



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 22, 2017

Mr. Robert Coffey
Site Vice President
NextEra Energy Point Beach, LLC
6610 Nuclear Road
Two Rivers, WI 54241

SUBJECT: POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF
AMENDMENTS - REMOVAL OF COMPLETED LICENSE CONDITIONS AND
CHANGES TO THE VENTILATION FILTER TESTING PROGRAM (CAC NOS.
MF7352 AND MF7353)

Dear Mr. Coffey:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment Nos. 258 and 262 to Renewed Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Plant (Point Beach), Units 1 and 2, respectively. The amendments consist of changes to the Renewed Facility Operating Licenses and Technical Specifications (TSs) in response to your application dated February 12, 2016, supplemented by letters dated July 11, 2016, and November 4, 2016. These amendments remove license conditions that have been completed and are no longer in effect and revise a charcoal testing criterion for the control room emergency filtration system.

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink, appearing to read "Chawla", written over a light blue horizontal line.

Mahesh L. Chawla, Project Manager
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-266 and 50-301

Enclosures:

1. Amendment No. 258 to DPR-24
2. Amendment No. 262 to DPR-27
3. Safety Evaluation

cc w/encls: Distribution via ListServ



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NEXTERA ENERGY POINT BEACH, LLC

DOCKET NO. 50-266

POINT BEACH NUCLEAR PLANT, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 258
License No. DPR-24

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by NextEra Energy Point Beach, LLC (the licensee), dated February 12, 2016, as supplemented by letters dated July 11, 2016, and November 4, 2016, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 4.B, 4.E, 4.I, and 4.J of the Renewed Facility Operating License No. DPR-24 is hereby amended to read as follows:
 - B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 258, are hereby incorporated in the renewed operating license. NextEra Energy Point Beach shall operate the facility in accordance with the Technical Specifications.

E. Safety Injection Logic

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I. Deleted

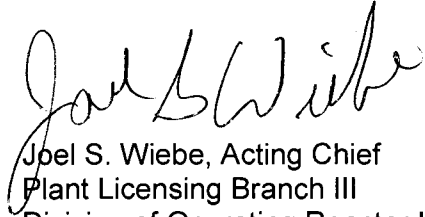
J. Deleted

3. The following additional conditions of the Appendix C, "Additional Conditions," of the Renewed Facility Operating License No. DPR-24 are deleted:

<u>Amendment Number</u>	<u>Additional Condition</u>	<u>Implementation Date</u>
201	Deleted	
201	Deleted	
238	Deleted	
238	Deleted	
240	Deleted	
240	Deleted	
240	Deleted	
240	Deleted	
240	Deleted	
240	Deleted	
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241	Deleted	
241	Deleted	

4. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Joel S. Wiebe, Acting Chief
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the
Technical Specifications and
Renewed Facility Operating License

Date of issuance: February 22, 2017



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NEXTERA ENERGY POINT BEACH, LLC

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 262
License No. DPR-27

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by NextEra Energy Point Beach, LLC (the licensee), dated February 12, 2016, as supplemented by letters dated July 11, 2016, and November 4, 2016, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 4.B, 4.E, 4.H, and 4.I of the Renewed Facility Operating License No. DPR-27 is hereby amended to read as follows:
 - B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 262, are hereby incorporated in the renewed operating license. NextEra Point Beach shall operate the facility in accordance with Technical Specifications.

E. Safety Injection Logic

Deleted

H. Deleted

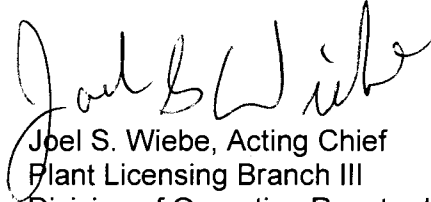
I. Deleted

3. The following additional conditions of the Appendix C, "Additional Conditions," of the Renewed Facility Operating License No. DPR-27 are deleted:

<u>Amendment Number</u>	<u>Additional Condition</u>	<u>Implementation Date</u>
178	Deleted	
206	Deleted	
206	Deleted	
242	Deleted	
242	Deleted	
244	Deleted	
244	Deleted	
244	Deleted	
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4. This license amendment is effective as of the date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Joel S. Wiebe, Acting Chief
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation
Attachment: Changes to the

Technical Specifications and
Renewed Facility Operating License

Date of issuance: February 22, 2017

ATTACHMENT TO LICENSE AMENDMENT NO. 258
TO RENEWED FACILITY OPERATING LICENSE NO. DPR-24
AND LICENSE AMENDMENT NO. 262
TO RENEWED FACILITY OPERATING LICENSE NO. DPR-27
DOCKET NOS. 50-266 AND 50-301

Replace the following pages of Renewed Facility Operating License Nos. DPR-24 and DPR-27, and Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Renewed Facility Operating License

<u>REMOVE</u>	<u>INSERT</u>
3	3
4	4
5	5
6	6
7	7
8	8
9	None
C-1	C-1
C-2	C-2
C-3	None
C-4	None

Technical Specifications

<u>REMOVE</u>	<u>INSERT</u>
5.5-11	5.5-11

- D. Pursuant to the Act and 10 CFR Parts 30, 40 and 70, NextEra Energy Point Beach to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
 - E. Pursuant to the Act and 10 CFR Parts 30 and 70, NextEra Energy Point Beach to possess such byproduct and special nuclear materials as may be produced by the operation of the facility, but not to separate such materials retained within the fuel cladding.
4. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:
- A. Maximum Power Levels

NextEra Energy Point Beach is authorized to operate the facility at reactor core power levels not in excess of 1800 megawatts thermal.
 - B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 258, are hereby incorporated in the renewed operating license. NextEra Energy Point Beach shall operate the facility in accordance with Technical Specifications.
 - C. Spent Fuel Pool Modification

The licensee is authorized to modify the spent fuel storage pool to increase its storage capacity from 351 to 1502 assemblies as described in licensee's application dated March 21, 1978, as supplemented and amended. In the event that the on-site verification check for poison material in the poison assemblies discloses any missing boron plates, the NRC shall be notified and an on-site test on every poison assembly shall be performed.

D. Physical Protection

NextEra Energy Point Beach shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans, including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contain Safeguards Information protected under 10 CFR 73.21, is entitled: "Point Beach Nuclear Plant Physical Security Plan, (Revision 4)," submitted by letter dated May 10, 2006. NextEra Energy Point Beach, LLC shall fully implement and maintain in effect all provisions of the Commission-approved Point Beach Nuclear Plant Cyber Security Plan (CSP), including changes made pursuant to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The NextEra Energy Point Beach CSP was approved by License Amendment No. 243 as supplemented by a change approved by License Amendment No. 247 and License Amendment No. 252.

E. Deleted

F. NextEra Energy Point Beach Unit 1 shall implement and maintain in effect all provisions of the approved fire protection program that comply with 10 CFR 50.48(a) and 10 CFR 50.48(c), as specified in the license amendment request dated June 26, 2013, and supplements dated September 16, 2013, July 29, 2014, August 28, 2014, September 25, 2014, November 14, 2014, December 19, 2014, January 16, 2015, May 12, 2015, August 26, 2015, February 22, 2016, April 07, 2016, and May 3, 2016, and as approved in the safety evaluation report dated September 8, 2016. Except where NRC approval for changes or deviations is required by 10 CFR 50.48(c), and provided no other regulation, technical specification, license condition or requirement would require prior NRC approval, the licensee may make changes to the fire protection program without prior approval of the Commission if those changes satisfy the provisions set forth in 10 CFR 50.48(a) and 10 CFR 50.48(c), the change does not require a change to a technical specification or license condition, and the criteria listed below are satisfied.

1. Risk-Informed Changes that May Be Made Without Prior NRC Approval

A risk assessment of the change must demonstrate that the acceptance criteria below are met. The risk assessment approach, methods, and data shall be acceptable to the NRC and shall be appropriate for the nature and scope of the change being evaluated; be based on the as-built, as-operated, and maintained plant; and reflect the operating experience at the plant. Acceptable methods to assess the risk of the change may include methods that have been used in the

peer-reviewed fire PRA model, methods that have been approved by NRC through a plant-specific license amendment or NRC approval of generic methods specifically for use in NFPA 805 risk assessments, or methods that have been demonstrated to bound the risk impact.

- a. Prior NRC review and approval is not required for changes that clearly result in a decrease in risk. The proposed change must also be consistent with the defense-in-depth philosophy and must maintain sufficient safety margins. The change may be implemented following completion of the plant change evaluation.
 - b. Prior NRC review and approval is not required for individual changes that result in a risk increase less than 1×10^{-7} /year (yr) for CDF and less than 1×10^{-8} /yr for LERF. The proposed change must also be consistent with the defense-in-depth philosophy and must maintain sufficient safety margins. The change may be implemented following completion of the plant change evaluation.
2. Other Changes that May Be Made Without Prior NRC Approval
- a. Changes to NFPA 805, Chapter 3, Fundamental Fire Protection Program.

Prior NRC review and approval are not required for changes to the NFPA 805, Chapter 3, fundamental fire protection program elements and design requirements for which an engineering evaluation demonstrates that the alternative to the Chapter 3 element is functionally equivalent or adequate for the hazard. The licensee may use an engineering evaluation to demonstrate that a change to an NFPA 805, Chapter 3 element is functionally equivalent to the corresponding technical requirement. A qualified fire protection engineer shall perform the engineering evaluation and conclude that the change has not affected the functionality of the component, system, procedure, or physical arrangement, using a relevant technical requirement or standard.

The licensee may use an engineering evaluation to demonstrate that changes to certain NFPA 805, Chapter 3 elements are acceptable because the alternative is "adequate for the hazard." Prior NRC review and approval would not be required for alternatives to four specific sections of NFPA 805, Chapter 3, for which an engineering evaluation demonstrates that the alternative to the Chapter 3 element is adequate for the hazard. A qualified fire protection engineer shall perform the engineering evaluation and conclude that the change has not affected the functionality of the component, system, procedure, or physical arrangement, using a relevant technical requirement or standard. The four specific sections of NFPA 805, Chapter 3, are as follows:

- "Fire Alarm and Detection Systems" (Section 3.8);
- "Automatic and Manual Water-Based Fire Suppression Systems" (Section 3.9);
- "Gaseous Fire Suppression Systems" (Section 3.10); and,
- "Passive Fire Protection Features" (Section 3.11).

(This License Condition does not apply to any demonstration of equivalency under Section 1.7 of NFPA 805.)

b. Fire Protection Program Changes that Have No More than Minimal Risk Impact

Prior NRC review and approval are not required for changes to the licensee's fire protection program that have been demonstrated to have no more than a minimal risk impact. The licensee may use its screening process as approved in the NRC safety evaluation report dated September 8, 2016 to determine that certain fire protection program changes meet the minimal criterion. The licensee shall ensure that fire protection defense-in-depth and safety margins are maintained when changes are made to the fire protection program.

3. Transition License Conditions

- a. Before achieving full compliance with 10 CFR 50.48(c), as specified by 3.b and 3.c below, risk-informed changes to the licensee's fire protection program may not be made without prior NRC review and approval unless the change has been demonstrated to have no more than a minimal risk impact, as described in 2.b above.
- b. The licensee shall implement the modifications to its facility as described in Attachment S, Table S-2 "Plant Modifications Committed," of NextEra Energy Point Beach letter NRC-2016-0013 to complete the transition to full compliance with 10 CFR 50.48(c) no later than prior to startup from the second refueling outage (for each unit) after receipt of the license amendment. The licensee shall maintain appropriate compensatory measures in place until completion of these modifications.
- c. The licensee shall implement the items in Attachment S, Table S-3, "Implementation Items," of NextEra Energy Point Beach letter NRC-2016-0021, with the exception of items noted below, within 12 months after NRC approval unless that falls within a scheduled outage window; then in that case, completion will occur 60 days after the startup from that scheduled outage.
 - i. Implementation item 120 is an exception as the industry guidance is under review by the NRC and the final resolution will occur 12 months after the guidance is available unless that falls within a scheduled outage window; then in that case, completion will occur 60 days after startup from that scheduled outage.
 - ii. Implementation items 142 and 150 are exceptions because they are associated with completion of committed modifications identified in LAR Attachment S, Table S-2 and will not be completed until 3 months following the last refueling outage identified in item 3.b above.

G. Secondary Water Chemistry Monitoring Program

NextEra Energy Point Beach shall implement a secondary water chemistry monitoring program to inhibit steam generator tube degradation. This program shall include:

1. Identification of a sampling schedule for the critical parameters and control points for these parameters;
2. Identification of the procedures used to quantify parameters that are critical to control points;
3. Identification of process sampling points;
4. Procedure for the recording and management of data;
5. Procedures defining corrective actions for off control point chemistry condition; and
6. A procedure for identifying the authority responsible for the interpretation of the data, and the sequence and timing of administrative events required to initiate corrective action.

H. The licensee is authorized to repair Unit 1 steam generators by replacement of major components. Repairs shall be conducted in accordance with the licensee's commitments identified in the Commission approved Point Beach Nuclear Plant Unit No. 1 Steam Generator Repair Report dated August 9, 1982 and revised March 1, 1983 and additional commitments identified in the staff's related safety evaluation.

I. Deleted

J. Deleted

K. All capsules in the reactor vessel that are removed and tested must meet the test procedures and reporting requirements of American Society for Testing and Materials (ASTM) E 185-82 to the extent practicable for the configuration of the specimens in the capsule. Any changes to the capsule withdrawal schedule, including spare capsules, must be approved by the NRC prior to implementation. All capsules placed in storage must be maintained for future insertion. Any changes to storage requirements must be approved by the NRC, as required by 10 CFR Part 50, Appendix H.

L. Mitigation Strategy

Strategies shall be developed and maintained for addressing large fires and explosions that include the following key areas:

1. Fire fighting response strategy with the following elements:
 - a. Pre-defined coordinated fire response strategy and guidance
 - b. Assessment of mutual aid fire fighting assets
 - c. Designated staging areas for equipment and materials
 - d. Command and control
 - e. Training of response personnel

2. Operations to mitigate fuel damage considering the following:
 - a. Protection and use of personnel assets
 - b. Communications
 - c. Minimizing fire spread
 - d. Procedures for implementing integrated fire response strategy
 - e. Identification of readily-available pre-staged equipment
 - f. Training on integrated fire response strategy
 - g. Spent fuel pool mitigation measures

3. Actions to minimize release to include consideration of:
 - a. Water spray scrubbing
 - b. Dose to onsite responders

M. Additional Conditions

The additional conditions contained in Appendix C, as revised through Amendment No. 241, are hereby incorporated into this license. NextEra Energy Point Beach shall operate the facility in accordance with the additional conditions.

5. The issuance of this renewed operating license is without prejudice to subsequent licensing action which may be taken by the Commission with regard to the ongoing rulemaking hearing on the Interim Acceptance Criteria for Emergency Core Cooling Systems (Docket No. RM 50-1).

6. This renewed operating license is effective as of the date of issuance, and shall expire at midnight on October 5, 2030.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By

R. W. Borchardt, Deputy Director
Office of Nuclear Reactor Regulation

Attachments:

1. Appendix A - Technical Specifications
2. Appendix B - Environmental Technical Specifications
3. Appendix C - Additional Conditions

Date of Issuance: December 22, 2005

APPENDIX C
ADDITIONAL CONDITIONS
OPERATING LICENSE DPR-24

NextEra Energy Point Beach, LLC shall comply with the following conditions and the schedules noted below:

<u>Amendment Number</u>	<u>Additional Conditions</u>	<u>Implementation Date</u>
474	Deleted	
174	This amendment is authorized contingent on compliance with commitments provided by the licensee to operate Point Beach Nuclear Plant in accordance with its service water system analyses and approved procedures. Specifically, each unit will utilize only one component cooling water heat exchanger until such time as analyses are completed and the service water system reconfigured as necessary to allow operation of one or both units with two heat exchangers in service. If two component cooling water heat exchangers are required in one or both units for maintaining acceptable component cooling water temperature prior to completion of necessary analyses to allow operation in the required configuration, the service water system will be considered in an unanalyzed condition, declared inoperable, and action taken as specified by TS LCO 3.0.3 except for short periods of time as necessary to effect procedurally controlled changes in system lineups and unit operating conditions.	Immediately
201	Deleted	
201	Deleted	
228	At the time of the closing of the transfer of the licenses from Wisconsin Electric Power Company (WEPCO) to FPLE Point Beach ¹ , WEPCO shall transfer to FPLE Point Beach WEPCO's decommissioning funds in an aggregate minimum value of \$200.8 million for Point Beach Unit 1. FPLE Point Beach shall deposit such funds in an external decommissioning trust fund established by FPLE Point Beach for Point Beach Units 1 and 2. The trust agreement shall be in a form acceptable to the NRC.	Immediately
	NextEra Energy Point Beach shall take no actions to cause FPL Group Capital, or its successors and assigns, to void, cancel, or modify its \$70 million Support Agreement (Agreement) to NextEra Energy Point Beach, as presented in its application dated January 26, 2007, or cause it to fail to perform or impair its performance under the Agreement, without the prior written consent from the NRC. The Agreement may not be amended or modified without 30 days prior written notice to the Director of Nuclear Reactor Regulation or his designee. An executed copy of the Agreement shall be submitted to the NRC no later than 30 days after the completion of the license transfers. Also, NextEra Energy Point Beach shall inform the NRC in writing anytime it draws upon the \$70 million Agreement.	Immediately
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238	Deleted	
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¹ On April 16, 2009, the name "FPLE Point Beach, LLC" was changed to "NextEra Energy Point Beach, LLC."

APPENDIX C
ADDITIONAL CONDITIONS
OPERATING LICENSE DPR-24

NextEra Energy Point Beach, LLC shall comply with the following conditions and the schedules noted below:

<u>Amendment Number</u>	<u>Additional Conditions</u>	<u>Implementation Date</u>
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- C. Pursuant to the Act and 10 CFR Parts 30, 40 and 70, NextEra Energy Point Beach to receive, possess and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed source for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - D. Pursuant to the Act and 10 CFR Parts 30, 40 and 70, NextEra Energy Point Beach to receive, possess and use in amounts as required any byproduct, source of special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
 - E. Pursuant to the Act and 10 CFR Parts 30 and 70, NextEra Energy Point Beach to possess such byproduct and special nuclear materials as may be produced by the operation of the facility, but not to separate such materials retained within the fuel cladding.
4. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:
- A. Maximum Power Levels

NextEra Energy Point Beach is authorized to operate the facility at reactor core power levels not in excess of 1800 megawatts thermal.
 - B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 262, are hereby incorporated in the renewed operating license. NextEra Energy Point Beach shall operate the facility in accordance with Technical Specifications.
 - C. Spent Fuel Pool Modification

The licensee is authorized to modify the spent fuel storage pool to increase its storage capacity from 351 to 1502 assemblies as described in licensee's application dated March 21, 1978, as supplemented and amended. In the event that the on-site verification check for poison material in the poison assemblies discloses any missing boron plates, the NRC shall be notified and an on-site test on every poison assembly shall be performed.

D. Physical Protection

NextEra Energy Point Beach shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans, including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contain Safeguards Information protected under 10 CFR 73.21, is entitled: "Point Beach Nuclear Plant Physical Security Plan, (Revision 4)," submitted by letter dated May 10, 2006. NextEra Energy Point Beach, LLC shall fully implement and maintain in effect all provisions of the Commission-approved Point Beach Nuclear Plant Cyber Security Plan (CSP), including changes made pursuant to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The NextEra Energy Point Beach CSP was approved by License Amendment No. 247 as supplemented by a change approved by License Amendment No. 251 and License Amendment No. 256.

E. Deleted

F. NextEra Energy Point Beach Unit 2 shall implement and maintain in effect all provisions of the approved fire protection program that comply with 10 CFR 50.48(a) and 10 CFR 50.48(c), as specified in the license amendment request dated June 26, 2013, and supplements dated September 16, 2013, July 29, 2014, August 28, 2014, September 25, 2014, November 14, 2014, December 19, 2014, January 16, 2015, May 12, 2015, August 26, 2015, February 22, 2016, April 07, 2016, and May 3, 2016 and as approved in the safety evaluation report dated September 8, 2016. Except where NRC approval for changes or deviations is required by 10 CFR 50.48(c), and provided no other regulation, technical specification, license condition or requirement would require prior NRC approval, the licensee may make changes to the fire protection program without prior approval of the Commission if those changes satisfy the provisions set forth in 10 CFR 50.48(a) and 10 CFR 50.48(c), the change does not require a change to a technical specification or license condition, and the criteria listed below are satisfied.

1. Risk-Informed Changes that May Be Made Without Prior NRC Approval

A risk assessment of the change must demonstrate that the acceptance criteria below are met. The risk assessment approach, methods, and data shall be acceptable to the NRC and shall be appropriate for the nature and scope of the change being evaluated; be based on the as-built, as-operated, and maintained plant; and reflect the operating experience at the plant. Acceptable methods to assess the risk of the change may include methods that have been used in the peer-reviewed fire PRA model, methods that have been approved by NRC through a plant-specific license amendment or NRC approval of generic methods specifically for use in NFPA 805 risk assessments, or methods that have been demonstrated to bound the risk impact

- a. Prior NRC review and approval is not required for changes that clearly result in a decrease in risk. The proposed change must also be consistent with the defense-in-depth philosophy and must maintain sufficient safety margins. The change may be implemented following completion of the plant change evaluation.
 - b. Prior NRC review and approval is not required for individual changes that result in a risk increase less than 1×10^{-7} /year (yr) for CDF and less than 1×10^{-8} /yr for LERF. The proposed change must also be consistent with the defense-in-depth philosophy and must maintain sufficient safety margins. The change may be implemented following completion of the plant change evaluation.
2. Other Changes that May Be Made Without Prior NRC Approval
 - a. Changes to NFPA 805, Chapter 3, Fundamental Fire Protection Program.

Prior NRC review and approval are not required for changes to the NFPA 805, Chapter 3, fundamental fire protection program elements and design requirements for which an engineering evaluation demonstrates that the alternative to the Chapter 3 element is functionally equivalent or adequate for the hazard. The licensee may use an engineering evaluation to demonstrate that a change to an NFPA 805, Chapter 3 element is functionally equivalent to the corresponding technical requirement. A qualified fire protection engineer shall perform the engineering evaluation and conclude that the change has not affected the functionality of the component, system, procedure, or physical arrangement, using a relevant technical requirement or standard.

The licensee may use an engineering evaluation to demonstrate that changes to certain NFPA 805, Chapter 3 elements are acceptable because the alternative is "adequate for the hazard." Prior NRC review and approval would not be required for alternatives to four specific sections of NFPA 805, Chapter 3, for which an engineering evaluation demonstrates that the alternative to the Chapter 3 element is adequate for the hazard. A qualified fire protection engineer shall perform the engineering evaluation and conclude that the change has not affected the functionality of the component, system, procedure, or physical arrangement, using a relevant technical requirement or standard. The four specific sections of NFPA 805, Chapter 3, are as follows:

- "Fire Alarm and Detection Systems" (Section 3.8);
- "Automatic and Manual Water-Based Fire Suppression Systems" (Section 3.9);
- "Gaseous Fire Suppression Systems" (Section 3.10); and,
- "Passive Fire Protection Features" (Section 3.11).

(This License Condition does not apply to any demonstration of equivalency under Section 1.7 of NFPA 805.)

b. Fire Protection Program Changes that Have No More than Minimal Risk Impact

Prior NRC review and approval are not required for changes to the licensee's fire protection program that have been demonstrated to have no more than a minimal risk impact. The licensee may use its screening process as approved in the NRC safety evaluation report dated September 8, 2016 to determine that certain fire protection program changes meet the minimal criterion. The licensee shall ensure that fire protection defense-in-depth and safety margins are maintained when changes are made to the fire protection program.

3. Transition License Conditions

- a. Before achieving full compliance with 10 CFR 50.48(c), as specified by 3.b and 3.c below, risk-informed changes to the licensee's fire protection program may not be made without prior NRC review and approval unless the change has been demonstrated to have no more than a minimal risk impact, as described in 2.b above.
- b. The licensee shall implement the modifications to its facility as described in Attachment S, Table S-2 "Plant Modifications Committed," of NextEra Energy Point Beach letter NRC-2016-0013 to complete the transition to full compliance with 10 CFR 50.48(c) no later than prior to startup from the second refueling outage (for each unit) after receipt of the license amendment. The licensee shall maintain appropriate compensatory measures in place until completion of these modifications.
- c. The licensee shall implement the items in Attachment S, Table S-3, "Implementation Items," of NextEra Energy Point Beach letter NRC-2016-0021, with the exception of items noted below, within 12 months after NRC approval unless that falls within a scheduled outage window; then in that case, completion will occur 60 days after the startup from that scheduled outage.
 - i. Implementation item 120 is an exception as the industry guidance is under review by the NRC and the final resolution will occur 12 months after the guidance is available unless that falls within a scheduled outage window; then in that case, completion will occur 60 days after startup from that scheduled outage.
 - ii. Implementation items 142 and 150 are exceptions because they are associated with completion of committed modifications identified in LAR Attachment S, Table S-2 and will not be completed until 3 months following the last refueling outage identified in item 3.b above.

G. Secondary Water Chemistry Monitoring Program

NextEra Energy Point Beach shall implement a secondary water chemistry monitoring program to inhibit steam generator tube degradation. This program shall include:

1. Identification of a sampling schedule for the critical parameters and control points for these parameters;
2. Identification of the procedures used to quantify parameters that are critical to control points;
3. Identification of process sampling points;
4. Procedure for the recording and management of data;
5. Procedures defining corrective actions for off control point chemistry condition; and
6. A procedure for identifying the authority responsible for the interpretation of the data, and the sequence and timing of administrative events required to initiate corrective action.

H. Deleted

I. Deleted

J. All capsules in the reactor vessel that are removed and tested must meet the test procedures and reporting requirements of American Society for Testing and Materials (ASTM) E 185-82 to the extent practicable for the configuration of the specimens in the capsule. Any changes to the capsule withdrawal schedule, including spare capsules, must be approved by the NRC prior to implementation. All capsules placed in storage must be maintained for future insertion. Any changes to storage requirements must be approved by the NRC, as required by 10 CFR Part 50, Appendix H.

K. Mitigation Strategy

Strategies shall be developed and maintained for addressing large fires and explosions that include the following key areas:

1. Fire fighting response strategy with the following elements:
 - a. Pre-defined coordinated fire response strategy and guidance
 - b. Assessment of mutual aid fire fighting assets
 - c. Designated staging areas for equipment and materials
 - d. Command and control
 - e. Training of response personnel

2. Operations to mitigate fuel damage considering the following:
 - a. Protection and use of personnel assets
 - b. Communications
 - c. Minimizing fire spread
 - d. Procedures for implementing integrated fire response strategy
 - e. Identification of readily-available pre-staged equipment
 - f. Training on integrated fire response strategy
 - g. Spent fuel pool mitigation measures

3. Actions to minimize release to include consideration of:
 - a. Water spray scrubbing
 - b. Dose to onsite responders

L. Additional Conditions

The additional conditions contained in Appendix C, as revised through Amendment No. 245, are hereby incorporated into this license. NextEra Energy Point Beach shall operate the facility in accordance with the additional conditions.

5. The issuance of this renewed operating license is without prejudice to subsequent licensing action which may be taken by the Commission with regard to the ongoing rulemaking hearing on the Interim Acceptance Criteria for Emergency Core Cooling Systems (Docket No. RM 50-1).

6. This renewed operating license is effective as of the date of issuance, and shall expire at midnight on March 8, 2033.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By

R. W. Borchardt, Deputy Director
Office of Nuclear Reactor Regulation

Attachments:

1. Appendix A -Technical Specifications
2. Appendix B - Environmental Technical Specifications
3. Appendix C - Additional Conditions

Date of Issuance: December 22, 2005

APPENDIX C
ADDITIONAL CONDITIONS
OPERATING LICENSE DPR-27

NextEra Energy Point Beach, LLC shall comply with the following conditions and the schedules noted below:

<u>Amendment Number</u>	<u>Additional Conditions</u>	<u>Implementation Date</u>
478	Deleted	
178	This amendment is authorized contingent on compliance with commitments provided by the licensee to operate Point Beach Nuclear Plant in accordance with its service water system analyses and approved procedures. Specifically, each unit will utilize only one component cooling water heat exchanger until such time as analyses are completed and the service water system reconfigured as necessary to allow operation of one or both units with two heat exchangers in service. If two component cooling water heat exchangers are required in one or both units for maintaining acceptable component cooling water temperature prior to completion of necessary analyses to allow operation in the required configuration, the service water system will be considered in an unanalyzed condition, declared inoperable, and action taken as specified by TS LCO 3.0.3 except for short periods of time as necessary to effect procedurally controlled changes in system lineups and unit operating conditions.	Immediately
206	Deleted	
206	Deleted	
233	At the time of the closing of the transfer of the licenses from Wisconsin Electric Power Company (WEPCO) to FPLE Point Beach ¹ , WEPCO shall transfer to FPLE Point Beach* WEPCO's decommissioning funds in an aggregate minimum value of \$189.2 million for Point Beach Unit 2. FPLE Point Beach* shall deposit such funds in an external decommissioning trust fund established by FPLE Point Beach for Point Beach Units 1 and 2. The trust agreement shall be in a form acceptable to the NRC.	Immediately
	NextEra Energy Point Beach shall take no actions to cause FPL Group Capital, or its successors and assigns, to void, cancel, or modify its \$70 million Support Agreement (Agreement) to NextEra Energy Point Beach, as presented in its application dated January 26, 2007, or cause it to fail to perform or impair its performance under the Agreement, without the prior written consent from the NRC. The Agreement may not be amended or modified without 30 days prior written notice to the Director of Nuclear Reactor Regulation or his designee. An executed copy of the Agreement shall be submitted to the NRC no later than 30 days after the completion of the license transfers. Also, NextEra Energy Point Beach shall inform the NRC in writing anytime it draws upon the \$70 million Agreement.	Immediately
242	Deleted	
242	Deleted	
244	Deleted	

¹ On April 16, 2009, the name "FPLE Point Beach, LLC" was changed to "NextEra Energy Point Beach, LLC."

APPENDIX C
ADDITIONAL CONDITIONS
OPERATING LICENSE DPR-27

NextEra Energy Point Beach, LLC shall comply with the following conditions and the schedules noted below:

<u>Amendment Number</u>	<u>Additional Conditions</u>	<u>Implementation Date</u>
244	Deleted	
244	Deleted	
244	Deleted	
244	Deleted	
244	Deleted	
244	Deleted	
244	Deleted	
245	Deleted	
245	Deleted	

5.5 Programs and Manuals

5.5.10 Ventilation Filter Testing Program (VFTP) (continued)

- c. Demonstrate for the Control Room Emergency Filtration System (F-16) that a laboratory test of a sample of the charcoal adsorber, when obtained in accordance with the methodology of ANSI N510-1980, Section 13, excluding subsection 12.3, shows the methyl iodide penetration $\leq 2.5\%$, when tested in accordance with ASTM D3803-1989 at a temperature of 30°C and a relative humidity of 95%, applying the tolerances of ASTM D3803-1989.
- d. Demonstrate for the Control Room Emergency Filtration System (F-16) that the pressure drop across the combined HEPA filters and the charcoal adsorbers is less than 6 inches of water when tested in accordance with the methodology of ANSI N510-1980, Sections 10 and 12, excluding subsections 10.3 and 12.3, at a system flowrate of 4950 cfm $\pm 10\%$.

The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the VFTP test frequencies.

5.5.11 Explosive Gas Monitoring Program

This program provides controls for potentially explosive gas mixtures contained in the on-service Gas Decay Tank.

The program shall include a limit for oxygen concentration in the on-service Gas Decay Tank and a surveillance program to ensure the limit is maintained. This limit shall be appropriate to the system's design criteria (i.e., whether or not the system is designed to withstand a hydrogen explosion).

The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the Explosive Gas Monitoring Program surveillance frequencies.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 258

TO RENEWED FACILITY OPERATING LICENSE NO. DPR-24

AND AMENDMENT NO. 262

TO RENEWED FACILITY OPERATING LICENSE NO. DPR-27

NEXTERA ENERGY POINT BEACH, LLC

POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-266 AND 50-301

1.0 INTRODUCTION

By application to the U.S. Nuclear Regulatory Commission (NRC, Commission) dated February 12, 2016 (Reference 8.4), pursuant to Section 50.90 of Title 10 of the *Code of Federal Regulations* (10 CFR 50.90), NextEra Energy Point Beach (NextEra or the licensee) submitted a license amendment request (LAR) proposing revisions to the Point Beach Nuclear Plant (PBNP), Units 1 and 2, Renewed Facility Operating License Nos. DPR-24 and DPR-27, respectively.

The supplements dated July 11, 2016 (Ref. 8.3), and November 4, 2016 (Ref. 8.1), provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the NRC staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on April 26, 2016 (81 FR 24662).

The LAR would revise technical specification (TS) 5.5.10, "Ventilation Filter Testing Program (VFTP)" by changing a charcoal testing criterion for the control room emergency filtration system and remove license conditions that have been completed and are no longer in effect.

2.0 REGULATORY EVALUATION

2.1 Background

The renewed facility operating licenses for PBNP, Units 1 and 2, contain a number of conditional requirements, many of which were incorporated into the license through the LAR process under 10 CFR 50.90. Most of the conditions that were issued in conjunction with an LAR have been left intact in the operating license although they are no longer required. As a result, NextEra is proposing an administrative change to remove those license conditions that are no longer applicable. The objective is to update the licenses so that they contain only the

license conditions that remain pertinent to current operations and do not contain unnecessary or obsolete requirements.

2.2 Applicable Regulatory Requirements

The NRC's regulatory requirements related to the content of the TSs are contained in 10 CFR, Part 50, Section 50.36, "Technical specifications." The TS requirements in 10 CFR 50.36 include the following categories: (1) safety limits, limiting safety systems settings, and limiting control settings, (2) limiting conditions for operation (LCOs), (3) surveillance requirements (SRs), (4) design features, (5) administrative controls, (6) decommissioning, (7) initial notification, and (8) written reports.

10 CFR 50.36(c)(3) states that SRs are requirements relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the LCOs will be met.

10 CFR 50.59, "Changes, tests and experiments," Section (c)(1), states, "A licensee may make changes in the facility as described in the final safety analysis report [FSAR] (as updated), make changes in the procedures as described in the final safety analysis report (as updated), and conduct tests or experiments not described in the final safety analysis report (as updated) without obtaining a license amendment pursuant to [Section] 50.90 only if: (i) A change to the technical specifications incorporated in the license is not required, and (ii) The change, test, or experiment does not meet any of the criteria in paragraph (c)(2) of this section."

10 CFR 50.67, "Accident source term," establishes acceptance criteria for design basis accident radiological analyses. 10 CFR 50.67 states in part that the applicant's analysis must demonstrate with reasonable assurance that: (1) an individual located at any point on the boundary of the exclusion area (EAB) for any 2-hour period following the onset of the postulated fission product release, would not receive a radiation dose in excess of 0.25 sievert (Sv) (25 roentgen equivalent man (rem)) total effective dose equivalent (TEDE); (2) an individual located at any point on the outer boundary of the low population zone (LPZ), who is exposed to the radioactive cloud resulting from the postulated fission product release during the entire period of its passage, would not receive a total radiation dose in excess 0.25 Sv (25 rem) TEDE; and (3) adequate radiation protection is provided to permit access to and occupancy of the control room under accident conditions without personnel receiving radiation exposures in excess of 0.05 Sv (5 rem) TEDE for the duration of the accident.

10 CFR 50.71, "Maintenance of records, making of reports," Section (e)(4) states, "Subsequent revisions [of the UFSAR] must be filed annually or 6 months after each refueling outage provided the interval between successive updates does not exceed 24 months. The revisions must reflect all changes up to a maximum of 6 months prior to the date of filing. For nuclear power reactor facilities that have submitted the certifications required by § [Section] 50.82(a)(1), subsequent revisions must be filed every 24 months."

10 CFR 50.90, "Application for amendment of license, construction permit, or early site permit," requires that when a licensee desires to amend a license or permit, an application for an amendment must be filed with the Commission fully describing the changes desired, and following as far as applicable, the form prescribed for original applications.

PBNP is not licensed to the general design criteria (GDC) of 10 CFR Part 50, Appendix A. The Atomic Energy Commission published proposed GDCs for public comment in 1967, and

the Atomic Industrial Forum (AIF) reviewed these proposed criteria and recommended changes. The PBNP GDC as documented in the updated final safety analysis report (UFSAR) are similar in content to the AIF version of the 1967 proposed GDC.

PBNP GDC 11 - The facility shall be provided with a control room from which actions to maintain safe operational status of the plant can be controlled. Adequate radiation protection shall be provided to permit continuous occupancy of the control room under any credible post-accident condition or as an alternative, access to other areas of the facility as necessary to shut down and maintain safe control of the facility without excessive radiation exposures of personnel.

Regulatory Guide (RG) 1.52, "Design, Inspection, and Testing Criteria for Air Filtration and Adsorption Units of Post-Accident Engineered-Safety-Feature Atmosphere Cleanup Systems in Light-Water-Cooled Nuclear Power Plants," June 2001, Revision 3. This guidance specifies the necessary criteria that filtration systems must meet in order to credit the removal of radioactivity in design basis dose consequence analyses. RG 1.52, Revision 3, was in effect during the time period in which the NRC staff reviewed the PBNP AST submittal.

Regulatory Guide 1.183, "Alternative Radiological Source Terms for Evaluating Design Basis Accidents at Nuclear Power Reactors." This RG provides guidance to licensees of operating power reactors on acceptable applications of ASTs; the scope, nature, and documentation of associated analyses and evaluations; consideration of impacts on analyzed risk; and content of submittals. This guide establishes an acceptable AST and identifies the significant attributes of other ASTs that may be found acceptable by the NRC staff. This RG also identifies acceptable radiological analysis assumptions for use in conjunction with the accepted AST.

3.0 TECHNICAL EVALUATION

3.1 Proposed Changes

In addition to the proposed changes to TS 5.5.10, the LAR proposes the removal of following license conditions from Unit 1 and Unit 2 Renewed Facility Operating Licenses.

No.	License Condition Heading	Unit 1	Unit 2	Technical Discussion in SE Section
1	Safety Injection Logic	Renewed Facility Operating License Section 4.E		3.9
2	FSAR Supplement	Renewed Facility Operating License Section 4.I	Renewed Facility Operating License Section 4.H	3.2
3	Completion of Activities	Renewed Facility Operating License Section 4.J	Renewed Facility Operating License Section 4.I	3.3
4	Relocate Certain	Appendix C Amendment 201/ Amendment 206		3.14

No.	License Condition Heading	Unit 1	Unit 2	Technical Discussion in SE Section
	Technical Specification Requirements	(Improved Technical Specifications Conversion) (Ref. 8.19)		
5	Schedule for Performing Surveillance Requirements			
6	Auxiliary Feedwater Modifications	Appendix C Amendment 238/ Amendment 242		3.11
7	Modifications to Reduce Emergency Diesel Generator Loading	(Auxiliary Feedwater Modifications) (Ref. 8.11)		3.15
8	Control Room Envelope Testing Requirements	Appendix C Amendment 240/ Amendment 244		3.5
9	Control Room Shielding			3.18
10	Emergency Operating Procedures	(AST) (Ref. 8.10)		3.10
11	Control Room Emergency Filtration System Modifications			3.6
12	Primary Auxiliary Building Ventilation System Modifications			3.8
13	Emergency Diesel Generator Loading			3.16
14	Control Room Emergency Filtration System Seismic Requirements			3.7
15	Mitigating Filtration Unit Motors			3.17
16	Eliminate Reliance on Local Manual Action	Appendix C Amendment 241/ Amendment 245 (Extended Power Uprate)		3.12
17	Air Cooled Air Compressor	(Ref. 8.8 and 8.9)		3.13

3.2 License Condition 4.I for Unit 1 and 4.H for Unit 2, FSAR Supplement

License condition:

The FSAR supplement, dated February 25, 2004, as revised, submitted pursuant to 10 CFR 54.21(d), shall be included in the next scheduled update to the FSAR required by 10 CFR 50.71(e)(4) following the issuance of this renewed operating license. Until that update is complete, FPLE [Florida Power and Light Energy] Point Beach¹ may make changes to the programs and activities described in the supplement without prior Commission approval, provided that FPLE Point Beach¹ evaluates such changes pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.

Justification for Deletion:

The license's letter submitted on August 15, 2006 (Reference 1), provided an update to the FSAR in accordance with 10 CFR 50.71(e). License conditions 4.I for Unit 1 and 4.H for Unit 2 are complete; therefore, NextEra proposes removing the license condition.

Staff Evaluation:

The NRC staff verified the letter submitted by Nuclear Management Company, LLC dated August 15, 2006 (Ref. 8.17), providing the periodic update of the FSAR, and noting that license conditions 4.I for Unit 1 and 4.H for Unit 2 are complete. Therefore, the NRC staff concludes that removal of these license conditions is acceptable.

3.3 License Condition 4.J for Unit 1 and 4.I for Unit 2, Completion of Activities

License Condition:

The FSAR supplement, dated February 25, 2004, as revised, describes certain future activities to be completed prior to the period of extended operation. NextEra Energy Point Beach shall complete these activities no later than October 5, 2010, for Unit 1 and March 8, 2013, for Unit 2, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC staff inspection.

Justification for Deletion:

By letter dated September 3, 2010, NextEra notified the NRC that activities required to be completed no later than October 5, 2010, had been completed for Unit 1 and could be verified by NRC inspection. Similarly, in a letter dated February 28, 2013, NextEra notified the NRC that the activities required to be complete no later than March 8, 2013, had been completed for Unit 2 and could be verified by NRC inspection. License conditions 4.J for Unit 1 and 4.I for Unit 2 are complete; therefore, NextEra proposes removing the license conditions.

Staff Evaluation:

The NRC staff verified the letter submitted by NextEra Energy Point Beach, LLC dated September 3, 2010, for Unit 1 (Ref. 8.12), and letter dated February 28, 2013, for Unit 2 (Ref. 8.6), provided the notification of completion of activities to support entry into the period of extended operation of PBNP, Units 1 and 2. Therefore, the NRC staff concludes that

¹ On April 16, 2009, the name "FPLE Point Beach, LLC" was changed to "NextEra Energy Point Beach, LLC."

license conditions 4.J for Unit 1 and 4.I for Unit 2 are complete and removal of these license conditions is acceptable.

3.4 Control Room Emergency Filtration System (CREFS)

3.4.1 Description of the CREFS

The control room ventilation system (VNCR) is designed to provide heating, ventilation, air conditioning, and radiological habitability for the control and computer rooms, which are both within the control room envelope (CRE). The CREFS (F-16) is a subset of the VNCR system that ensures habitability of the control room during challenges from radioactivity, hazardous chemicals, and fire byproducts. The CREFS consists of one emergency air filtration unit, two emergency fans, two recirculation fans, and required ducts, valves, instrumentation, doors, barriers, and dampers necessary to establish the required flowpaths and isolation boundaries that recirculate and filter the air within the CRE. The emergency air filtration unit has no installed heaters to control the humidity of the process air entering the carbon adsorber bed.

The PBNP UFSAR 2009, page I.2-7 of 8, Section I.2.6b, "Ventilation and Exhaust Systems," reads:

Decontamination factors of 10 for iodines through carbon adsorbers and 100 for particulates through high-efficiency particulate air (HEPA) filters are assumed for the evaluation, consistent with recommendations in Reference 1 [Draft Regulatory Guide 1.BB, "Calculation of Releases of Radioactive Materials in Liquid and Gaseous Effluents From Pressurized Water Reactors (PWR's)," Sept. 1975.]. All carbon adsorbers at PBNP have a bed depth of 2 inches and a maximum face velocity of 40 feet per minute. Periodic testing of filtration equipment indicates decontamination factors at least equal to the above values. Credit is taken only for those filters which are normally in service.

Automatic actuation of the CREFS in the emergency mode is initiated by a containment isolation signal, high radiation signal from the control room monitor, or a high radiation signal from the noble gas monitor located in the supply duct to the control room. Operating in the emergency mode, CREFS provides for control room pressurization to limit in-leakage, and makeup and recirculated air is filtered through HEPA and charcoal filters to remove contaminants.

3.4.2 Licensee's Proposed Changes for the CREFS

The license proposes to change TS 5.5.10 to increase the value of methyl iodide penetration in TS 5.5.10.c from ≤ 1.0 percent to ≤ 2.5 percent as shown below.

- c. Demonstrate for the Control Room Emergency Filtration System (F-16) that a laboratory test of a sample of the charcoal adsorber, when obtained in accordance with the methodology of [American National Standards Institute] ANSI N51 0-1980, Section 13, excluding subsection 12.3, shows the methyl iodide penetration $\leq 2.5\%$, when tested in accordance with [American Society for Testing Materials] ASTM D3803-1989 at a temperature of 30°C and a relative humidity of 95%, applying the tolerances of ASTM D3803-1989.

3.4.3 Staff Evaluation

Change in methyl iodide penetration test criterion

The change in methyl iodide penetration from ≤ 1.0 percent to ≤ 2.5 percent is based on the credit taken in the dose consequence analysis for the removal of methyl (organic) iodide as a result of the operation of the control room filter system. As shown in Table 3.2-9 of the NRC staff's AST SE entitled, "Point Beach Nuclear Plant Units 1 and 2 Control Room Parameters," the control room dose consequence analyses credits a removal efficiency of 95 percent for methyl iodide (Ref. 8.10). As shown below, Table 1 from RG 1.52, Revision 3, indicates that in order to credit an elemental and organic removal efficiency of 95 percent the acceptance criterion is a penetration of ≤ 2.5 percent when tested in accordance with ASTM D-3803-1991.

**Table 1 from RG 1.52, Revision 3
Laboratory Tests for Activated Carbon**

Activated Carbon Total Bed Depth	Maximum Assigned Credit for Activated Carbon Decontamination Efficiencies		Methyl Iodide Penetration Acceptance Criterion for Representative Sample
2 inches	Elemental iodine	95%	Penetration $\leq 2.5\%$ when tested in accordance with ASTM D-3803-1989
	Organic iodide	95%	
4 inches or greater	Elemental iodine	99%	Penetration $\leq 0.5\%$ when tested in accordance with ASTM D-3803-1989
	Organic iodide	99%	

The licensee's proposed change in testing criteria is consistent with the RG 1.52 guidance for the credit assumed in the AST dose consequence analyses.

TS 5.5.10.c currently requires that testing a sample of the carbon adsorber from CREFS filter (F-16) results in a methyl iodide penetration of less than or equal to 1.0 percent.

The NRC staff notes that the TS 5.5.10 current acceptance criteria, of a methyl iodide penetration of less than or equal to 1.0 percent, is consistent with the acceptance criteria of Table 2, "Laboratory Tests for Activated Carbon," of RG 1.52, Revision 2 (Ref. 8.26), for a 2 inch "Activated Carbon Bed Depth" air filtration system operated outside the containment.

Generic Letter (GL) 99-02, "Laboratory Testing of Nuclear-Grade Activated Charcoal" (Ref. 8.23), and NUREG-1431, Standard Technical Specifications (STS), Westinghouse Plants (Ref.8.22), discuss that the methyl iodide efficiency should be the value that was incorporated in the licensee's accident analysis. These two NRC documents also state that the staff will accept a safety factor greater than or equal to two when ASTM D3803-1989 (Ref. 8.21), is used with 30 ° C (86 ° F) and 95 percent relative humidity (RH) for charcoal bed filtration trains without humidity control.

In particular, the sixth paragraph of "Discussion" contained in GL 99-02, reads in part:

Because ASTM D3803-1989 is a more accurate and demanding test than older tests, addressees that upgrade their TS to this new protocol will be able to use a safety factor as low as 2 for determining the acceptance criteria for charcoal filter efficiency. This safety factor can be used for systems with or without humidity control because the lack of humidity control is already accounted for in the test conditions (systems without humidity control test at 95 percent RH and systems with humidity control can test at 70 percent RH). The staff has previously approved reductions in the safety factor for plants adopting the ASTM D3803-1989 standard on a case-by-case basis. (The staff plans to make conforming changes to RG 1.52.)

The NRC staff notes that Table 1, "Laboratory Tests for Activated Carbon" of RG 1.52, Revision 3 (Ref. 8.27) for a 2 inch "Activated Carbon Total Bed Depth" exhibits a "Methyl Iodide Penetration Acceptance Criterion for Representative Sample" of ≤ 2.5 percent when tested in accordance with ASTM D3803-1989. This fulfilled the parenthetical statement in the preceding GL 99-02 excerpt.

To ensure consistency with previous NRC staff licensing actions, the staff reviewed both Oconee Units 1, 2, and 3 TS 5.5.12 and Davis-Besse TS 5.5.10 "Ventilation Filter Testing Program (VFTP)" as examples of precedents set for NextEra's proposed LAR. It was noted in GL 99-02 that these nuclear plants were granted emergency TS amendments after each plant's licensee determined that the laboratory tests being performed at the time, were not in compliance with the plant's technical specifications.

The PBNP TS 5.5.10c already requires testing in accordance with ASTM D3803-1989 at a temperature of 30 °C and a relative humidity of 95 percent. This is consistent with the guidance of GL 99-02, since the PBNP emergency air filtration unit lacks installed heaters to control the humidity of the process air entering the carbon adsorber beds. Accordingly, the licensee has proposed in the LAR to apply a safety factor of two. As noted above, this is allowed by the guidance of GL 99-02.

The CREFS filter efficiency for organic (methyl) iodine is 95 percent as documented Table 3.2-9, "Point Beach Nuclear Plant Unit[s] 1 and 2 Control Room Parameters," page 14 of Attachment 1, of the SE for PBNP Amendments 240 and 244 (Ref. 8.10) regarding AST.

Therefore, for the purpose of analysis, the required Methyl Iodide Efficiency equals 95 percent and the lowest permissible Safety Factor equals two (2).

Attachment 2, page 1 of 2, of GL 99-02 and TS 5.5.11, page 5.5-11 of NUREG-1431, STS provide the following formula for allowable penetration:

Allowable Penetration = [(100 percent - Methyl Iodide Efficiency * for charcoal credited in the licensee's accident analysis)/safety factor].

*this value should be the efficiency that was incorporated in the licensee's accident analysis which was reviewed and approved by the staff in a safety evaluation

Using the applicable values:

$$\begin{aligned}\text{Allowable Penetration} &= [(100 \text{ percent} - 95 \text{ percent})/2.0] \\ &= 2.5 \text{ percent}\end{aligned}$$

Therefore, using the required PBNP CREFS (F-16) organic (methyl) iodine filter efficiency of 95 percent and a safety factor of 2 in the above equation yields an allowable penetration of 2.5 percent. This is in agreement with the licensee's proposed change to TS 5.5.10c.

Based on the above, the NRC staff concludes that the requirements of both PBNP GDC 11 and 10 CFR 50.67 will continue to be satisfied with implementation of the licensee's proposed change to TS 5.5.10c in Renewed Facility Operating License Nos. DPR-24 and DPR-27. Accordingly, the staff concludes the change to TS 5.5.10c is acceptable.

3.5 License Appendix C, Additional Condition Amendment 240 for Unit 1 and Amendment 244 for Unit 2, Control Room Envelope Testing Requirements

License Condition:

Appendix C, Page C-3 of both Renewed Facility Operating Licenses DPR-24 and DPR-27 reads:

Upon implementation of Amendment Nos. 240/244 adopting TSTF-448, Revision 3, the determination of CRE unfiltered air inleakage as required by SR 3.7.9.6, in accordance with TS 5.5.18.c.(i), the assessment of CRE habitability as required by Specification 5.5.18.c.(ii), and the measurement of CRE pressure as required by Specification 5.5.18.d, shall be considered met. Following implementation:

- a. The first performance of SR 3.7.9.6, in accordance with TS 5.5.18.c.(i), shall be within 18 months of implementation of this amendment.
- b. The first performance of the periodic assessment of CRE habitability, Specification 5.5.18.c.(ii), shall be within three (3) years of completion of the testing prescribed in item a. above.
- c. The first performance of the periodic measurement of CRE pressure, Specification 5.5.18.d, shall be within 18 months of implementation of this amendment.

Justification for Deletion:

NextEra performed SR 3.7.9.6 in accordance with TS 5.5.18.c.(i) on November 26, 2012. The periodic assessment of CRE habitability was completed in February 2013. NextEra obtained measurements of CRE pressure in accordance with TS 5.5.18.d in September 2012. The performance of the test requirements in this license condition have been completed; therefore, the license condition is complete and NextEra proposes removing this license condition.

Staff Evaluation:

The NRC staff notes that "Appendix C Additional Conditions Operating License DPR-24," page C-3, Amendment 240 and "Appendix C Additional Conditions Operating License DPR-27," page C-3, Amendment 244 for the subject License Condition, both list an "Implementation Date" of "Immediately."

The NRC staff issued Amendments 240 and 244 for Renewed Facility Operating License Nos. DPR-24 and DPR-27, respectively, on April 14, 2011. In the enclosure to the LAR, on pages 4 and 5 of 12, the licensee indicated that:

- a. The first performance of SR 3.7.9.6, in accordance with Specification 5.5.18.c.(i), was completed on November 26, 2012.
- b. The first performance of the periodic assessment of CRE habitability, in accordance with TS 5.5.18.c.(ii), was completed in February 2013.
- c. The first performance of the periodic measurement of CRE pressure, in accordance with TS 5.5.18.d was completed in September 2012.

The NRC staff notes that the first performance of SR 3.7.9.6 appears to fail the 18-month completion requirement. However, in a letter dated May 19, 2011, page 2 (Ref. 8.7), the licensee stated that "Amendments Nos. 240 and 244 are being implemented concurrently with the Auxiliary Feedwater (AFW) Amendment Nos. 238 and 242 for PBNP, Units 1 and 2, respectively" In a letter dated March 25, 2011 (Ref. 8.11), the NRC staff issued PBNP Amendment No. 238, Enclosure 1, page 2, for Renewed License No. DPR-24 and Amendment No. 242, Enclosure 2, page 2, for Renewed License No. DPR-27. Both Amendment Nos. 238 and 242 stated that "The license amendment is effective as of its date of issuance shall be implemented within 180 days."

Therefore, the NRC staff concludes that the requirements of the license conditions associated with Amendment 240 (DPR-24) and Amendment 244 (DPR-27) pertaining to the control room envelop testing requirements have been satisfied. Accordingly, the staff concludes removal of these two license conditions from Renewed Facility Operating License Nos. DPR-24 and DPR-27 is acceptable.

3.6 License Appendix C, Additional Condition Amendment 240 for Unit 1 and Amendment 244 for Unit 2, Control Room Emergency Filtration System Modifications

License Condition:

Appendix C, page C-3, of both Renewed Facility Operating Licenses DPR-24 and DPR-27, reads:

NextEra Energy Point Beach, LLC shall modify the control room emergency filtration system (CREFS) to create a new alignment for the accident mode that provides a combination of filtered outside air and filtered recirculation air. The modifications shall include redundancy for all CREFS active components that must reposition from their normal operating position, and auto-start capability on loss of offsite power in conjunction with a containment isolation or high control room radiation signal from an

emergency diesel generator supplied source for the CREFS fans required for the new system alignment.

The "Implementation Date" for both units reads, "No later than the Unit 2 (2011) refueling outage."

Justification for Deletion:

NextEra modified the CREFS in accordance with this license condition in June 2011; therefore, NextEra proposes to remove this license condition.

Staff Evaluation:

In the enclosure to the LAR, page 6 of 12, the licensee stated that, "NextEra modified the CREFS in accordance with this license condition in June 2011; therefore, NextEra proposes to remove this license condition."

The NRC staff reviewed the PBNP UFSAR, Section 9.8, "Control Room Ventilation System (VNCR)," dated 2012, to confirm that the current plant "Design Basis" and the current "System Design and Operation" description both reflect the requirements of the license condition.

UFSAR Section 9.8, page 9.8-1 of 11, reads in part:

" ... CREFS consists of one emergency air filtration unit, two emergency fans, two recirculation fans, and required ducts, valves, instrumentation, doors, barriers, and dampers necessary to establish the required flowpaths and isolation boundaries that recirculate and filter the air within the CRE. ..."

UFSAR Section 9.8.1, page 9.8-2 of 11, second paragraph, reads in part:

"CREFS is capable of operating in five different modes as described in Section 9.8.2. Mode 5 places the system in the configuration necessary for radiological habitability by providing for control room pressurization to limit inleakage, makeup and recirculation through HEPA and charcoal filters to remove contaminants. ..."

Mode 5, as described in UFSAR, Section 9.8.2, "System Design and Operation," fully satisfies the requirements of the two license conditions proposed for deletion.

Therefore, the NRC staff concludes that the requirements of the license conditions associated with Amendment 240 (DPR-24) and Amendment 244 (DPR-27) pertaining to modifying the control room emergency filtration system has been completed. Accordingly, the staff concludes that removal of these two license conditions from Renewed Facility Operating License Nos. DPR-24 and DPR-27 is acceptable.

3.7 License Appendix C, Additional Condition Amendment 240 for Unit 1 and Amendment 244 for Unit 2, Control Room Emergency Filtration System Seismic Requirements

License condition:

Appendix C, Page C-4 of both Operating Licenses DPR-24 and DPR-27 reads:

NextEra Energy Point Beach, LLC shall install and support CREFS mitigating filtration unit(s) and associated ductwork and bubble tight dampers to Seismic Class I requirements as defined in FSAR Appendix A.5. The mitigating filtration unit(s) shall be seismically qualified in accordance with the guidelines provided in the Seismic Qualification Utility Group (SQUG) Generic Implementation Procedure for Seismic Verification of Nuclear Plant Equipment, Revision 2, as corrected on February 14, 1992, and in the December 2006, Electric Power Research Institute (EPRI) Final Report 1014608, "Seismic Evaluation Guidelines for HVAC Duct and Damper Systems: Revision to 1007896," as applicable.

The "Implementation Date" for both units reads "No later than the Unit 2 (2011) refueling outage."

Justification for Deletion:

NextEra completed the upgrade to the CREFS required by this license condition in June 2011. This license condition is complete; therefore, NextEra proposes to remove this license condition.

Staff Evaluation:

In the enclosure to the LAR, page 7 of 12, the licensee stated, "NextEra completed the upgrade to the CREFS required by this license condition in June 2011. This license condition is complete; therefore, NextEra proposes to remove this license condition."

The NRC staff reviewed the PBNP, UFSAR, Section 9.8, "Control Room Ventilation System (VNCR)," dated 2012, to confirm that either the current plant "Design Basis" or the current "System Design and Operation" description reflect the requirements of the license condition.

UFSAR Section 9.8, page 9.8-3 of 11, fourth paragraph, reads in part:

" ... The filtration unit, ductwork, and associated dampers are installed and supported to seismic Class I requirements. The seismic qualification is in accordance with the guidelines of Reference 5 and Reference 6. ... "

The NRC staff notes that a review of the PBNP, UFSAR, Section 9.8.5, "References," dated 2012, reveals that Reference 5 and Reference 6 as invoked in the above UFSAR excerpt are identical to guidelines invoked in the license condition.

Therefore, as documented in the PBNP UFSAR, the license condition requirement to install the control room emergency filtration system components in accordance with Seismic Class I guidelines has been completed. Accordingly, the NRC staff concludes that removal of these two license conditions from Renewed Facility Operating License Nos. DPR-24 and DPR-27 is acceptable.

3.8 License Appendix C, Additional Condition Amendment 240 for Unit 1 and Amendment 244 for Unit 2, Primary Auxiliary Building Ventilation System Modifications

Description of the Primary Auxiliary Building Ventilation System (VNPAB)

The auxiliary building ventilation air is supplied by a central supply fan which includes an air filter, heating coils, and service water supplied cooling coils. Sufficient outside air is supplied to

maintain a once-through system with provisions available to recirculate air from the auxiliary building central area. The system is balanced to maintain the auxiliary building at a slightly negative pressure with respect to outside pressure and adjacent building pressures. This is accomplished by providing an exhaust flow capacity larger than the supply capacity. All the exhaust air is filtered through roughing and high efficiency filters for removal of particulates.

The VNPAB exhaust system consists of two filter fans, two stack fans, and the associated ductwork, filter housings, and dampers necessary to ensure the required exhaust flow path can be maintained. Each of the two filter fans and each of the two stack fans are powered by independent safety-related power supplies with emergency diesel generator (EDG) backup.

The VNPAB system is classified as nonsafety-related; however, components in the exhaust system required to direct radioactive releases in the auxiliary building to the vent stack are classified as Augmented Quality. The VNPAB exhaust system design provides redundancy for all active mechanical components and active and passive electrical components needed to provide primary auxiliary building (PAB) exhaust flow.

The third paragraph of the "System Evaluation" subsection of PBNP, UFSAR, Chapter 9.5, "Primary Auxiliary Building Ventilation System," dated 2010, states:

No credit is given for the VNPAB exhaust system in the control room or offsite dose bounding analyses described in UFSAR, Chapter 14.3.5, Radiological Consequences of a Loss of Coolant Accident.

License condition:

Appendix C, Page C-3 of both Renewed Facility Operating Licenses DPR-24 and DPR-27, reads:

NextEra Energy Point Beach, LLC shall modify the primary auxiliary building (PAB) ventilation system (VNPAB) to ensure redundancy of active components needed to operate the PAB exhaust system. VNPAB components required to direct radioactive releases in the PAB to the vent stack shall be upgraded to an augmented quality status. No credit is taken by AST for the PAB charcoal filters. NextEra Energy Point Beach, LLC shall revise PBNP [emergency operating procedures] EOPs to address starting the VNPAB fans.

The "Implementation Date" for both units reads, "No later than the Unit 2 (2011) refueling outage."

Justification for Deletion:

As described in PBNP UFSAR section 9.5, the VNPAB exhaust system design provides redundancy for all active mechanical components and active and passive electrical components needed to provide primary auxiliary building (PAB) exhaust flow. Components in the exhaust system required to direct radioactive releases in the PAB to the vent stack are classified as augmented quality.

In addition, the emergency operating procedures ensure that the VNPAB exhaust system is in operation within 30 minutes following the alignment of the residual heat removal system to the containment sump recirculation mode of operation.

The requirements of this license condition have been met; therefore, NextEra proposes to remove this license condition.

Additionally, NextEra submitted LAR 279, Elimination of Technical Specification 3.7.14, Primary Auxiliary Building Ventilation, on January 15, 2016, to delete TS 3.7.14, Primary Auxiliary Building Ventilation (VNPAB) in its entirety on the basis that the VNPAB is not credited for accident mitigation and meets none of the criteria of 10 CFR 50.36 for inclusion in the TS.

Staff Evaluation:

In the enclosure to the LAR, page 6 of 12, the licensee stated that, "The requirements of this license condition have been met; therefore, NextEra proposes to remove this license condition."

The NRC staff reviewed the PBNP, UFSAR, Section 9.5, "Primary Auxiliary Building Ventilation System," dated 2010 with the most recent revisions identified to confirm that Subsections: 9.5.1, "Design Basis;" 9.5.2, "System Design and Operation;" 9.5.3, "Safety Evaluation;" and 9.5.4, "Required Procedures and Tests" collectively reflect the requirements of the two license conditions.

The second paragraph of Subsection 9.5.2 reads, in part:

The VNPAB exhaust system consists of two filter fans (W-30A&B), two stack fans (W-21A&B), and the associated ductwork, filter housings, and dampers necessary to ensure the required exhaust flow path can be maintained. Each of the two filter fans and each of the two stack fans are powered by independent safety related power supplies with EDG backup. ...

The third paragraph of Subsection 9.5.3 reads, in part:

No credit is given for the VNPAB exhaust system in the control room or offsite dose bounding analysis described in FSAR Chapter 14.3.5, Radiological Consequences of a Loss of Coolant Accident.

The fifth paragraph of Subsection 9.5.3 reads, in part:

The VNPAB system is classified as non-safety related, however components in the exhaust system required to direct radioactive releases in the PAB to the vent stack are classified as AQ (Augmented Quality). The seismic adequacy of the VNPAB exhaust system has been demonstrated The VNPAB exhaust system design provides redundancy for all active mechanical components and active and passive electrical components needed to provide PAB exhaust flow. ...

The second paragraph of Subsection 9.5.4 reads in part:

Emergency Operating Procedures (EOP)s ensure that the VNPAB exhaust system is in operation within 30 minutes following the alignment of the residual heat removal system to the containment sump recirculation mode of operation

Based on the above licensee statement and the foregoing PBNP UFSAR, excerpts, the NRC staff concludes that the requirements of the license conditions associated with Amendment 240

(DPR-24) and Amendment 244 (DPR-27) pertaining to the Primary Auxiliary Building Ventilation System Modifications have been completed. Accordingly, the NRC staff concludes that removal of these two license conditions from Renewed Facility Operating License Nos. DPR-24 and DPR-27 is acceptable.

3.9 License Condition 4.E. (Unit 1 and 2), Safety Injection Logic

License Condition:

The licensee is authorized to modify the safety injection actuation logic and actuation power supplies and related changes as described in licensee's application for amendment dated April 27, 1979, as supplemented May 7, 1979. In the interim period until the power supply modification has been completed, should any DC [direct current] powered safety injection actuation channel be in a failed condition for greater than one hour, the unit shall thereafter be shutdown using normal procedures and placed in a block-permissive condition for safety injection actuation.

Justification for Deletion:

The license condition specified interim actions until the power supply modification was complete. The power supply modification was completed on May 18, 1979. License Condition 4.E is complete; therefore, NextEra proposes removing it.

Staff Evaluation:

The NRC staff reviewed the proposed deletion of license condition 4.E., Safety Injection Logic (Units 1 and 2). The license condition stated that the licensee is authorized to modify the safety injection actuation logic and actuation power supplies and related changes as described in licensee's application for amendment dated April 27, 1979, as supplemented May 7, 1979. The justification for deletion specified that the power supply modification was completed on May 18, 1979, thus the license condition 4.E is complete. The NRC staff reviewed the SE (Ref. 8.20) for the LAR dated April 27, 1979, as supplemented May 7, 1979. The NRC staff reviewed the PBNP UFSAR for the sections related to this change. The NRC staff noted that Table 7.3-1 of the UFSAR discusses the changes associated with the LAR and confirmed the modification to the safety injection logic was completed. The NRC staff finds the deletion of this license condition acceptable based on its review of the UFSAR related to the changes made in SE.

3.10 License Appendix C, Additional Condition Amendment 240 for Unit 1 and Amendment 244 for Unit 2, EOPs

License Condition:

NextEra Energy Point Beach, LLC shall revise PBNP Emergency Operating Procedures (EOPs) to direct continued containment spray while on sump recirculation.

Justification for Deletion:

NextEra revised the EOPs with instructions to maintain containment spray for 2 hours while on sump recirculation; therefore, NextEra proposes to remove this license condition.

Staff Evaluation:

LAR 241 (Ref. 8.16) contained the following commitment:

FPL Energy Point Beach will revise PBNP Emergency Operating Procedures (EOPs) to direct continued [containment spray] CS while on sump recirculation, if containment radiological conditions and/or core damage indicates it is required. These procedure changes will be implemented following NRC approval of this LAR 241 "Alternative Source Term" and following the completion of each unit specific installation of the CS and [residual heat removal] RHR system modifications to provide throttling capability during the ECCS recirculation phase.

As described in the following excerpt from the NRC staff's AST SE, the LOCA dose consequence analysis assumes that the containment spray (CS) will be maintained throughout the injection phase and continued for 2 hours during the ECCS recirculation phase:

The dose projections for the LOCA radiological analysis assume that CS is maintained throughout the injection phase and continued for two hours during the ECCS recirculation phase. It is also assumed that there will be no more than a 20-minute spray interruption to switch from injection to recirculation spray. In order to credit a manual operator action, the time limit for taking the manual action must allow for the operator to diagnose plant conditions and take the necessary action. The proposed EOP 1.3, Step 33, directs operator actions to align CS for recirculation, eliminating the diagnosis aspect of the action. The new EOP-1.3 actions are carried out in the control room. The licensee committed to validate the 20-minute interruption to establish CS recirculation in accordance with the approved administrative procedure governing the EOP verification and validation process. The licensee stated that alignment to recirculation spray has been demonstrated on the simulator to ensure that the necessary action can be accomplished well within 20 minutes. Timing requirements will be confirmed and documented as part of the verification and validation process.

The NRC staff reviewed the proposed deletion of the license condition for Amendment 240/Amendment 244, EOPs. The license condition stated that NextEra shall revise the PBNP EOPs to direct continued containment spray while on sump recirculation. NextEra thus revised the EOPs with instructions to maintain containment spray for 2 hours while on sump recirculation. In the supplemental letter dated July 11, 2016, the licensee confirmed that the PBNP EOPs were revised with instructions to maintain containment spray for 2 hours while on sump recirculation. Based on this confirmation in the supplemental letter, the NRC staff finds the deletion of this license condition to be acceptable.

3.11 License Appendix C, Additional Condition Amendment 238 for Unit 1 and Amendment 242 for Unit 2, Auxiliary Feedwater (AFW) Modifications

License Condition:

NextEra Energy Point Beach, LLC shall modify the motor-driven auxiliary feedwater (MDAFW) and the turbine driven auxiliary feedwater [TDAFW] pump systems to ensure they are powered from independent DC power sources.

Justification for Deletion:

NextEra completed the modifications to the AFW system in June 2011; therefore, NextEra proposes to remove this license condition.

Staff Evaluation:

The NRC staff reviewed the proposed deletion of the license condition for Amendment 238 for Unit 1 and Amendment 242 for Unit 2, AFW modifications. The license condition stated that NextEra shall modify the MDAFW and TDAFW pump systems to ensure they are powered from independent dc power sources. The justification for deletion specified that the AFW modifications were completed in June 2011, thus, the license condition is complete.

The NRC staff reviewed the SE dated March 25, 2011(Ref. 8.11) for the LAR dated April 7, 2009, and the PBNP UFSAR, for the sections related to this change. Section 10.2.1 of the UFSAR discusses the changes associated with the LAR and confirmed the modification to the AFW system was completed. The UFSAR confirms that the redundant supplies are provided by two 100 percent capacity pump systems using different sources of power for the pumps and different trains of dc for valve and control power. The dc power for the PBNP, Units 1 and 2, TDAFW pumping systems are also from different trains.

Based on the above licensee statement and the review of the PBNP UFSAR, the NRC staff concludes that the requirements of the license conditions associated with Amendment 238 for Unit 1 and Amendment 242 for Unit 2 have been completed. Accordingly, the NRC staff concludes that removal of these two license conditions from Renewed Facility Operating License Nos. DPR-24 and DPR-27 is acceptable.

3.12 License Appendix C, Additional Condition Amendment 241 for Unit 1 and Amendment 245 for Unit 2, Eliminate Reliance on Local Manual Action

License Condition:

NextEra Energy Point Beach, LLC shall eliminate the reliance on local manual action to gag the Motor Driven and Turbine Driven AFW pump mini-recirculation valves open.

Justification for Deletion:

NextEra completed a modification to address this license condition in June 2011, and therefore, proposes to remove this license condition.

Staff Evaluation:

The NRC staff reviewed the proposed deletion for Amendment 241/Amendment 245, Eliminate Reliance on Local Manual Action. The license condition stated that NextEra shall eliminate the reliance on local manual action to gag the MDAFW and TDAFW pump mini-recirculation valves open. This license condition was necessary for a decrease in core damage frequency (CDF) and large early release frequency (LERF) for the EPU. The justification for deletion specified that the modifications were completed in June 2011, thus, the license condition is complete.

The NRC staff reviewed the SE dated May 3, 2011for issuance of license amendments regarding extended power uprate (Ref. 8.8 and 8.9) for the LAR dated April 7, 2009, and the

PBNP UFSAR, for the sections related to this change regarding elimination of the reliance on local manual action to gag the MDAFW and TDAFW pump mini-recirculation valves open. Section 10.2.2 of the UFSAR discusses the changes associated with the LAR and confirmed the modifications to the AFW system to eliminate reliance on the local manual action to gag the MDAFW and TDAFW pump minimum recirculation valve was completed. Section 10.2.2 of the UFSAR states that, the TDAFW pump has a single minimum flow recirculation line, isolated by a fail-closed air-operated valve (AOV) (1/2AF-4002) and the MDAFW pump has two parallel minimum flow recirculation lines, each isolated by a fail-closed AOV (1/2 AF 4073 A(B)). The minimum flow recirculation AOVs have a safety-related function to close to ensure the required AFW flow is not diverted away from the steam generators. These valves also have a safety-related function to open to ensure that a minimum flow is maintained through the pumps to prevent damage due to hydraulic instabilities or increased temperature. Safety-related backup pneumatic systems are provided for the system's air operated valves.

Based on the above licensee statement and the review of the PBNP UFSAR, the NRC staff concludes that the requirements of the license conditions associated with Amendment 241 for Unit 1 and Amendment 245 for Unit 2 have been completed. Accordingly, the NRC staff concludes that removal of these two license conditions from Renewed Facility Operating License Nos. DPR-24 and DPR-27 is acceptable.

3.13 License Appendix C, Additional Condition Amendment 241 for Unit 1 and Amendment 245 for Unit 2, Air Cooled Air Compressor

License Condition:

NextEra Energy Point Beach, LLC shall install a self-cooled (i.e., air-cooled) air compressor capable of supplying Instrument Air. The compressor shall be independent of service water cooling and normally aligned for automatic operation.

Justification for Deletion:

NextEra installed an air-cooled compressor in response to this license condition in November 2011, and, therefore, proposes to remove this license condition.

Staff Evaluation:

The NRC staff reviewed the proposed deletion for Amendment 241/Amendment 245, air cooled air compressor. The license condition stated that NextEra shall install a self-cooled (i.e., air-cooled) air compressor capable of supplying Instrument Air. The compressor shall be independent of Service Water cooling and normally aligned for automatic operation. This license condition was necessary to for a decrease in CDF and LERF for the EPU. The justification for deletion specified that the air-cooled compressor in response to this license condition was installed in November 2011, thus, the license condition is complete.

The NRC staff reviewed the SE dated May 3, 2011 (Ref. 8.8 and 8.9) for the LAR dated April 7, 2009, and the PBNP UFSAR, for the sections related to this change. Section 9.7.2 of the UFSAR states the service air system consists of two air compressors (K-3A and K-3B). Service air compressor K-3A is powered from 480 VAC bus 1-B04 and both the compressor and aftercooler are air cooled. Normally, one service air compressor is running and the other aligned to start automatically when service air or instrument air pressure drops to a preset level.

Based on the above licensee statement and the review of the PBNP UFSAR, the NRC staff concludes that the requirements of the license conditions associated with Amendment 241 for Unit 1 and Amendment 245 for Unit 2 have been completed. Accordingly, the NRC staff concludes that removal of these two license conditions from Renewed Facility Operating License Nos. DPR-24 and DPR-27 is acceptable.

3.14 License Appendix C, Additional Condition Amendment 201 for Unit 1 and Amendment 206 for Unit 2, Relocate Certain Technical Specifications Requirements and Schedule for Performing Surveillance Requirements

The licensee proposed to delete license conditions that were added as a part of the Improved Technical Specification (ITS) conversion, license amendment Nos. 201 and 206 for PBNP, Units 1 and 2, respectively. The NRC staff approved the license amendments on August 8, 2001 (ADAMS Accession No. ML012260052).

The first ITS conversion license condition addressed relocation of information:

The licensee is authorized to relocate certain Technical Specifications requirements previously included in Appendix A to licensee controlled documents, as described in Table R, Relocated Specifications and Removal of Details Matrix, attached to the NRC Staff's safety evaluation dated August 8, 2001. These requirements shall be relocated to the appropriate documents no later than December 31, 2001.

Justification for Deletion:

This license condition is associated with the amendment that approved conversion to the Improved Technical Specification (TS), which required relocating certain requirements from the TS to licensee-controlled documents where further changes to the requirements would be controlled by regulations or other requirements (e.g., in accordance with 10 CFR 50.59). Point Beach implemented the Improved TS on November 20, 2001, and at the same time, issued revision 0 of the Technical Requirements Manual, which contained requirements relocated from the TS as part of the of Improved TS implementation.

The license condition was completed in November of 2001; therefore, NextEra proposes removing this license condition.

The second ITS conversion license condition pertained to the handling of TS SRs added to or modified by the amendment:

The schedule for performing SRs that are new or revised in Amendment 201 [206] shall be as follows:

For SRs that are new in this amendment, the first performance is due at the end of the first surveillance interval that begins on the date of implementation of this amendment.

For SRs that existed prior to this amendment, whose intervals of performance are being reduced, the first reduced surveillance interval begins upon completion of the first surveillance performed after implementation of this amendment.

For SRs that existed prior to this amendment that have modified acceptance criteria, the first performance is due at the end of the first surveillance interval that began on the date the surveillance was last performed prior to the implementation of this amendment.

For SRs that existed prior to this amendment, whose intervals of performance are being extended, the first extended surveillance interval begins upon completion of the last surveillance performed prior to the implementation of this amendment.

Justification for Deletion:

This license condition is associated with the amendment that approved conversion to the Improved TS. Point Beach implemented the Improved TS approximately 14 years ago (November 2001); therefore, the license condition regarding scheduling of surveillance requirements associated with the conversion to the improved TS is complete and NextEra proposes removing this license condition.

Staff Evaluation

In a letter dated November 26, 2001 (ADAMS Accession No. ML020160187), the licensee notified the NRC that the Improved TSs were implemented for PBNP, Units 1 and 2, on November 20, 2001, "in their entirety, along with the associated license conditions."

The NRC staff reviewed the LAR and agrees with the licensee's justifications for deletion of the two license conditions related to the PBNP ITS conversion. The deletion of these license conditions is an administrative change, which the NRC staff finds acceptable because the license conditions have been satisfied and are no longer applicable. The regulation at 10 CFR 50.36 (c)(3) continues to require the licensee to have SRs that assure the quality of plant systems is maintained, assure the plant operates within safety limits, and that limiting conditions for operation are met.

3.15 License Appendix C, Additional Condition Amendment 238 for Unit 1 and Amendment 242 for Unit 2, Modifications to Reduce Emergency Diesel Generator Loading

License Condition:

NextEra Energy Point Beach, LLC shall implement modifications to reduce emergency diesel generator loading such that the maximum loading will not exceed the 2000-hour rating of the EDGs.

Justification for Deletion:

NextEra completed modifications to reduce EDG loading in May 2011; therefore, NextEra proposes to remove this license condition.

Staff Evaluation:

The NRC staff reviewed the proposed deletion of the license condition for Amendment 238/ Amendment 242, Modifications to Reduce Emergency Diesel Generator Loading. The license condition stated that the licensee shall implement modifications to reduce EDG loading such that the maximum loading will not exceed the 2000-hour rating of the EDGs. The licensee's justification for deletion of this license condition was that the modifications to reduce EDG loading were completed in May 2011. In supplemental letter dated April 15, 2010 (Ref. 8.14), NextEra originated this regulatory commitment to initiate specific modifications to remove unnecessary loads from the EDGs. The design modifications included a revised EDG loading calculation which indicated, as shown below, that the loads are within the 2000-hr rating of the EDGs:

	Train A		Train B	
	G-01	G-02	G-03	G-04
Worst Case Load	2817 kW	2817 kW	2831 kW	2831 kW
2000-hour Rating	2850 kW	2850 kW	2848 kW	2848 kW
Margin to 2000-hour Rating	33 kW	33 kW	17 kW	17 kW

In an email dated October 4, 2016 (Ref. 8.2), the NRC staff requested the licensee to provide a summary of the calculations and modifications performed to (1) conclude that the EDG maximum loading is now within the 2000-hour rating of the EDGs, and (2) confirm that these calculations included load additions from the AST modifications which included the AST control room (CR) modifications to automatically start the CR emergency fans documented in letter dated April 14, 2011 (Ref. 8.10), and the new auxiliary feedwater (AFW) pump motor modifications documented in letter dated March 25, 2011 (Ref. 8.11). In supplemental letter dated November 4, 2016 (Ref. 8.1), the licensee stated that "the current calculation for the Emergency Diesel Generator (EDG) Steady State Loading Analysis considers the additional electrical loads associated with the AFW pump/motor modifications; and the AST modifications, including the Control Room emergency fans." Also, the licensee stated that "the modifications included the removal of non-essential loads from the EDG automatic load sequence to ensure that each EDG remains within its 2000-hour rating. The calculation envelopes the injection phase, the recirculation phase, and a main steam line break and it assumes that only one EDG is available to support both units." The licensee provided the following tables which demonstrates that the electrical load for each EDG is within its 2000-hour rating with some margins.

Table 1: EDG Load (kW) • Injection Phase

EDG	G-01	G-02	G-03	G-04
2000-hour Rating	2850.0	2850.0	2848.0	2848.0
Load	2720.0	2720.0	2814.1	2815.1
Margin to 2000-hour Rating	130.0	130.0	33.9	32.9

Table 2: EDG Load (kW) • Recirculation Phase

EDG	G-01	G-02	G-03	G-04
2000-hour Rating	2850.0	2850.0	2848.0	2848.0
Load	2241.0	2241.0	2335.1	2336.1

Margin to 2000-hour Rating	609.0	609.0	512.9	511.9
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Table 3: EDG Load (kW) • Main Steam Line Break

EDG	G-01	G-02	G-03	G-04
2000-hour Rating	2850.0	2850.0	2848.0	2848.0
Load	2811.0	2811.0	2790.1	2791.1
Margin to 2000-hour Rating	39.0	39.0	57.9	56.9

Based on the above information, the NRC staff finds that the worst-case maximum loading on the EDGs are within the 2000-hr rating of the EDGs. Therefore, the staff finds the deletion of this license condition to be acceptable.

3.16 License Appendix C, Additional Condition Amendment 240 for Unit 1 and Amendment 244 for Unit 2, Emergency Diesel Generator Loading

License Condition:

NextEra Energy Point Beach, LLC shall perform Train B Emergency Diesel Generator load testing over a range of 2877 to 2950 kilowatt (kW) at rated power factor. This license condition will remain in effect until implementation of LAR 261 for Unit 2.

Justification for Deletion:

In a letter dated May 19, 2011, NextEra notified the NRC that no action needed to be taken to satisfy this license condition. Amendments 240 and 244 were being implemented concurrently with AFW Amendments 238 and 242 for Units 1 and 2, respectively. Amendments 238 and 242 contained a license condition to implement modifications to reduce EDG loading such that the maximum loading will not exceed the 2000-hour rating of the EDGs and a new surveillance requirement for testing the EDGs at the 2000-hour rating. The 2000-hour rating of the train B EDGs is 2848 kW. Therefore, load testing over a range of 2877 to 2950 kW accounting for automatic AST loads prior to extended power uprate modifications is no longer necessary. There will be no period of time where the AST loading on the Train B EDGs is a concern. Accordingly, there is no action that must be taken to satisfy this license condition. Therefore, NextEra proposes to remove this license condition.

Staff Evaluation:

The NRC staff reviewed the proposed deletion of the license condition for Amendment 240/Amendment 244, EDG loading.

For the EDG loading, the license condition stated that the licensee shall perform Train B EDG load testing over a range of 2877 to 2950 kW at rated power factor. This license condition was to remain in effect until implementation of LAR 261 (Point Beach EPU) for Unit 2 is completed. The licensee's justification for deletion of this license condition states that the worst-case maximum loading on Train B EDGs are within the 2000-hour rating of the EDGs (2848 kW). Therefore, load testing over a range of 2877 to 2950 kW accounting for automatic AST loads prior to EPU modifications is no longer necessary and there is no action that must be taken to satisfy this license condition. In supplemental letter dated November 4, 2016 (Ref. 8.1), the

licensee provided the current EDG steady-state loading analysis tables summary which provides sufficient technical basis to conclude that the Train B EDG loadings are within the 2000-hour rating of the EDGs (2848 kW). Therefore, the NRC staff finds the deletion of this license condition to be acceptable.

3.17 License Appendix C, Additional Condition Amendment 240 for Unit 1 and Amendment 244 for Unit 2, Mitigating Filtration Unit Motors

License Condition:

NextEra Energy Point Beach, LLC shall procure mitigating filtration unit motors equivalent to W-14A/B (equivalent [horsepower] HP), efficiency, power factor, and voltage requirements).

Justification for Deletion:

Next Era completed this license condition in April 2011 and therefore, proposes to remove this license condition.

Staff Evaluation:

The NRC staff reviewed the proposed deletion of the license condition for licensee to procure mitigating filtration unit motors equivalent to W-14A/B. Specifically, the license condition for the mitigating filtration unit motors stated that the licensee shall procure mitigating filtration unit motors equivalent to W-14A/B (equivalent HP, efficiency, power factor, and voltage requirements). The licensee states that these motors were procured in April 2011, and, therefore, proposes to remove this license condition. In supplemental letter dated November 4, 2016 (Ref. 8.1), the staff requested the licensee to provide a comparison of the mitigating filtration unit motors that were procured and installed at PBNP, Units 1 and 2, with W-14A/B motors to conclude that the electrical power requirements for these motors are equivalent (HP, efficiency, power factor, and voltage requirements) and that they did not increase loading on the EDGs. In the supplemental letter dated November 4, 2016, the licensee provided the following comparison chart of the control room filter fan motors (W-14A/B-M) and the mitigating filtration unit motors (W-275-M).

Motor Comparison: W-14A/B-M and W-275-M

Motor Data	W-14A/B-M	W-275-M
Horsepower	7.5 Hp	7.5 Hp
Efficiency	87.5 %	87.5 %
Power Factor	90.5 %	90.5 %
Voltage	460 VAC	460 VAC

Considering the above additional information provided by the licensee, the NRC staff finds that the mitigating filtration unit motors (W-275-M) were an exact replacement of the control room filter fan motors (W-14A/B-M) and they did not increase the loading on the EDGs. Therefore, the staff finds the deletion of this license condition to be acceptable.

3.18 License Appendix C, Additional Condition Amendment 240 for Unit 1 and Amendment 244 for Unit 2, Control Room Shielding

LAR 241 (Ref. 8.16) contained the following commitment:

FPL Energy Point Beach will modify the PBNP CR radiation shielding to ensure [control room] CR habitability requirements are maintained. This modification will be installed under 10 CFR 50.59 and is scheduled to be completed following NRC approval of LAR 241 "Alternative Source Term" at the Unit 1 (2010) refueling outage.

This modification to the CR shielding will install permanent shielding that replaces the temporary shielding installed in response to NUREG-0737 Item II1.D.3.4. Current radiological analyses assume placement of 0.5 inches of lead-equivalent thickness shielding for the window and 0.25 inches of lead-equivalent thickness shielding for the doors to reduce the post-accident dose from the passing plume. The temporary shielding is being replaced with permanent shielding and additional shielding is being installed around the [control room envelope] CRE. The shine doses to the CR operators will be reduced by shielding modifications made to the CR outside of the CRE.

Amendment Nos. 240 and 244 contained the following license condition:

NextEra Energy Point Beach, LLC shall modify the PBNP control room (CR) radiation shielding to ensure CR habitability requirements are maintained.

Justification for Deletion:

NextEra modified the control room shielding in May 2011; therefore, NextEra proposes removing this license condition.

Staff Evaluation:

As described in the following excerpt from the NRC staff's AST SE (Ref. 8.10), the licensee took credit for the additional CR shielding that would be installed in accordance with the stated license condition.

The licensee analyzed the gamma shine dose to the CR operators for the bounding [loss of coolant accident] LOCA accident. The dose contribution in the CR due to direct shine from the external cloud and from contained sources is analyzed. The external cloud contribution includes containment leakage, [emergency core cooling system] ECCS leakage, and [reactor water storage tank] RWST back-leakage. The contained sources include shine from the containment structure and the CR heating, ventilation, and air conditioning filter. The 30-day deep dose equivalent to a CR operator due to the airborne source in containment, the passing plume source and the CR filter source is calculated. The analysis takes credit for shielding modifications to the CRE, including the CR walls and ceiling.

PBNP FSAR, Section 14.3.5, states the following:

The dose contribution in the CR due to direct shine from the external cloud and from contained sources is 0.32 rem. The external cloud contribution includes containment leakage, ECCS leakage, and RWST back-leakage. The contained sources include

shine from the containment structure and the control room charcoal and [high efficiency particulate air] HEPA ventilation filters. The analysis takes credit for control room walls and ceiling and shielding modifications to the control room envelope done per Engineering Change (EC) 11691 (258119) (Reference 3 [Engineering Change EC 11691 (258119), Revision 3, "Addition of Control Room Shielding," July 12, 2011]). The analysis assumed an operator located 5 feet from the control room east window (Reference 9 [Calculation 129187-M-0105, Revision 1, Control Room Direct Shine Dose Due to a Loss of Coolant Accident Following Extended Power Uprate and Using Alternative Source Term Methodology, Approved April 28, 2011]).

Computer code SW-QADCGGP was used to calculate the direct shine dose to an operator in the control room from the airborne source inside containment, external plume source, and the control room charcoal and HEPA filter sources. SW-QADCGGP is a Shaw S&W version of the industry standard point-kernel radiation shielding computer code QAD-CGGP (Reference 2 [QAD-CGGP, "A Combinatorial Geometry Version of QAD-P5A, A Point Kernel Code System for Neutron and Gamma-Ray Shielding Calculations Using the GP Buildup Factor."]). Stone & Webster computer program [Passive/Evolutionary Regulatory Consequence] PERC 2 was used to calculate the radiation source term in post-LOCA containment atmosphere, in the external plume passing the control room due to containment and ECCS leakage, and in the control room emergency filters due to containment and ECCS leakage.

The following table is reproduced from the PBNP FSAR and shows that the calculated dose consequences from the maximum hypothetical accident meet the NRC's acceptance criteria.

PBNP FSAR Table 14.3.5-6 Large Break Offsite and Control Room Doses

	Dose (rem TEDE)	Acceptance Criteria (rem TEDE)
Exclusion Area Boundary (0.5 - 2.5 hours)	14.0	25
Low Population Zone (0 - 30 days)	1.4	25
Control Room (0 – 30 days)		
All Pathways (excluded shine)	4.4	
Shine	0.32	
Total Dose	4.72	5.0

As documented in the PBNP FSAR the commitment to install CR shielding control to ensure habitability requirements are maintained has been fulfilled and, therefore, the licensee's proposal to remove this license condition is acceptable to the NRC staff.

4.0 SUMMARY

The NRC staff reviewed the guidance documents cited in the regulatory evaluation in relation to LAR 280 with respect to the licensee's radiological dose consequences analyses. The NRC staff finds that the proposed removal of the referenced license conditions are acceptable since the technical evaluation has demonstrated that these license conditions have been fulfilled. In addition, the proposed change to TS 5.5.10 of the VFTP is acceptable since the revised value conforms to the guidance identified in Section 2.0 above. The NRC staff also finds, with

reasonable assurance that the estimates of the EAB, LPZ, and CR doses comply with the acceptance criteria specified in Section 2.0. The NRC staff further finds with reasonable assurance that PBNP, Units 1 and 2, as modified by this license amendment, will continue to provide sufficient safety margins with adequate defense-in-depth to address unanticipated events and to compensate for uncertainties in accident progression and analysis assumptions and parameters. Therefore, the proposed LAR is acceptable with respect to the radiological consequences of design basis accidents.

The NRC staff evaluated the licensee's proposed changes to the renewed facility operating licenses to remove license conditions that have been completed and are no longer in effect. Based on the evaluation discussed above, the NRC staff concludes that the proposed removal of completed license conditions that have been completed and are no longer in effect are acceptable and that PBNP, Units 1 and 2, continue to meet the requirements of 10 CFR 50.90. Therefore, the staff finds the proposed changes acceptable and consistent with the NRC regulations and the regulatory guidance.

5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Wisconsin State official was notified of the proposed issuance of the amendments. The State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or change a surveillance requirement. The NRC staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluent that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding (81 FR 24662). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

8.0 REFERENCES:

- 8.1 NextEra Energy Point Beach to the NRC, letter (NRC 2016-0048) dated November 4, 2016, Response to Request for Additional Information – Point Beach Nuclear Plant, Units 1 and 2 – LAR 280 – Removal of Completed License Conditions and Change to the Ventilation Filter Testing Program (ADAMS Accession No. ML16309A329).

- 8.2 NRC email dated October 4, 2016, Request for Additional Information – Point Beach Nuclear Plant, Units 1 and 2 – LAR 280 - Removal of Completed License Conditions and Change to the Ventilation Filter Testing Program (ADAMS Accession No. ML16278A666).
- 8.3 NextEra Energy Point Beach to the NRC, letter (NRC 2016-0029) dated July 11, 2016, Response to Request for Additional Information – Point Beach Nuclear Plant, Units 1 and 2 – LAR 280 – Removal of Completed License Conditions and Change to the Ventilation Filter Testing Program (ADAMS Accession No. ML16194A440).
- 8.4 NextEra Energy Point Beach, LLC, letter (NRC 2016-0002) dated February 12, 2016, License Amendment Request 280, Removal of Completed License Conditions and Change to the Ventilation Filter Testing Program (ADAMS Accession No. ML16043A217).
- 8.5 NextEra Energy Point Beach, LLC, letter (NRC 2015-0075) dated January 15, 2016, License Amendment Request 279, Elimination of Technical Specification 3.7.14, Primary Auxiliary Building Ventilation (ADAMS Accession No. ML16015A112).
- 8.6 NextEra Energy Point Beach, LLC, letter (NRC 2013-0006) dated February 28, 2013, Completion of Activities to Support Entry Into the Period of Extended Operation (ADAMS Accession No. ML13063A290).
- 8.7 NextEra Energy Point Beach, LLC, letter (NRC 2011-0052) dated May 19, 2011, License Condition in Support of Alternate Source Term (ADAMS Accession No. ML111390406).
- 8.8 NRC letter to NextEra Energy Point Beach, LLC, dated May 3, 2011, Point Beach Nuclear Plant (PBNP), Units 1 and 2 – Issuance of License Amendments Regarding Extended Power Uprate (TAC Nos. ME1044 and ME 1045) (ADAMS Accession No. ML111170513).
- 8.9 Point Beach Unit 1 and 2 – Safety Evaluation re: Extended Power Uprate (TAC Nos. ME1044 and ME 1045) (ADAMS Accession No. ML110450159).
- 8.10 NRC letter to NextEra Energy Point Beach (PBNP), LLC, dated April 14, 2011, Units 1 and 2 - Issuance of License Amendments Regarding Use of Alternate Source Term (TAC No. ME0219 and ME0220) (ADAMS Accession No. ML110240054).
- 8.11 NRC letter to NextEra Energy Point Beach LLC, dated March 25, 2011, Point Beach Nuclear Plant (PBNP), Units 1 and 2 - Issuance of License Amendments RE: Auxiliary Feedwater System Modification (TAC No. ME1081 and ME1082) (ADAMS Accession No. ML110230016).
- 8.12 NextEra Energy Point Beach, LLC, letter (NRC 2010-0142) dated September 3, 2010, Completion of Activities to Support Entry into the Period of Extended Operation (ADAMS Accession No. ML102460327).
- 8.13 NextEra Energy Point Beach, LLC, letter (NRC 2010-0044) dated April 22, 2010, License Amendment Request 261, Extended Power Uprate – Implementation of New Auxiliary Feedwater System at Current Licensed Power Level (ADAMS Accession No. ML101130030).

- 8.14 NextEra Energy Point Beach, LLC, letter (NRC 2010-0040) dated April 15, 2010, License Amendment Request 261, Supplement 4, Extended Power Uprate (ADAMS Accession No. ML101050357).
- 8.15 FPL Energy Point Beach Nuclear Plant, letter (NRC 2009-0030) dated April 7, 2009, License Amendment Request 261, Extended Power Uprate (ADAMS Accession No. ML091250564).
- 8.16 FPL Energy Point Beach Nuclear Plant, letter (NRC 2008-0081) dated December 8, 2008, License Amendment Request 241, Alternative Source Term (ADAMS Accession No. ML083450683).
- 8.17 Nuclear Management Company, LLC, letter (NRC 2006-0067) dated August 15, 2006, Periodic Update of the Final Safety Analysis Report (ADAMS Accession No. ML062360522).
- 8.18 Nuclear Management Company, LLC, letter (NRC 2001-0079) dated November 26, 2001, Dockets 50-266 and 50-301, Implementation of Improved Technical Specification, Point Beach Nuclear Plant, Units 1 and 2 (ADAMS Accession No. ML020160187).
- 8.19 NRC letter to Nuclear Management Company, LLC, dated August 8, 2001, Point Beach Nuclear Power Plant, Units 1 and 2 – Issuance of Amendments RE: The Conversion to Improved Technical Specifications (TAC Nos. MA7186 and MA7187) (ADAMS Accession No. ML012250504).
- 8.20 NRC letter to Wisconsin Electric Power Company, dated May 11, 1979, regarding issuance of amendment nos. 38 and 43 to Facility Operating License Nos. DPR-24 and DPR-27 for the Point Beach Nuclear Power Plant, Unit Nos. 1 and 2 (ADAMS Accession No. ML021930043).
- 8.21 ASTM D3803 –1989, “Standard Test Method for Nuclear-Grade Activated Carbon”
- 8.22 NUREG 1431, Standard Technical Specifications Westinghouse Plants, Revision 4.0, April 2012. (ADAMS Accession No. ML12100A222)
- 8.23 Generic Letter 99-02, Laboratory Testing of Nuclear-Grade Activated Charcoal, June 3, 1999. (ADAMS Accession No. ML030970814).
- 8.24 Generic Letter 99-02, Laboratory Testing of Nuclear-Grade Activated Charcoal (Generic Letter 99-02 – Errata), August 23, 1999 (ADAMS Accession No. ML031110094).
- 8.25 ANSI/ASME N510-1980; Date of Issuance: May 31, 1980; “Testing of Nuclear Air-Cleaning Systems”.
- 8.26 Regulatory Guide 1.52, Revision 2, March 1978: “Design, Testing, and Maintenance Criteria for Post-Accident Engineered-Safety-Feature Atmosphere Cleanup System Air Filtration and Adsorption Units of Light-Water-Cooled Nuclear Power Plants”.

8.27 Regulatory Guide 1.52, Revision 3, June 2001; "Design, Inspection, and Testing Criteria for Air Filtration and Adsorption Units of Post-Accident Engineered-Safety-Feature Atmosphere Cleanup Systems in Light-Water-Cooled Nuclear power Plants."

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***Via Memorandum**

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