

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

March 24, 2017

Mr. Daniel G. Stoddard Senior Vice President and Chief Nuclear Officer Innsbrook Technical Center 5000 Dominion Blvd. Glenn Allen, VA 23060

SUBJECT: NORTH ANNA POWER STATION, UNIT 2 - REVIEW OF THE STEAM **GENERATOR TUBE INSERVICE INSPECTIONS DURING THE SPRING 2016** REFUELING OUTAGE (CAC NO. MF8165)

Dear Mr. Stoddard:

By letters dated July 12 and October 19, 2016 (Agencywide Documents Access and Management System Accession Nos. ML16202A066 and ML16298A388, respectively), Virginia Electric & Power Co. (the licensee) submitted information summarizing the results of the spring 2016 steam generator tube inspections at North Anna Power Station, Unit 2. These inspections were performed during refueling outage 24.

The U.S. Nuclear Commission (NRC) staff has completed its review of the submittal and concludes that the licensee provided the information required by the North Anna Power Station, Unit 2, technical specifications. No additional follow-up is required at this time. The results of the NRC staff's review and observations are enclosed.

If you have any questions, please contact me at (301) 415-2020, or via email at Brenda.Mozafari@nrc.gov.

Sincerely.

Brenda Mozafari, Senior Project Mahager Plant Licensing Branch II-1 **Division of Operating Reactor Licensing** Office of Nuclear Reactor Regulation

Docket Nos. 50-339

cc: Distribution via Listserv

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REVIEW OF THE SPRING 2016 STEAM GENERATOR TUBE INSPECTION REPORT NORTH ANNA POWER STATION, UNIT 2 DOCKET NO. 50-339 CAC NO. MF8185

By letters dated July 12 and October 19, 2016 (Agencywide Documents Access and Management System Accession Nos. ML16202A066 and ML16298A388, respectively), Virginia Electric & Power Co. (the licensee) submitted information summarizing the results of the spring 2016 steam generator tube inspections at North Anna Power Station, Unit 2. These inspections were performed during refueling outage 24.

North Anna, Unit 2 has three Westinghouse Model 54F replacement steam generators (SGs) that were installed in May 1995. Each SG contains 3,592 thermally treated Alloy 690 tubes, with a nominal outside diameter of 0.875 inches and a nominal wall thickness of 0.050 inches. The tubes are supported by seven Type 405 stainless steel tube support plates. The tube support plate holes are quatrefoil-shaped. The U-bend region of the first eight rows of tubing was stress relieved after bending.

The licensee provided the scope, extent, methods, and results of their SG tube inspections in the documents referenced above. In addition, the licensee described corrective actions, such as tube plugging, taken in response to the inspection findings.

Based on a review of the information provided, the NRC staff concludes that the licensee provided the information required by the Technical Specifications. In addition, the NRC staff concludes that there are no technical issues that warrant follow-up action at this time since the inspections appear to be consistent with the objective of detecting potential tube degradation, and the inspection results appear to be consistent with industry operating experience at similarly designed and operated units.

Principal Contributor: A. Johnson