

UNITED STATES OF AMERICA  
NUCLEAR REGULATORY COMMISSION

In the Matter of

NIAGARA MOHAWK POWER CORPORATION  
(Nine Mile Point Nuclear Station  
Unit No. 1)

Docket No. 50-220

APPLICATION FOR AMENDMENT

TO

OPERATING LICENSE

Pursuant to Section 50.90 of the regulations of the Nuclear Regulatory Commission, Niagara Mohawk Power Corporation, holder of Facility Operating License No. DPR-63, hereby requests that Section 4.3.3 and the Bases of the Technical Specifications set forth in Appendix A to that License be amended. Those proposed changes have been reviewed by the Site Operations Review Committee and the Safety Review and Audit Board.

The proposed Technical Specifications changes are set forth in Attachment A to this Application. Supporting information, which demonstrates that the proposed changes do not involve a significant hazards consideration, is set forth in Attachment B. The proposed changes would not authorize any change in the types or any increase in the amounts of effluents or any change in the authorized power level of the facility. Justification for classification of the amendment pursuant to 10CFR Section 170.22 is included as Attachment C. A check for the appropriate fee accompanies this Application.

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WHEREFORE, Applicant respectfully requests that Appendix A to Facility Operating License No. DPR-63 be amended in the form attached hereto as Attachment A.

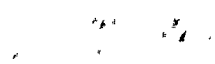
NIAGARA MOHAWK POWER CORPORATION

By Donald P. Dise  
Donald P. Dise  
Vice President - Engineering

Subscribed and sworn to before  
me on this 24 day of January,  
1979.

Phyllis D. Voytko  
NOTARY PUBLIC

PHYLLIS D. VOYTKO  
Notary Public in the State of New York  
Qualified in Onon. Co. No. 34-9485535  
My Commission Expires March 30, 1980



ATTACHMENT A

NIAGARA MOHAWK POWER CORPORATION

LICENSE DPR-63

DOCKET 50-220

Proposed Changes to Technical Specifications (Appendix A)

Attached are revised pages 136, 140 and 143. These revised pages have been retyped in their entirety and the marginal markings indicate the specific changes to the text.



LIMITING CONDITION FOR OPERATION

SURVEILLANCE REQUIREMENT

- (2) Subsequent leakage rate tests shall be performed without preliminary leak detection surveys or leak repairs immediately prior to or during the test, at an initial pressure of approximately 22 psig.
- (3) Leak repairs, if necessary to permit integrated leakage rate testing, shall be preceded by local leakage measurements. The leakage rate difference, prior to and after repair when corrected to  $P_t$  shall be added to the final integrated leakage rate result.
- (4) Closure of the containment isolation valves for the purpose of the test shall be accomplished by the means provided for normal operation of the valves.
- (5) After the containment test conditions have stabilized, the test duration shall not be less than eight hours for integrated leak rate measurements. The test shall be extended for sufficient duration, to verify by a supplemental test method the accuracy of the integrated leak rate test results.





LIMITING CONDITION FOR OPERATION

SURVEILLANCE REQUIREMENT

(ii) any one penetration or  
isolation valve

5%  $L_{to}$  (22)

(c) primary containment air purge  
penetrations and reactor  
building to torus vacuum  
relief valves

50%  $L_{to}$  (22)

g. Inspection

The accessible interior surfaces of the  
drywell shall be visually inspected each  
operating cycle for evidence of deterioration.



- (1) Appendix E, FSAR
- (2) Volume 1, Section VI, FSAR
- (3) TID-20583, Leakage Characteristics of Steel Containment Vessels and the Analysis of Leakage Rate Determinations.
- (4) 10CFR50 Appendix J, "Reactor Containment Leakage Testing for Water Cooled Power Reactors".



ATTACHMENT B

NIAGARA MOHAWK POWER CORPORATION

License No. DPR-63

Docket No. 50-220

Supporting Information

Currently, Specification 4.3.3a(5) requires that the integrated leak rate test to be for a duration of not less than 24 hours. However, improved instrumentation and test methods now make it possible to obtain a statistically acceptable set of data points during a shorter test period.

The new test method utilizes the Bechtel Corporation procedure submitted and approved by the NRC in Topical Report BN-TOP-1, Revision 1, dated November 1, 1972 and titled "Testing Criteria for Integrated Leakage Rate Testing of Primary Containment Structures for Nuclear Power Plants." The Bechtel test procedure is a computerized technique which obtains sufficient data points in eight hours to meet the required statistical analysis accuracy.

Specification 4.3.3g currently requires the primary containment to be continuously monitored for gross leakage by monitoring the nitrogen make-up requirements of the inerting system. This method of continuous monitoring is not a practical system of determining if leakage occurs for the following reasons:

1. Normal changes in atmospheric pressure can effect the nitrogen make-up required to maintain the containment inerted and pressurized.
2. During power operation, nitrogen is continuously being bled and fed to maintain the correct differential pressure between the drywell and torus.

Because of the above, it is proposed to delete the requirement for continuous leak rate monitoring.



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ATTACHMENT C

NIAGARA MOHAWK POWER CORPORATION

LICENSE DPR-63

DOCKET NO. 50-220

Amendment Classification

The proposed amendment to the Operating License has been evaluated and determined to fall within the definition of Class II of 10CFR 170.22 thereby requiring a fee of one thousand two hundred dollars (\$1,200.00).

The proposed amendment for Nine Mile Point Unit 1 is administrative in nature and does not have any safety or environmental significance. Therefore, it meets the requirements of Class II of 10CFR 170.22.



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