NRC FORM 195 (2-76)

U.S. NUCLEAR REGULATORY COMMISSION

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

DOCKET NUMBER: 50-220

FILE NUMBER:

TO: Mr. Edson G. Case

FROM: LeBoeuf, Lamb, Leiby & MacRae
Washington, D. C.
E. B. Thomas, Jr.

DATE OF DOCUMENT: 5/11/77

DATE RECEIVED: 5/11/77

☒ LETTER ☒ NOTORIZED
☒ ORIGINAL ☒ UNCLASSIFIED
☐ COPY

PROP

INPUT FORM

NUMBER OF COPIES RECEIVED: 3 SIGNED

DESCRIPTION

Ltr. trans the following:

DO NOT REMOVE

ACKNOWLEDGED (1-P)

PLANT NAME:

Nine Mile Point Unit No. 1

RJL

ENCLOSURE

Amdt. to OL/change to Appendix A tech specs....notorized 5/6/77.....concerns certain repairs at facility and concommitantly to allow a continuation of a slightly lower water level in the reactor vessel.....

(6-P)

40 encl.

FOR ACTION/INFORMATION

SAFETY		ENVIRO
ASSIGNED AD:		ASSIGNED AD:
✓ BRANCH CHIEF: (6)	LEAR	BRANCH CHIEF:
✓ PROJECT MANAGER:	NOWICKI	PROJECT MANAGER:
✓ LIC. ASST.:	PARRISH	LIC. ASST.:

INTERNAL DISTRIBUTION

✓ REG FILE	SYSTEMS SAFETY	PLANT SYSTEMS	SITE SAFETY
✓ NRC PDR	HEINEMAN	TEDESCO	ENVIRO ANALYSIS
✓ I & E	SCHROEDER	BENAROYA	DENTON & MULLER
✓ OELD		LAINAS	
✓ GOSSICK & STAFF	ENGINEERING	IPPOLITO	ENVIRO TECH.
MIPC	MACARY	KIRKWOOD	ERNST
CASE	BOSNAK		BALLARD
HANAUER	SIHWEIL	OPERATING REACTORS	YOUNGBLOOD
HARLESS	PAWLICKI	STELLO	
			SITE TECH.
PROJECT MANAGEMENT	REACTOR SAFETY	OPERATING TECH.	GAMMILL
BOYD	ROSS	✓ EISENHUT	STEPP
P. COLLINS	NOVAK	✓ SHAO	HULMAN
HOUSTON	ROSZTOCZY	✓ BAER	
PETERSON	CHECK	✓ BUTLER	SITE ANALYSIS
MELTZ		✓ GRIMES	VOLLMER
HELTEMES	AT & I		BUNCH
SKOVHOLT	SALTZMAN		✓ J. COLLINS
	RUTBERG		KREGER

EXTERNAL DISTRIBUTION

			CONTROL NUMBER
✓ LPDR: OSWEGO NY	NAT. LAB:	BROOKHAVEN NAT. LAB.	
✓ TIC:	REG V.IE	ULRIKSON (ORNL)	
✓ NSIC:	LA PDR		771320257
ASLB:	CONSULTANTS:		
✓ ACRS 11 CYS HOLDING/SENT AS CAT. B			

Ap 2 GD

LAW OFFICES OF
LEBOEUF, LAMB, LEIBY & MACRAE
1757 N STREET, N.W.
WASHINGTON, D. C. 20036
TELEPHONE 202 457-7500
CABLE ADDRESS
LEBWIN, WASHINGTON, D. C.
TELEX: 440274

May 11, 1977

Mr. Edson G. Case
Acting Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Re: Niagara Mohawk Power Corporation
Nine Mile Point Unit 1
Docket No. 50-220

Dear Mr. Case:

I enclose herewith three (3) originals and nineteen (19) copies of an Application for Amendment to Operating License No. DPR-63. Also enclosed are forty (40) copies of each of the two technical documents accompanying the Application.

The purpose of this request is to change a recently approved Technical Specification allowing the Licensee more time in which to make certain repairs at its facility and concommitantly to allow a continuation of a slightly lower water level in the reactor vessel.

Very truly yours,

LEBOEUF, LAMB, LEIBY & MACRAE
Attorneys for Niagara Mohawk Power Corporation

By E. B. Thomas, Jr.
Eugene B. Thomas, Jr.
Partner

Enclosures

7713020257

UNITED STATES OF AMERICA

NUCLEAR REGULATORY COMMISSION

In the Matter of	)	
	)	
	)	
NIAGARA MOHAWK POWER CORPORATION	)	Docket No. 50-220
(Nine Mile Point Nuclear Station	)	
Unit No. 1)	)	

APPLICATION FOR AMENDMENT

TO

OPERATING LICENSE

Pursuant to Section 50.90 of the regulations of the Nuclear Regulatory Commission, Niagara Mohawk Power Corporation, holder of Facility Operating License No. DPR-63, hereby requests that Sections 2.1.1 and 3.1.4 of the Technical Specifications and Bases set forth in Appendix A to that License be amended. This proposed change has been concurred with by the Site Operations Review Committee and Safety Review and Audit Board.

The proposed Technical Specification change is set forth in Attachment A to this application. Supporting Information, which demonstrates that the proposed change does not involve a significant hazards consideration, is set forth in Attachment B. The proposed change would not authorize any change in the types or any increase in the amounts of effluents or any change in the authorized power level of the facility.

WHEREFORE, Applicant respectfully requests that Appendix A to Facility Operating License No. DPR-63 be amended in the form attached hereto as Attachment A.

NIAGARA MOHAWK POWER CORPORATION

By _____
Gerald K. Rhode
Vice President-Engineering

Subscribed and sworn to before me this 6th day of May, 1977.

Phyllis D. Voytko
NOTARY PUBLIC

PHYLLIS D. VOYTKO
Notary Public in the State of New York
Qualified in Onon. Co. No. 34-9485535
My Commission Expires March 30, 1978

ATTACHMENT A

NIAGARA MOHAWK POWER CORPORATION

License No. DPR-63

Docket No. 50-220

Proposed Changes to Technical Specifications

Attached are revisions to Pages 6 and 53a of Appendix A to DPR-63.

SAFETY LIMIT

c. The neutron flux shall not exceed its scram setting for longer than 1.5 seconds as indicated by the process computer. When the process computer is out of service, a safety limit violation shall be assumed if the neutron flux exceeds the scram setting and control rod scram does not occur.

To ensure that the Safety Limit established in Specifications 2.1.1a and 2.1.1b is not exceeded, each required scram shall be initiated by its expected scram signal. The Safety Limit shall be assumed to be exceeded when scram is accomplished by a means other than the expected scram signal.

d. Whenever the reactor is in the shutdown condition with irradiated fuel in the reactor vessel, the water level shall not be more than 7 feet 11 inches (127.1 inches indicator scale) below minimum normal water level (Elevation 302'9"), except as specified in "e" below.

e. For the purpose of performing major maintenance (not to exceed 12 weeks in duration) on the reactor vessel, the reactor water level may be lowered 9' below the minimum normal water level (Elevation 302'9"). Whenever the reactor water level is to be lowered below the low-low-low level set point redundant instrumentation will be provided to monitor the reactor water level.

LIMITING SAFETY SYSTEM SETTING

d. The reactor water low level scram trip setting shall be no lower than -12 inches (53 inches indicator scale) relative to the minimum normal water level (302'9").

e. The reactor water low-low level setting for core spray initiation shall be no less than -5 feet (5 inches indicator scale) relative to the minimum normal water level (Elevation 302'9").

f. The flow biased APRM rod block trip settings shall be less than or equal to that shown in Figure 2.1.1.

LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT
h. For the purpose of performing major maintenance (not to exceed 12 weeks in duration) on the reactor vessel, the reactor water level may be lowered to 9' below the minimum normal water level (elevation 302'9"). Whenever the reactor water level is to be lowered below the low-low-low level set point redundant instrumentation will be provided to monitor the reactor water level and written procedures will be developed and followed whenever the reactor water level is lowered below the low-low level set point. The procedures will define the valves that will be used to lower the vessel water level. All other valves that have the potential of lowering the vessel water level will be identified by valve number in the procedures and these valves will be red tagged to preclude their operation during the major maintenance with the water level below the low-low level set point. During the period of major maintenance requiring lowering the water level to more than 7 feet 11 inches below minimum normal water level (127.1 inches indicator scale), either both Core Spray Systems must be operable or, if one Core Spray System is inoperable because of the maintenance, all of the redundant components of the other Core Spray System must be operable.	

Attachment B

NIAGARA MOHAWK POWER CORPORATION

License No. DPR-63

Docket No. 50-220

Supporting Information

Our submittal of March 9, 1977 requested a change which would allow lowering of the reactor vessel water level below the low-low-low level set point. This was required so that maintenance could be performed on the feedwater spargers and nozzles.

The maintenance work on the spargers and nozzles has been slower than anticipated. Also, additional work has been found to be necessary on the control rod drive hydraulic return line. Therefore, a change in the duration from 6 to 12 weeks is necessary.

Since the core decay heat generation is constantly decreasing, the consequences of a LOCA while in the shutdown condition will continue to be less severe.