

ATTACHMENT TO LICENSE AMENDMENT NO. 65

TO FACILITY COMBINED LICENSE NO. NPF-93

DOCKET NO. 52-027

Replace the following page of the Facility Combined License No. NPF-93 with the attached revised page(s). The revised pages are identified by an amendment number and contain marginal lines indicating the areas of change.

Facility Combined License No. NPF-93

REMOVE

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Facility Combined License No. NPF-93  
Appendix A – Technical Specifications

REMOVE

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3.3.1-7

3.3.1-7

- (b) SCE&G shall report any violation of a requirement in Section 2.D.(3), Section 2.D.(4), Section 2.D.(5), and Section 2.D.(6) of this license within 24 hours. Initial notification shall be made to the NRC Operations Center in accordance with 10 CFR 50.72, with written follow up in accordance with 10 CFR 50.73.

(8) Incorporation

The Technical Specifications, Environmental Protection Plan, and ITAAC in Appendices A, B, and C, respectively of this license, as revised through Amendment No. 65, are hereby incorporated into this license.

(9) Technical Specifications

The technical specifications in Appendix A to this license become effective upon a Commission finding that the acceptance criteria in this license (ITAAC) are met in accordance with 10 CFR 52.103(g).

(10) Operational Program Implementation

SCE&G shall implement the programs or portions of programs identified below, on or before the date SCE&G achieves the following milestones.

- (a) Environmental Qualification Program implemented before initial fuel load;
- (b) Reactor Vessel Material Surveillance Program implemented before initial criticality;
- (c) Preservice Testing Program implemented before initial fuel load;
- (d) Containment Leakage Rate Testing Program implemented before initial fuel load;
- (e) Fire Protection Program
  - 1. The fire protection measures in accordance with Regulatory Guide (RG) 1.189 for designated storage building areas (including adjacent fire areas that could affect the storage area) implemented before initial receipt of byproduct or special nuclear materials that are not fuel (excluding exempt quantities as described in 10 CFR 30.18);
  - 2. The fire protection measures in accordance with RG 1.189 for areas containing new fuel (including adjacent areas where a fire could affect the new fuel) implemented before receipt of fuel onsite;

Table 3.3.1-1 (page 2 of 2)  
Reactor Trip System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS
7. Reactor Coolant Flow – Low	1 <sup>(b)</sup>	4 per hot leg	E	SR 3.3.1.1 SR 3.3.1.3 SR 3.3.1.6 SR 3.3.1.8 SR 3.3.1.11
8. Reactor Coolant Pump (RCP) Bearing Water Temperature – High	1,2	4 per RCP	D	SR 3.3.1.1 SR 3.3.1.6 SR 3.3.1.8 SR 3.3.1.11
9. RCP Speed – Low	1 <sup>(b)</sup>	4 (1/pump)	E	SR 3.3.1.1 SR 3.3.1.6 SR 3.3.1.8 SR 3.3.1.11
10. Steam Generator (SG) Narrow Range Water Level – Low	1,2	4 per SG	D	SR 3.3.1.1 SR 3.3.1.6 SR 3.3.1.8 SR 3.3.1.11
11. Steam Generator (SG) Narrow Range Water Level – High 2	1,2 <sup>(c)</sup>	4 per SG	D	SR 3.3.1.1 SR 3.3.1.6 SR 3.3.1.8 SR 3.3.1.11
12. Passive Residual Heat Removal Actuation	1,2	4 per valve	D	SR 3.3.1.9 SR 3.3.1.10 SR 3.3.1.11

(b) Above the P-10 (Power Range Neutron Flux) interlock.

(c) Above the P-11 (Pressurizer Pressure) interlock.