

U.S. NUCLEAR REGULATORY COMMISSION  
OFFICE OF INSPECTION AND ENFORCEMENT

Region I

Report No. 50-220/78-17

Docket No. 50-220

License No. DPR-63 Priority -- Category C

Licensee: Niagara Mohawk Power Corporation

300 Erie Boulevard West

Syracuse, New York 13202

Facility Name: Nine Mile Point Nuclear Station, Unit 1

Inspection at: Scriba, New York

Inspection conducted: October 18-20, 1978

Inspectors: Jerry E. Briggs, SR  
T. F. Stetka, Reactor Inspector

11/14/78  
date signed

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Approved by: E. C. McCabe, Jr.  
E. C. McCabe, Chief, Reactor Projects Section  
No. 2, RO&NS Branch

11/14/78  
date signed

Inspection Summary:

Inspection on October 18-20, 1978 (Report No. 50-220/78-17)

Areas Inspected: Routine, unannounced inspection of plant operations including a plant tour; licensee action on IE Bulletins and Circulars; selected licensee events; control rod drive scram times; reactor protection system power supply surveillance; and licensee action on previous inspection items. The inspection involved 26 inspector-hours onsite by one NRC regional based inspector.

Results: No items of noncompliance were identified.

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## DETAILS

### 1. Persons Contacted

- \* R. Coones, Assistant I&C Supervisor
- K. Dahlberg, Electrical Maintenance Supervisor
- W. Drews, Reactor Analyst Supervisor
- \* T. Lempges, General Superintendent of Nuclear Generation
- \* T. Perkins, Station Superintendent
- \* C. Stuart, Operations Supervisor

Other licensee employees were contacted during the inspection including personnel of the operations and administrative staff.

\* present at the exit interview.

### 2. Licensee Action on Previous Inspection Report Items

(Closed) Noncompliance (77-09-01): The inspector verified by review of SORC minutes, procedure APN #5 and the associated changes and review of an internal memorandum that corrective action, as stated in the licensee's response letter of July 13, 1977 was complete.

(Closed) Noncompliance (77-14-01): The due dates for all LER's are now entered on a computerized action list to assure timely issue of the required reports. The inspector verified the establishment and use of this list by the licensee.

(Closed) Unresolved Item (77-18-01): NRC review of this item showed that the licensee's practice of making the channel inoperative for a time period not to exceed two hours during the performance of Technical Specification (TS) required surveillance testing is adequate. This practice is consistent with the requirements of the BWR Standard Technical Specifications.

(Closed) Unresolved Item (77-18-02): The licensee has requested a TS change that will clarify plant operation if MCPR is exceeded. The change is presently under review by NRR.



### 3. Review of Plant Operations

The inspector observed Control Room operations, reviewed process computer printouts of core limits and the reactivity anomalies curve and conducted a plant tour accompanied by a licensee representative. This tour included the Reactor Building, Turbine Building, Diesel Generator Rooms, Battery Rooms, Traveling Screen House and the Offgas Building. The following items were observed:

a. Monitoring instrumentation listed below was observed to determine that the indicated parameters were in accordance with Technical Specification limits.

- Reactor Pressure;
- Standby Liquid Control System poison tank level and temperature;
- Condenser Hotwell Level;
- Condensate Storage Tank levels;
- Reactor Water Conductivity;
- Reactor Building access and integrity;
- Reactor Water Leakage;
- Spent Fuel Pool temperatures and level;
- Diesel Generator fuel oil levels; and,
- Fire Protection System CO<sub>2</sub> Tank volume and pressure.

No inadequacies were identified.

b. Annunciator alarms were observed by the inspector and selected alarms were discussed with the Shift Supervisor. The Shift Supervisor was knowledgeable of the alarm conditions and no inadequacies were identified.

c. The inspector verified that staffing for the operating shift was in accordance with the Technical Specifications with regard to numbers and licenses.



- d. Radiation control zones observed were properly established and identified. Additionally, the inspector reviewed the conditions of step off pads, disposal of SWP clothing and area posting. No inadequacies were identified.
- e. Plant housekeeping conditions were observed with respect to storage of material and components to determine if safety and fire hazards existed.

The inspector observed housekeeping. The Refuel Floor, Core Spray Pump Rooms and Containment Spray Pump Rooms have debris and are in need of a general cleanup. These housekeeping problems were identified to the licensee.

Plant housekeeping problems were previously identified in IE Inspection Report 78-16 and resulted in an item of noncompliance (220/78-16-01). These areas will be reviewed during subsequent inspections for consistency with the licensee's corrective action with respect to the item of noncompliance.

- f. No significant fluid leaks were noted.
- g. No excessive piping vibration was noted.
- h. The position of valves in the Emergency Diesel Start, Core Spray, Containment Spray and Standby Liquid Control Systems were observed to be in the positions required by the Technical Specifications.

#### 4. Closeout of IE Bulletins and Circulars

- a. The inspector reviewed the licensee's action with respect to Bulletin IEB 77-02, Potential Failure Mechanism in Certain Westinghouse AR Relays with Latch Attachments, to verify the licensee's responses were timely, accurate and adequate. No problem was identified with the licensee's response and actions.
- b. The inspector reviewed the licensee's action with respect to the following IE Circulars to determine if the licensee had received them and had taken appropriate action:





- IEC 77-09, Improper Fuse Coordination in BWR Standby Liquid Control System Control Circuits;
- IEC 77-13, Reactor Safety Signals Negated During Testing;
- IEC 77-14, Separation of Contaminated Water Systems from Noncontaminated Plant Systems;
- IEC 77-15, Degradation of Fuel Oil Flow to the Emergency Diesel Generator; and,
- IEC 77-16, Emergency Diesel Generator Electrical Trip Lock-Out Features.

Except for the unresolved item identified below for Circular 77-13, no inadequacies were identified.

- c. Circular 77-13 was reviewed during Site Operations Review Committee (SORC) meeting 77-71, however, it appears that discussions concerning this circular with licensee I&C technicians have not been accomplished. A licensee representative stated that he believed that such discussions were conducted, however, the supervisor involved was not available.

This item is unresolved (220/78-17-01).

## 5. Review of Reportable Occurrences

- a. The following Licensee Event Reports (LERs) were reviewed by the inspector:

78-23	78-29	78-32
78-25	78-31	78-33

The review consisted of verifying that:

- The report accurately describes the event;
- The safety significance of the event is as reported;
- The report is accurate as to cause;
- The report satisfies the reporting requirement with respect to information provided and timing of submittal;



- Corrective action is appropriate;
- Action has been taken; and,
- The event was reviewed and evaluated by the SORC.

With the exception of the unresolved item identified below, no inadequacies were identified.

- b. Review of LER's 78-25, 29 and 33 indicate consistent drift problems with high drywell pressure switches RE04A, B, C and D. On one instance (LER 78-29 dated July 24, 1978), three of the four pressure switches were found to have drifted in a less conservative direction thus violating the Technical Specification (TS).

The inspector asked the licensee whether setting of the switches to a more conservative point was a feasible means of assuring that switch drift did not result in nonconservative out of specification activation. Licensee evaluation of this consideration resulted in the conclusion that such action is feasible (communicated to the inspector by telephone on October 23, 1978). In addition, the licensee stated that other instrumentation that have been prone to drift problems will be reset in a sufficiently conservative direction to assure that TS limits will not be exceeded. This item is unresolved (220/78-17-02) and the setpoint changes will be examined during subsequent inspections.

The licensee also stated that these high drywell pressure switches would be replaced during the upcoming Refueling Outage scheduled for March, 1979. This item (220/78-17-03) is unresolved pending replacement of these switches.

#### 6. Review of Control Rod Drive (CRD) Scram Time Problems

A report to the NRC identified instances of slowed CRD scram times due to water in the air operated scram valve air system.

The inspector queried the licensee about such problems. The licensee stated he had not experienced this problem and that their CRD scram times were consistent and well within Technical Specification requirements.



A review of scram time records was conducted and no items of noncompliance were identified.

The inspector had no further questions on this item.

7. Review of Reactor Protection System (RPS) Power Supply Surveillance Program

During a review of the E. I. Hatch Unit 2 nuclear power plant (DN 50-366) reactor protection system (RPS) power supply, the NRC staff identified certain specific deficiencies in the design of the RPS motor generator voltage regulator. As a result of this finding, the NRC staff requested facilities with similar design RPS power supplies to perform a specific surveillance and calibration program on the voltage regulators to assure proper operation.

The Nine Mile Point surveillance and calibration program relative to these voltage regulators was reviewed to verify that the necessary actions were being accomplished.

No inadequacies were identified.

8. Unresolved Items

Items about which more information is required to determine acceptability are considered unresolved. Paragraphs 4.c and 5.b of this report contain unresolved items.

9. Exit Interview

An exit interview was conducted on October 20, 1978 with licensee representatives (denoted in Paragraph 1) at the conclusion of the inspection. The inspector summarized the purpose and scope of the inspection and discussed the findings.

