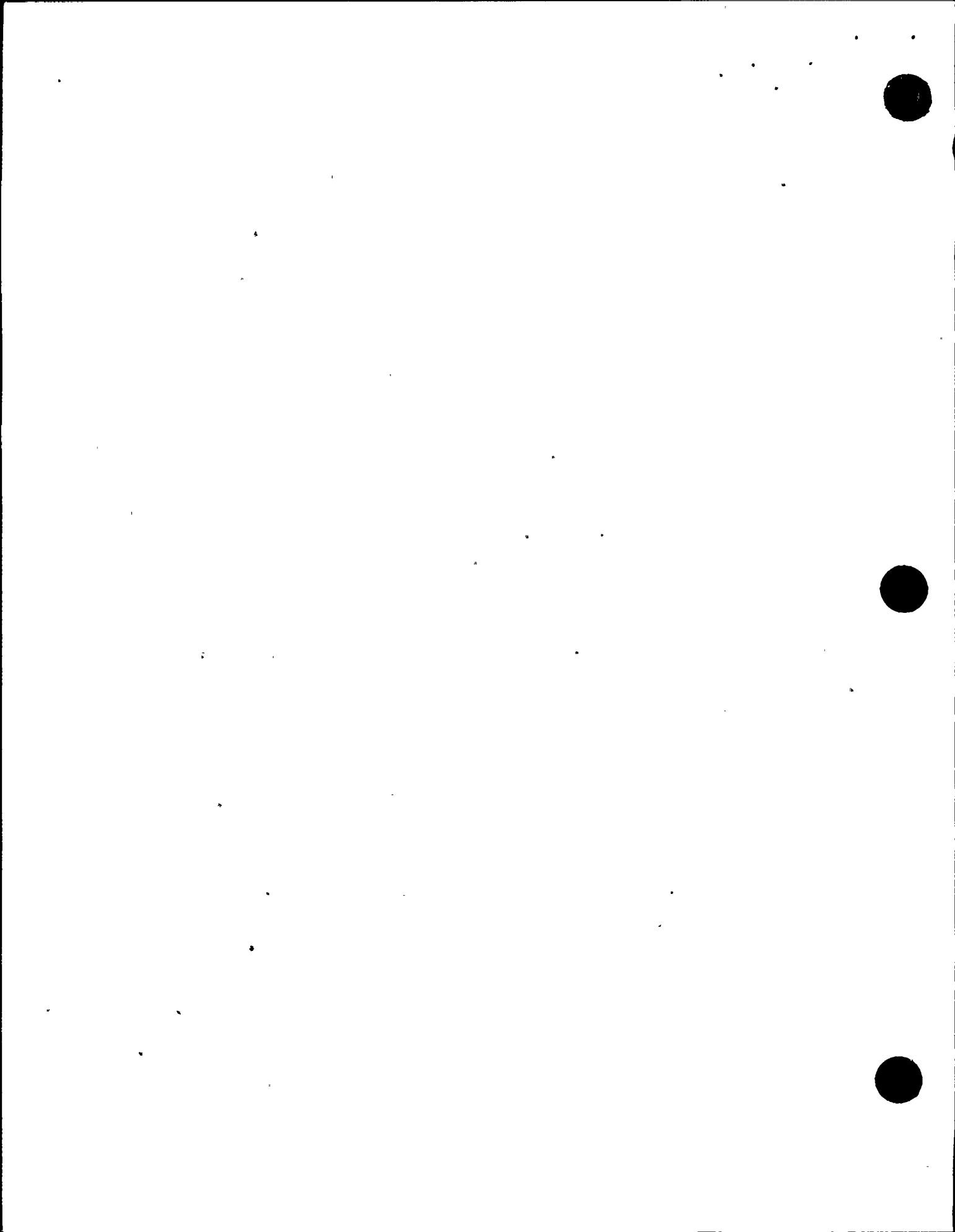
Other licensee employees were also interviewed.

2. Licensee Action on Previous Inspection Findings

(Closed) Unresolved Item (78-17-01): The inspector reviewed I&C technician training records to verify that discussions with the technicians were held concerning Circular 77-13.

(Open) Unresolved Item (78-17-02): Procedures N1-ISP-RE-22 for calibration of the Main Steam High Flow switches and N1-ISP-RE-04 for calibration of the Drywell High Pressure switches were revised to provide setpoints that are sufficiently conservative to prevent exceeding TS limits. The licensee has more procedures to revise and will complete these revisions by the end of the upcoming re-fueling outage scheduled to begin in March, 1979.

(Closed) Unresolved Item (78-08-01): The licensee revised procedure N1-OP-43, "Startup and Shutdown Procedure," to require verification of Rod Worth Minimizer operability prior to decreasing power below 20% or the availability of a qualified person.



(Closed) Unresolved Item (78-01-02): This item, setting of instruments within T.S. limits as opposed to tolerances specified in the T.S. basis, is identical to item 78-17-02 discussed above. The licensee stated that instrument set points would be established which are conservative with respect to surveillance requirements. Revision of procedures will be followed by unresolved item 78-17-02.

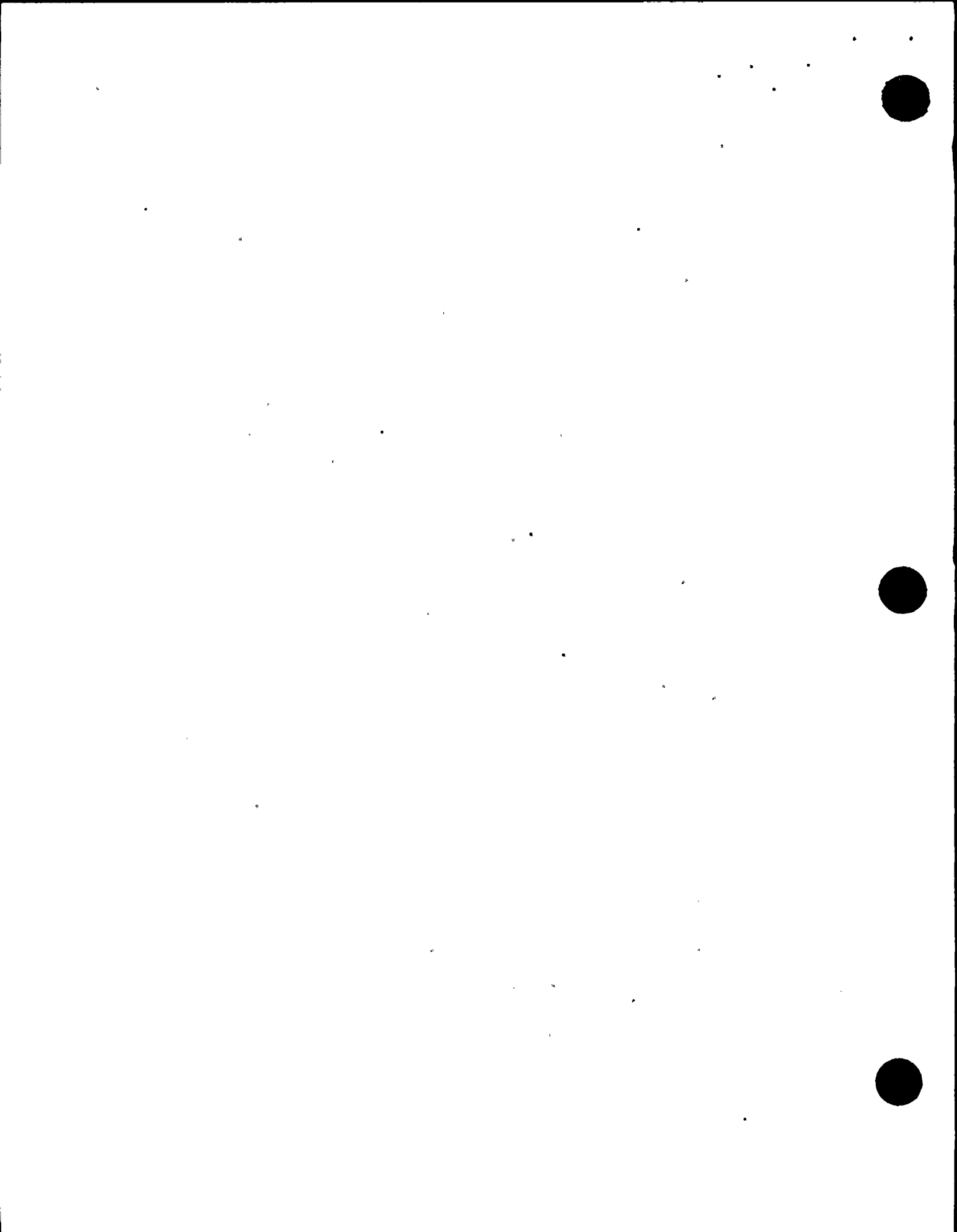
(Closed) Unresolved Item (78-16-03) Re-examination of pressure transmitters and level instruments during this inspection demonstrated that covers were in place and secured.

(Open) IE Circular 78-08, Qualification of Equipment for Post Accident Environment. The licensee's evaluation of design and as built conditions remains to be completed. The inspector informed the licensee of the NRC's position that position indication circuitry for containment isolation valves must be qualified. The licensee acknowledged the inspectors comments and stated that this item would be reviewed together with the evaluation for Circular 78-08 scheduled to be completed during the 1979 refueling outage.

3. Organization and Administration

- a. The facility organization and administration were reviewed to ascertain whether the licensee's onsite and offsite safety review committees and operating shift manning are in conformance with the requirements of the Technical Specifications (TS). This review verified that:
- The structure, membership, and qualifications of the Site Operations Review Committee (SORC), the licensee's onsite committee, and the Safety Review and Audit Board (SRAB), the licensee's offsite committee, are as described in the TS;
 - Facility management personnel qualification levels, and personnel authorities and responsibilities are in conformance with the TS and applicable standards; and,
 - Operations shift crew composition and licensed personnel requirements comply with the TS.

During this review, the inspector was unable to identify the "Staff Engineer - Mechanical or Electrical" member of the SRAB, required by TS 6.5.2.2. A subsequent telephone conversation with licensee personnel on December 18 identified this individual and the inspector had no further questions regarding this item.



With the exception of the following unresolved items, no inadequacies were identified.

- b. The Technical Specifications require the alternate SRAB members to be designated in writing. Three SRAB alternate members that participated in SRAB meetings 78-01 and 78-02 did not have these appointing letters available for review. This item is unresolved (220/78-21-01).
- c. Review of the license for a present watch-standing operator revealed that the license expired on October 15, 1978. The licensee requested renewal of this license on September 6, 1978, however the renewal was delayed by the NRC Operator Licensing Branch due to unanswered medical questions. The licensee submitted the additional requested information and the license renewal is still pending. This item is unresolved (220/78-21-02).

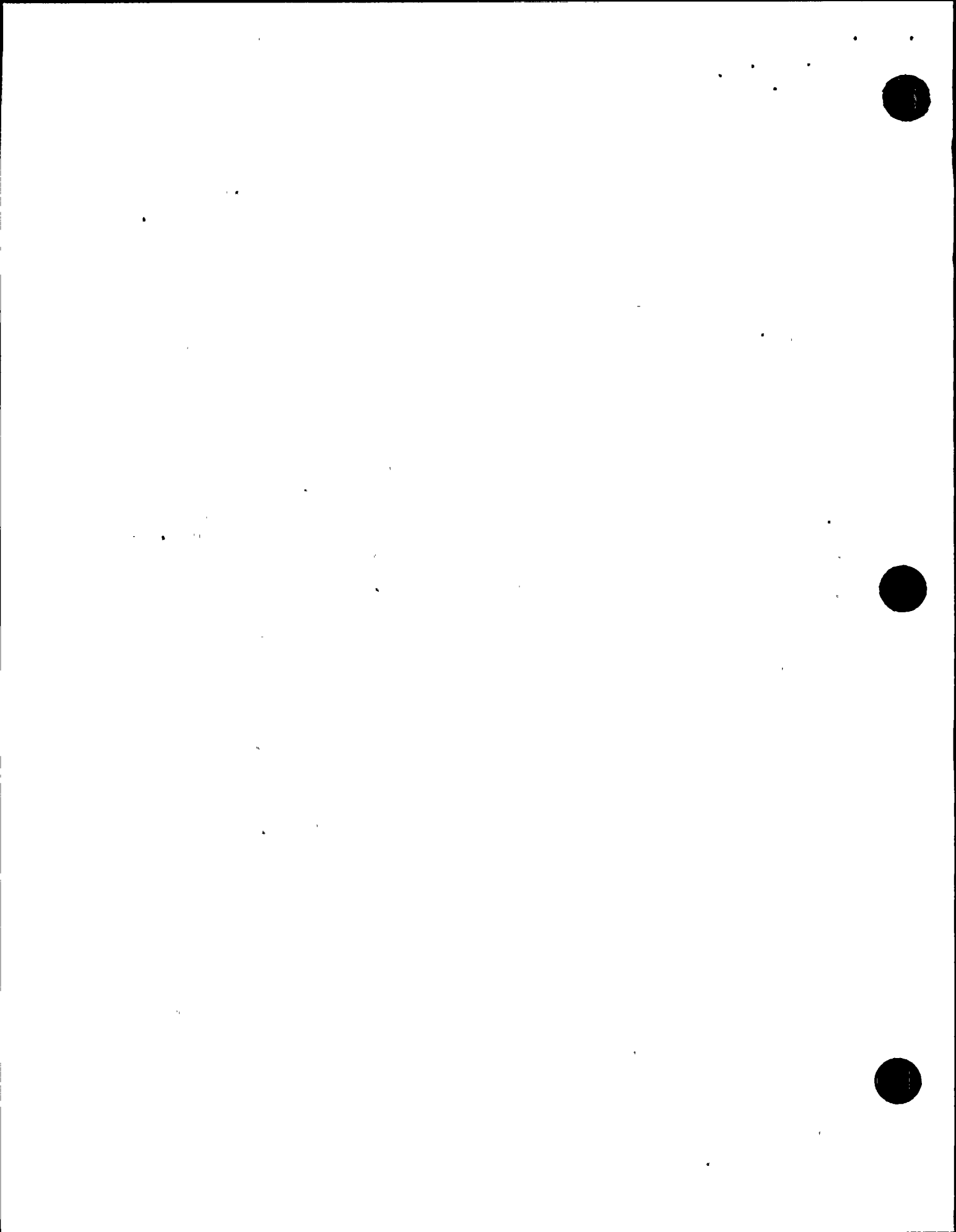
4. Review of Plant Operations

a. Shift Logs and Operating Records

The inspector reviewed the records listed below for compliance with the licensee's administrative procedure for Operations Department Logs.

- (1) Station Shift Supervisor's (SSS) Log for the period 10/25/78 - 12/8/78.
- (2) Control Station Operator's (CSO) Log for the period 10/22/78 - 11/23/78.
- (3) Jumper and Lifted Lead Log for all active entries.
- (4) Station Shift Supervisor Instructions (SSSI) for the period 11/1/78 - 12/8/78.

The SSSI of November 9, 1978 delineated a procedure for operating the Nine Mile Point Unit 1 (NMP-1) fire system to support construction activities at Nine Mile Point Unit 2 (NMP-2). This procedure, which required operation of the NMP-1 fire pump to support the NMP-2 fire system while piping modifications were made, directed cross-connection between the Unit 1



and Unit 2 fire systems. The procedure was implemented on November 10 and was completed in approximately 6 hours. Operating Procedure NI-OP-21, entitled Fire Protection System-Revision 1, does not provide for cross-connection of the Unit 1 and Unit 2 fire system and the SSSI's are not reviewed and approved by the Site Operations Review Committee (SORC).

Failure of the SORC to review the SSSI procedure and of the General Superintendent Nuclear Generation to approve this procedure prior to implementation is contrary to the requirements of TS 6.8.2. This is an item of noncompliance (220/78-21-03).

b. Plant Tour

The inspectors observed control room operations and conducted a facility tour accompanied by a licensee representative. The tour included the Reactor Building, Turbine Building, Diesel Generator Rooms, and the Off Gas Building. The following were among the items observed:

- Radiation Controls properly established;
- No fluid leaks of significance;
- Pipe vibrations not excessive;
- Control Room manning in accordance with Technical Specifications;
- Reasons for lighted annunciators in the Control Room; and,
- Housekeeping, including examination for fire hazards.

The inspector questioned the licensee concerning positioning of the air start isolation valve for the Emergency Diesel Generators, which were open. Closure of these valves would result in inoperability of the Diesels (failure to start) without annunciation of the disabled condition. The licensee stated that these valves would be added to the locked valve lists and required to be locked open to ensure Diesel Engine operability. This item (220/78-21-04) will be reexamined by the NRC during a subsequent inspection.



5. Review Committees

a. Site Operations Review Committee (SORC)

A review was conducted to determine whether the SORC performed in conformance with the requirements of the facility Technical Specifications.

The following meeting minutes were reviewed:

78-01 (1/5/78) to 78-50 (11/16/78)

Results of the review showed that meetings were conducted during the previous year at the frequency required by the Technical Specifications and meeting membership satisfied the quorum requirements.

Proposed Technical Specification changes, violations of Technical Specifications identified in NRC inspection reports, facility operations, facility changes, and License Event Reports were reviewed by the SORC as required.

The following items were verified to have been reviewed (SORC Meeting reviewed in parenthesis).

-- Prompt LERs: 78-01 (78-01), 78-09 (78-07), 78-19 (78-26), 78-29 (78-31), 78-34 (78-44), 78-35 (78-48), 78-36 (78-47).

-- 30 Day LERs: 78-02 (78-03), 78-03 (78-04), 78-07 (78-05), 78-14 (78-09), 78-22 (78-26).

-- Violations of Technical Specifications Identified in NRC Inspections (Inspection containing noncompliance, (Meeting(s) Reviewed)):

78-05 (78-19,29), 78-11 (78-41), 78-12 (78-44), 78-13 (78-44) 78-15 (78-51), 78-16 (78-51)

-- Modifications to Plant Systems:

N1. 78-01 (78-04), N1. 78-02 (78-05), N1. 78-03 (78-06), N1. 78-05 (78-15), N1. 78-06 (78-15), N1. 78-07 (78-15).



-- Proposed Technical Specification Changes:

Solenoid Actuated Pressure Relief Valves (78-03), Various Appendix A Changes, 3/17/78 (78-10), Various Appendix B Changes, 3/17/78 (78-10).

No unacceptable conditions were identified.

b. Safety Review and Audit Board (SRAB)

A review was conducted to determine whether the SRAB performed in conformance with the requirements of the facility Technical Specifications.

The following meeting minutes were reviewed: 78-01 (January 5, 1978) through 78-04 November 15, 1978).

Results of the review showed that meetings were conducted during the previous year in excess of the frequency required by the Technical Specifications and meeting membership satisfied the quorum requirements. Several proposed Technical Specification changes, violations of Technical Specifications, safety evaluations for selected changes performed without NRC review pursuant to 10 CFR 50.59, and prompt Licensee Event Reports were preselected by the inspector. Review of the meeting minutes showed that these items had been reviewed by the SRAB.

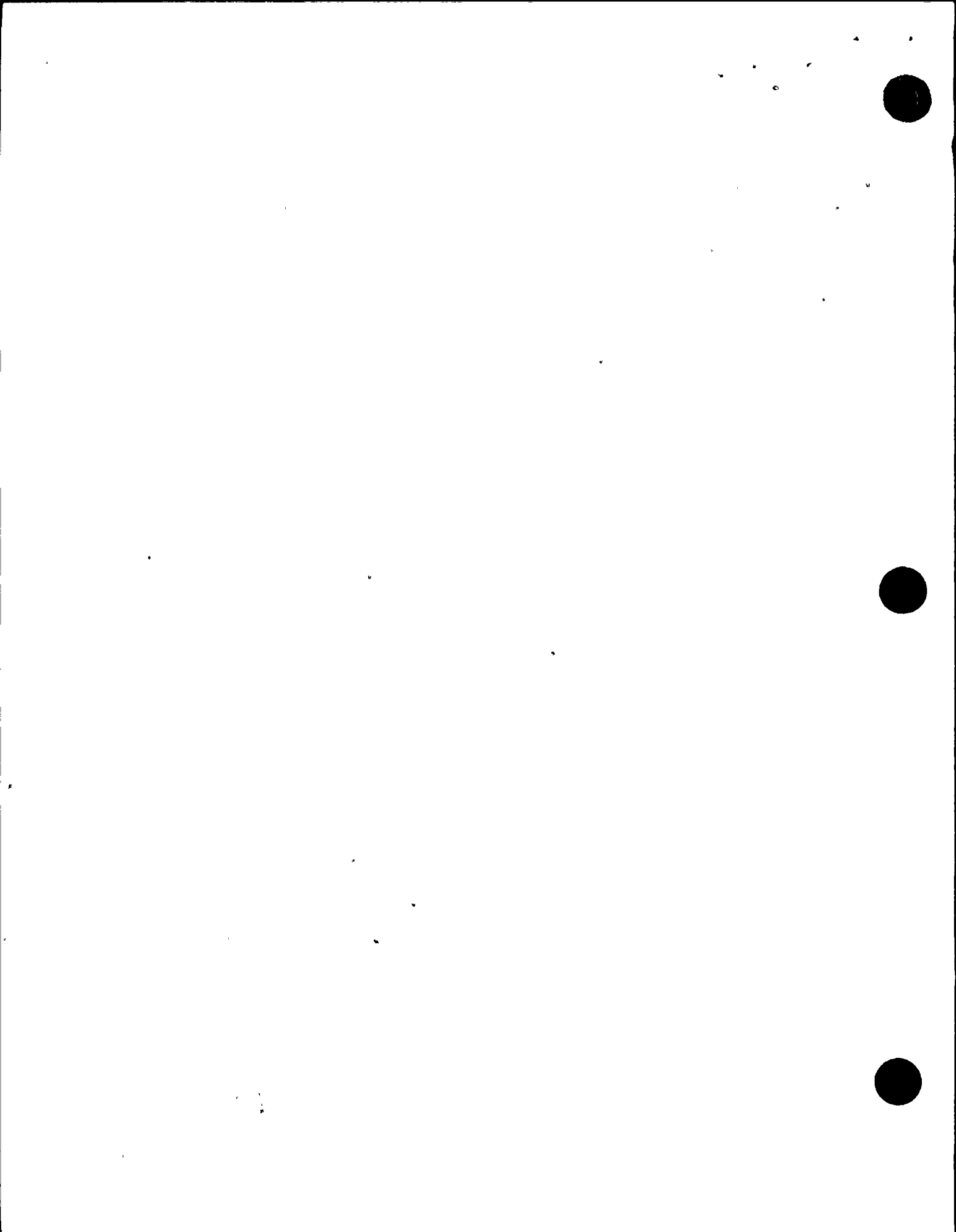
No unacceptable conditions were identified.

6. Licensee Event Report (LER) 78-35

This LER concerned setpoint drift of the eight Main Steam line high flow pressure switches to values greater than the 105 psid allowed by T.S. 3.6.2B. The licensee revised procedure N1-ISP-RC22 to require setting these instruments in a conservative fashion. Generic aspects of this LER remain open and will be followed under Unresolved Item 220/78-17-02 (See paragraph 2).

7. NRC In Office Review

- a. NRC: RI in office review of the LERs identified in the attached table has been completed during 1978 with no unacceptable conditions identified.

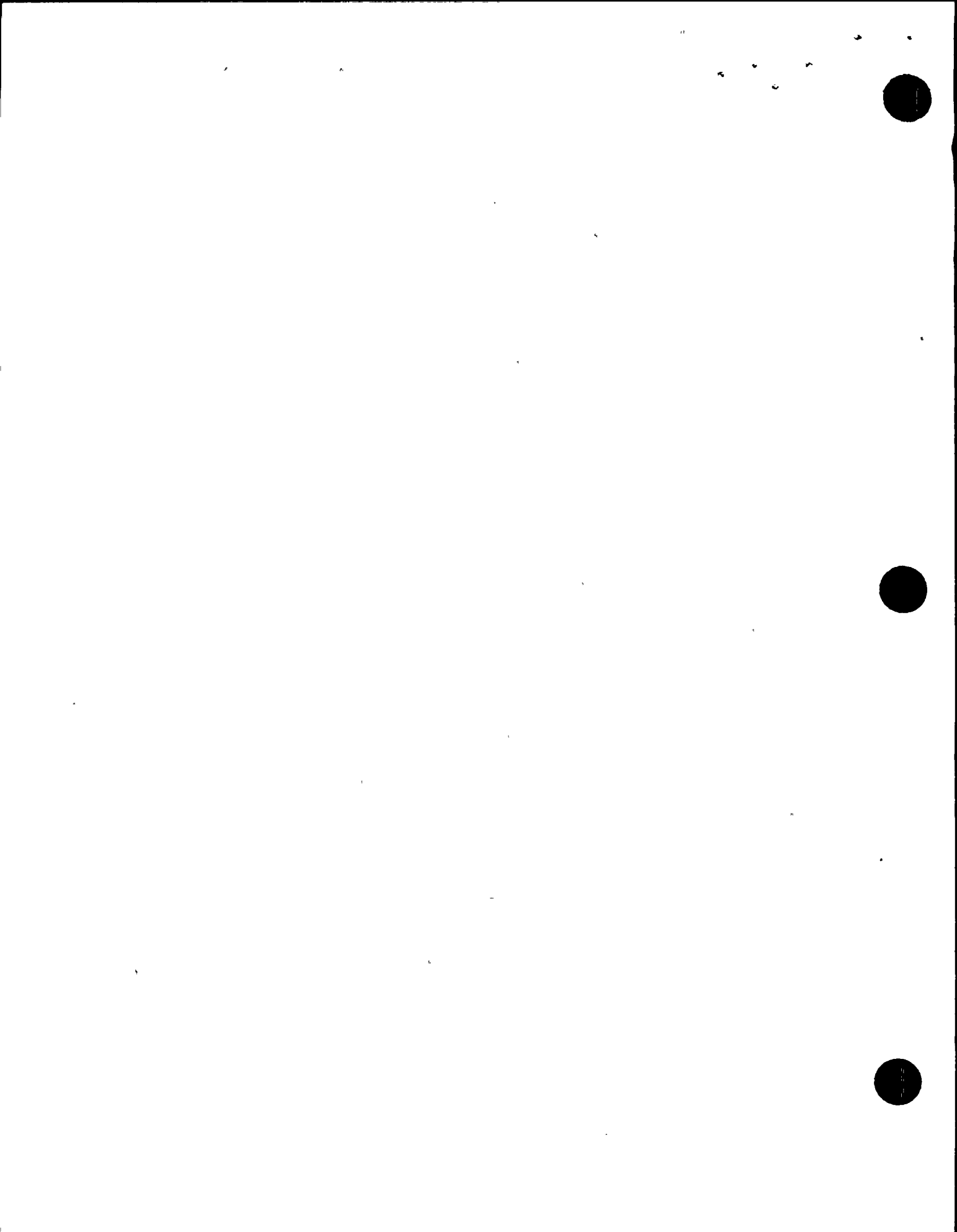


LER	DATED	SUBJECT
78-02	1/31/78	Conservative setpoint drift of drywell high pressure switches
78-03	1/31/78	Setpoint drift of main steam high flow switch
78-04	2/6/78	Reserve power breaker failed; system II HPCI inoperable
78-05	2/6/78	Outside MSIV 01-03 valve would not close
78-06	2/24/78	Setpoint drift on vacuum switch in vacuum relief system
78-07	2/28/78	Pinhole leaks found on liquid waste surge tank
78-08	2/28/78	Instrument drift on main steam line high flow indicating switch
78-10	2/28/78	Setpoint drift of drywell high pressure switches
78-11	3/15/78	Rx. Bldg. Ventilation Radiation Monitor failure
78-12	3/15/78	Setpoint drift of Rx. Lo-Lo-Lo water level switch
78-13	3/15/78	Conservative setpoint drift of drywell high pressure switches
78-14	3/28/78	Fuel transfer pump on 102 D/G would not fill day tank
78-15	3/28/78	#12 Control Room Air Treatment fan inoperable
78-16	4/7/78	Conservative setpoint drift of Rx Lo-Lo-Lo level switch
78-17	4/7/78	Setpoint drift of drywell high pressure switches
78-18	4/24/78	Setpoint drift of main steam high flow indicating switch
78-20	6/12/78	Setpoint drift of Rx Lo-Lo-Lo water level
78-21	6/12/78	Setpoint drift of drywell high pressure switches
78-22	6/12/78	Setpoint drift of turbine anticipatory trip bypass Rx scram
78-23	6/12/78	Two LPRM'S Cross Connected
78-24	6/21/78	Seven quarterly surveillance tests not performed
78-25	7/10/78	Drywell High Pressure Switch Drift
78-26	7/10/78	Small leaks found in sample and inst. pipes of waste concentrator
78-27	7/10/78	Setpoint drift of main steam line high flow indicating switches
78-28	7/25/78	Test interval for core spray power-operated valves exceeded
78-30	8/24/78	Feedwater Pump 29-03 OOS for routine maintenance
78-31	8/24/78	Liquid poison pumps unavailable due to drain valve repairs



78-32	8/24/78	APLHGR limit exceeded following Rx power increase
78-33	9/29/78	Setpoint drift on drywell high pressure switches
78-37	11/14/78	Setpoint Drift on Vacuum Relief System Vacuum Switch
78-38	11/14/78	Main Steam High Flow Transmitters (RE-22) Improperly Calibrated
78-39	11/14/78	Setpoint Drift On Rx Lo-Lo Water Level Switch

One item regarding format of the submitted report was discussed with the licensee. NUREG-0161, Instructions for Preparation of Data Entry Sheets, General Instructions paragraph 5, requests the inclusion of the specific requirement (e.g. 10 CFR, TS, license condition) for reporting the occurrence. This item is unresolved (220/78-21-05) pending inclusion of this reference in future LER's.



b. NRC: RI in office review of the following reports has been completed.

- January through November, 1978 Monthly Reports of Operating Statistics and Shutdown Experience.
- 1977 Report of Operating Experience and 10 CFR 50.59(b) Report (NMP letter dated February 27, 1978).

The inspector noted that a summary of safety related maintenance was not included in monthly reports following January, 1978. The inspector discussed this with the licensee by telephone on December 21, 1978. This item is unresolved (220/78-21-06) pending inclusion of safety related maintenance items in the monthly reports by the licensee.

8. Unresolved Item

An item about which more information is required to determine acceptability is considered unresolved. Paragraphs 3, 4, and 7 contain unresolved items.

9. Exit Interview

At the conclusion of the inspection the inspectors held a meeting (see paragraph 1 for attendees) to discuss the inspection scope and findings. The item of noncompliance and unresolved items were identified.

