

U.S. NUCLEAR REGULATORY COMMISSION OFFICE OF INSPECTION AND ENFORCEMENT

Region I

Report No. 50-410/78-07
Docket No. 50-410
License No. CPPR-112 Priority Category A
Licensee: Niagara Mohawk Power Corporation
300 Erie Boulevard, West
Syracuse, New York 13202
Facility Name: Nine Mile Point Nuclear Station, Unit 2
Inspection at: Scriba, New York and Syracuse, New York
Inspection conducted: September 26-28, 1978
Inspectors: Lewis / allful / date signed
L. Mail on , Mediciol Trispector
W. H. Bateman, Reactor Inspector date signed
date signed
Approved by: 11.11. Mattern 12.1478 date signed
R. W. McGaughy Chief, Construction Projects Section, RC & ES Branch
Inspection Summary:
Inspection on September 26-28, 1978 (Report No. 50-410/78-07)
Areas Inspected: Routine, unannounced inspection by a regional based inspector of preventive maintenance program during shutdown of construction; the QC program for "risk released to the construction of the construction of
of ocuinment, yender decumentation for satety related equipment; and the Status of Out-
standing items. The inspection involved 46 inspector-hours on site and at the licensee's office by two NRC inspectors.
Results: No items of noncompliance were identified.



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DETAILS

1. Persons Contacted

*R. J. Aldrich, QA Engineer

*J. J. Bebko, Project QA Engineer

*W. M. Bryant, Manager, Quality Assurance

*R. A. Dowd, Supervisor, Quality Assurance

*C. D. Terry, Senior Site QA Representative

K. D. Ward, QA Engineer

Stone and Webster Engineering

T. Bowden, Field QC, Structural

B. W. Gatlin, Superintendent Field QC

L. Martiniano, Senior Field Engineer

K. Schinners, QC Data Control Supervisor

The inspector also interviewed other licensee and contractor employees during the inspection.

*denotes those present at the exit interview.

2. Plant Tour

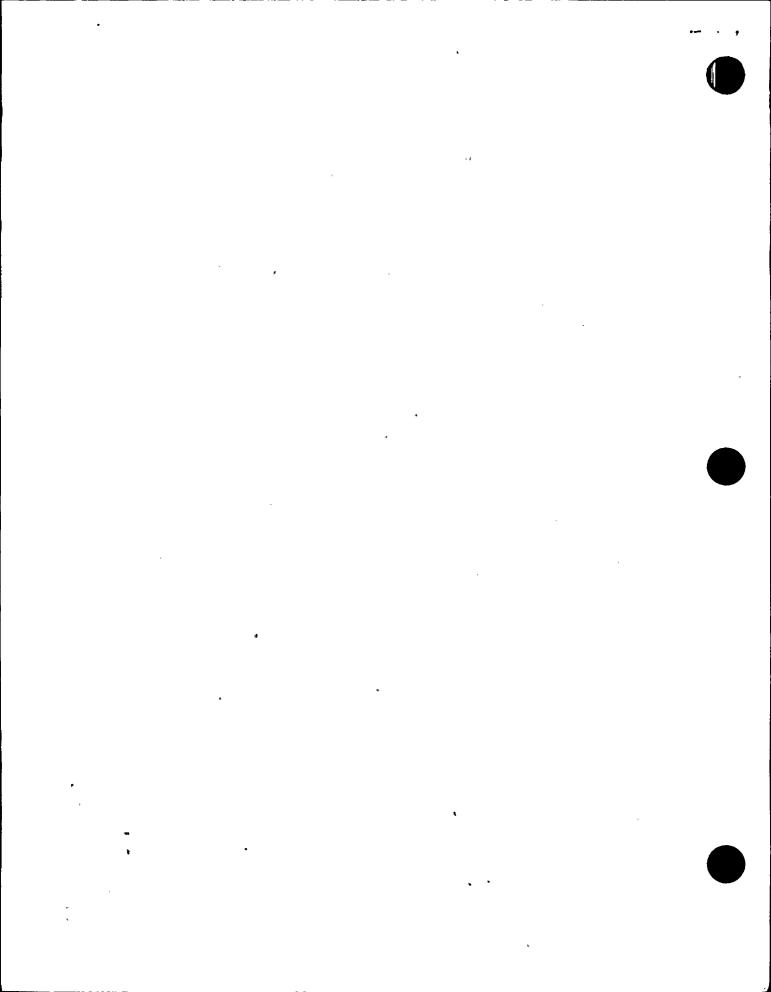
The inspectors made a tour of the site to observe work activites in progress, completed work and the status of construction. The inspector examined work items for any obvious defects or noncompliances with regulatory requirements and observed QC activities and evidence of quality control of the work. Specific activities observed by the inspector included fit-up and welding of containment liner, placing reinforcing steel for the Control Building and Auxiliary Building, and preplacement activities for the screen well base mat.

No items of noncompliance were identified.

3. Equipment Maintenance

The inspector examined a sampling of equipment preventive maintenance records for July and August, 1978, during shutdown of construction. Preventive maintenance had been performed except that rotation of certain pumps could not be performed as scheduled due to unavailability of the necessary craft personnel. Unsatisfactory inspection reports have been written for this equipment and Nonconformance and Disposition (N & D) Report No. 0670 has been issued.

The inspector had no further questions concerning this matter.



4. Risk Release of Equipment

The inspector examined the documents listed below for control of release of nonconforming equipment and material(risk release) to construction.

- -- Quality Assurance Program, Rev. 6.
- -- Quality Standard QS 15.2, Rev. A, Change Notice No. 4, "Risk Release."
- -- QC Instruction FN2-S15.3-010, dated September 2, 1977, "Risk Release of Nonconforming/ Unsat. Equipment and Material."

The inspector reviewed approximately 50% of the "risk releases" which had been issued. The releases had been documented and approved in accordance with the procedures identified above.

No items of noncompliance were identified.

5. Vendor Documentations

The inspector examined the documentation package for the below listed components to verify that Nuclear Steam System Supplier (NSSS) purchase material documentation is in compliance with NSSS purchase specification requirements and Paragraphs D.3.8 and D.4.4.5 of Appendix D of the PSAR.

- (1) 2RCS-MOV-10A & B-Recirculation Valves
- (2) 2RHS-P-1A-Residual Heat Removal Pump

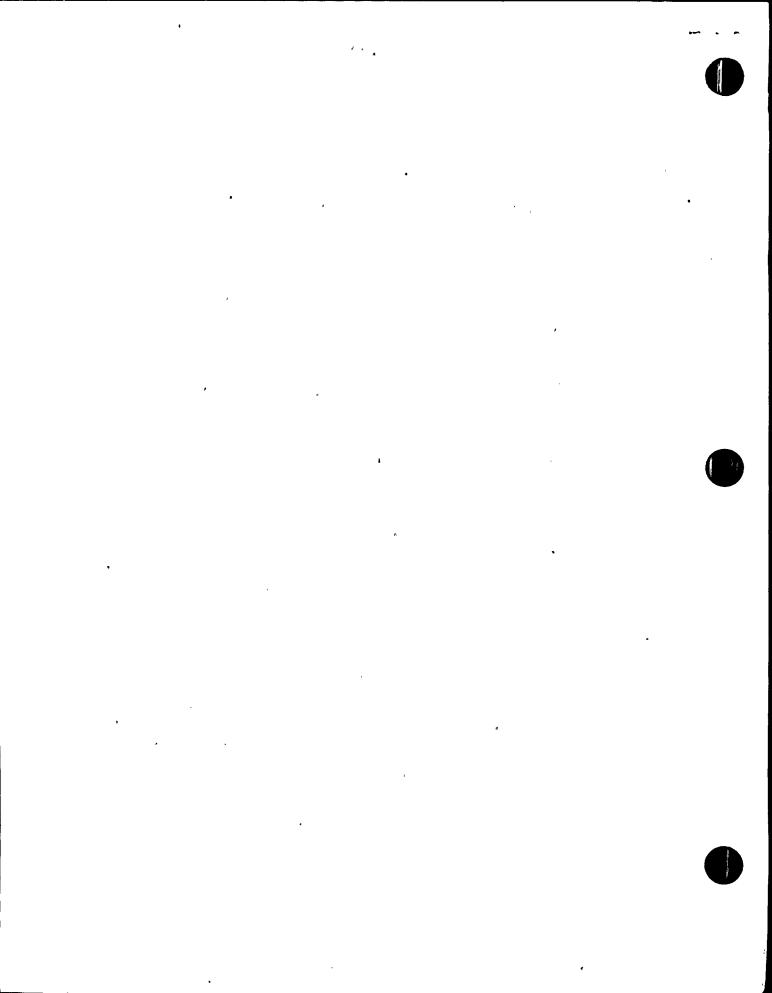
Site personnel whose purpose it is to examine documentation for completeness were interviewed. The site quality assurance procedures were reviewed to ensure that a program was available to receive review, organize, store, and retrieve documentation.

No items of noncompliance were identified.

6. Spacing of Reinforcing Steel

During the plant tour (Paragraph 2, above) the inspector noted that spacing of reinforcing steel for the screen well base mat, Pour No. 5-310-025P, did not, at some locations, meet the requirements of ACI 318 which is referenced in the PSAR as one of the structural codes. The required preplacement inspection of this part had





not been performed by QC. This item is unresolved pending rework of the rebar installation to provide the proper spacing and its inspection by field QC. (78-07-01)

7. Licensee Action on Previous Inspection Findings

(Closed) Unresolved Item (78-06-01): PSAR Change Notice (CN) No. 409 for revision of the requirements for tests of reinforcing steel for unacceptable inclusions has been approved.

8. Review of Nonroutine Events Reported by the Licensee

By letter dated October 5, 1977, the licensee had reported the existence of the potential for rock stress at the site and that this might affect the wall of an electrical tunnel. A study and analysis of this problem has been completed and a summary of the results has been submitted to the Office of Nuclear Reactor Regulation. The licensee will also address this problem in the Final Safety Analysis Report.

This item is resolved.

9. Licensee Action on IE Bulletins and Circulars

The inspector examined records and interviewed licensee representatives to verify that corrective action had been taken with respect to IE Bulletins and Circulars. For each of the Bulletins shown below the inspector verified that the identified equipment had been added to the Excluded Equipment list of the appropriate specification. In addition, licensee's Audit No. 15 of GE has confirmed that GE is taking the necessary actions for control of Bulletin and Circular information, and licensees Audit No. 27 had resulted in S&W Generic Change Notice (GCN) No. 0024 for incorporation of current Excluded Equipment Lists in equipment specifications for safety related equipment.

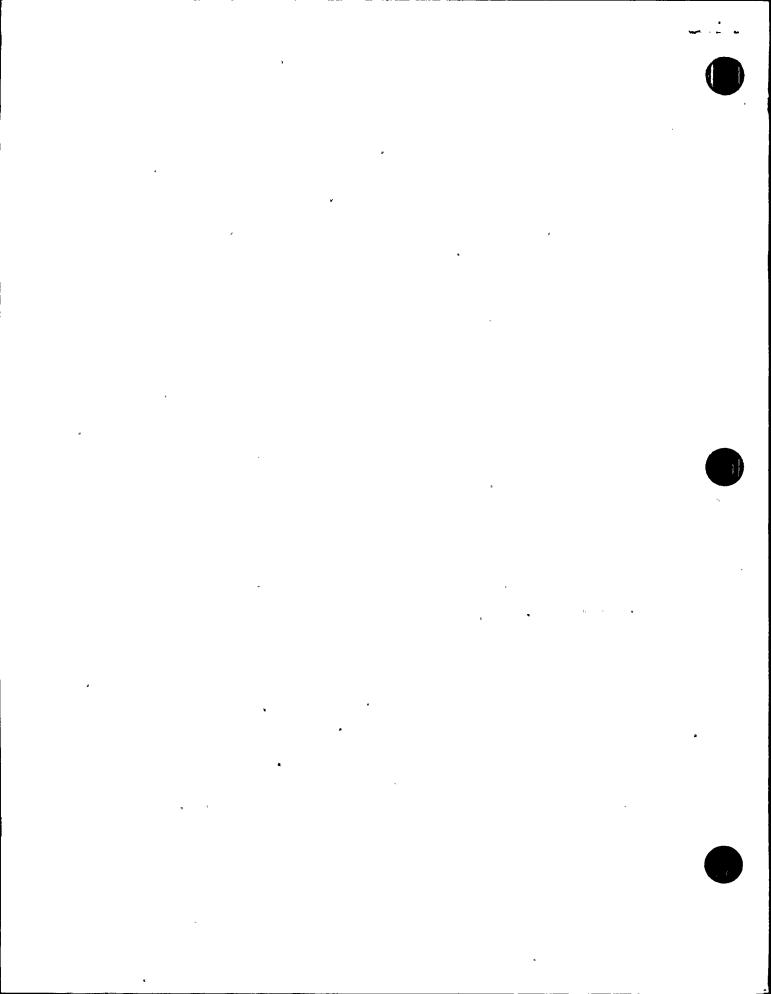
Bulletin 74-12 Closed. Westinghouse Type SG Relays

Bulletin 78-05 Closed. GE Auxiliary Contact Mechanisms (CR 105X-CB)

Bulletin 78-06 Closed. Cutler Hammer Type M Relays

The inspector also verified that S&W Engineering had reviewed the circulars listed below and established design requirements in accordance with the information and recommendations contained there in.







Circular 77-03 Closed. Fire in ITE Motor Control Center

Circular 77-15 Closed. Degradation of Fuel Oil Flow to Emergency Diesel Generator.

Circular 78-04 Closed. Installation Error That Could Prevent Closing of Fire Doors.

10. <u>Unresolved Items</u>

Unresolved items are matters about which more information is required in order to ascertain whether they are acceptable items, or items of noncompliance. An unresolved item disclosed during the inspection is discussed in Paragraph 6.

11. Exit Interview

At the conclusion of the inspection on September 28, 1978, a meeting was held with representatives of the licensee. The inspectors summarized the results of the inspection as described in this report.

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