NRC FORM 195 (2-76)

U.S. NUCLEAR REGULATORY COMMISSION

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TO: ~~XXXXXXXXX~~ Mr. E. Case

FROM: LeBoeuf, Lamb, Leiby & MacRae
Washington, D.C. 20036
E.B. Thomas, Jr.

DATE OF DOCUMENT: 7/22/77

DATE RECEIVED: 7/25/77

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DESCRIPTION: Ltr submitted on behalf of Niagara Mohawk Power Corp. trans the following: 1P

ENCLOSURE: Lic. No. DPR-63 Appl for Amend: tech specs proposed change concerning changes in management & organization & trans Attchmt's A & B..... Notarized 7-21-77.....

(3 Orig & 40 CC rec'd) 12P

DO NOT REMOVE

ACKNOWLEDGED

PLANT NAME: Nine Mile Pt. Unit 1

DHL 7-25-77

FOR ACTION/INFORMATION

SAFETY		ENVIRONMENTAL
ASSIGNED AD:	Lear (7)	ASSIGNED AD: V. MOORE (LTR)
✓ BRANCH CHIEF:		BRANCH CHIEF:
PROJECT MANAGER:		PROJECT MANAGER:
LICENSING ASSISTANT:		LICENSING ASSISTANT:
		B. HARLESS

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GOSSICK & STAFF	ENGINEERING	IPPOLITO	
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MIPC	BOSNAK		ERNST
CASE	SIHWELL	OPERATING REACTORS	BALLARD
BOYD	PAWLICKI	✓ STELLO	YOUNGBLOOD
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PROJECT MANAGEMENT	REACTOR SAFETY	✓ SHAO	~~SITE TECH.~~
SKOVHOLT	ROSS	✓ BAER	
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MELTZ	✓ CHECK		SITE ANALYSIS
HELTEMES			VOLLMER
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NRC FORM 195 (2-76)

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July 22, 1977

Mr. Edson G. Case
Acting Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Re: Niagara Mohawk Power Corporation
Nine Mile Point Nuclear Station,
Unit No. 1
Docket No. 50-220

Dear Mr. Case:

As counsel for Niagara Mohawk, I enclose the following:

(1) Three (3) originals and nineteen (19) copies of an application for amendment to operating license; and

(2) Forty (40) copies each of two (2) documents entitled Attachments A and B which set forth the requested change in the Technical Specifications along with its technical basis.

Very truly yours,

LeBOEUF, LAMB, LEIBY & MacRAE

By Eugene B. Thomas, Jr.
Eugene B. Thomas, Jr.

Enclosure

[illegible]

[illegible]

[illegible]

[illegible]

[illegible]

UNITED STATES OF AMERICA

NUCLEAR REGULATORY COMMISSION

Regulatory File Cy

In the Matter of	)	
	)	
	)	
NIAGARA MOHAWK POWER CORPORATION	)	Docket No. 50-220
(Nine Mile Point Nuclear Station	)	
Unit No. 1)	)	

APPLICATION FOR AMENDMENT

TO

OPERATING LICENSE

Pursuant to Section 50.90 of the regulations of the Nuclear Regulatory Commission, Niagara Mohawk Power Corporation, holder of Facility Operating License No. DPR-63, hereby requests that Sections 6.5.1.2, 6.5.1.6, 6.5.1.7, 6.5.1.8, 6.5.2.2, 6.5.2.8, 6.5.2.9, 6.5.2.10, 6.6.1, 6.7.1 and Figures 6.2-1 and 6.2-2 of the Technical Specifications and Bases set forth in Appendix A to that License be amended. The proposed changes have been concurred with by the Site Operations Review Committee and the Safety Review and Audit Board.

The proposed Technical Specification changes are set forth in Attachment A to this application. Supporting Information, which demonstrates that the proposed changes do not involve a significant hazards consideration, is set forth in Attachment B. The proposed changes would not authorize any change in the types or any increase in the amounts of effluents or any change in the authorized power level of the facility.

WHEREFORE, Applicant respectfully requests that Appendix A to Facility Operating License No. DPR-63 be amended in the form attached hereto as Attachment A.

NIAGARA MOHAWK POWER CORPORATION

By _____

Gerald K. Rhode
Vice President - Engineering

Subscribed and sworn to before me this 21st day of July, 1977.

Phyllis D. Voytko
NOTARY PUBLIC

PHYLLIS D. VOYTKO
Notary Public in the State of New York
Qualified in Onon. Co. No. 34-9485535
My Commission Expires March 30, 1978

ATTACHMENT A

Niagara Mohawk Power Corporation

License No. DPR-63
Docket No. 50-220

Proposed Changes to Technical Specifications

Attached are revised Pages 246, 247, 249, 250, 251, 252, 255 and 256.

Figure 6.2-1

Nine Mile Point Nuclear Station

Management Organization Chart

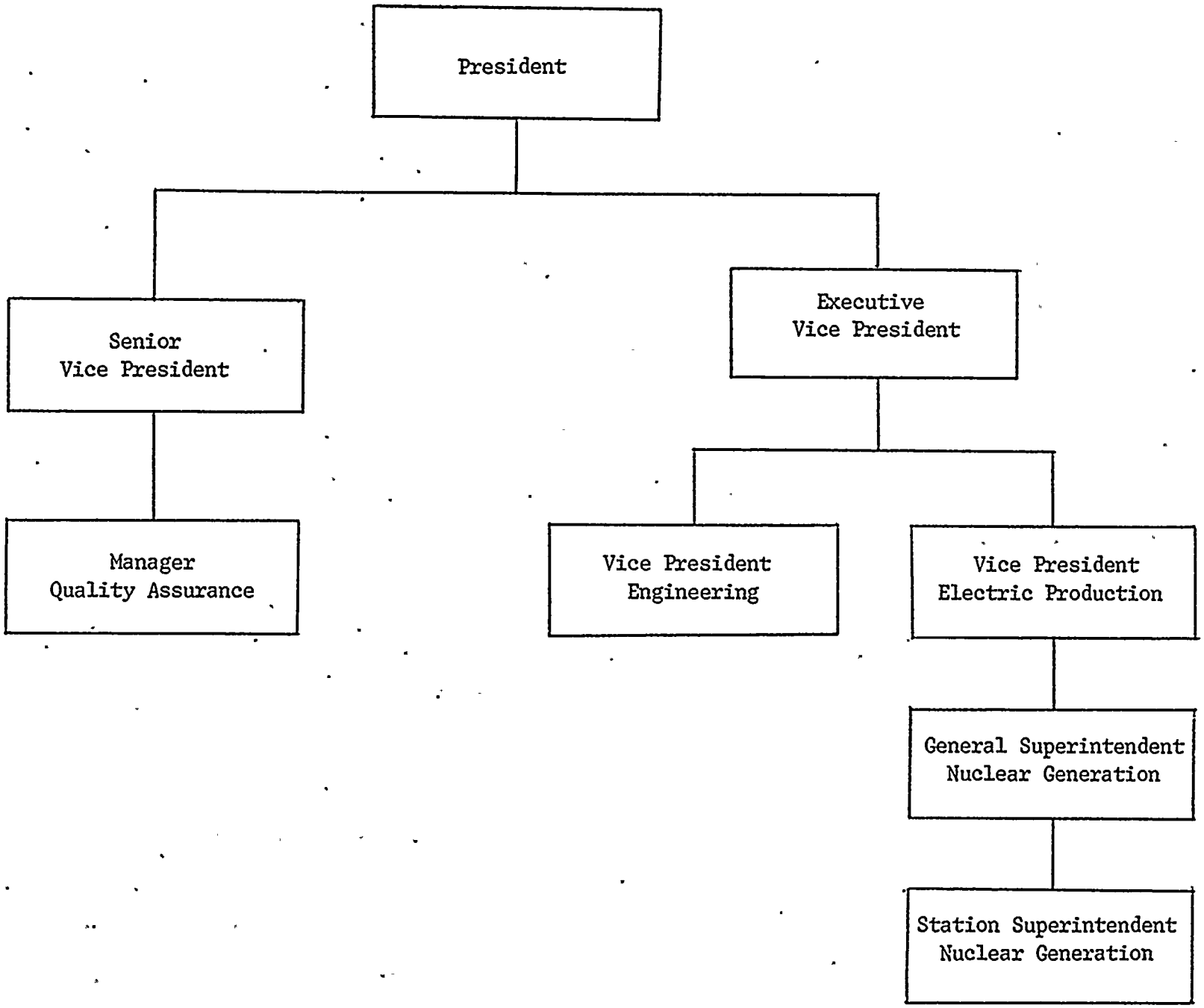
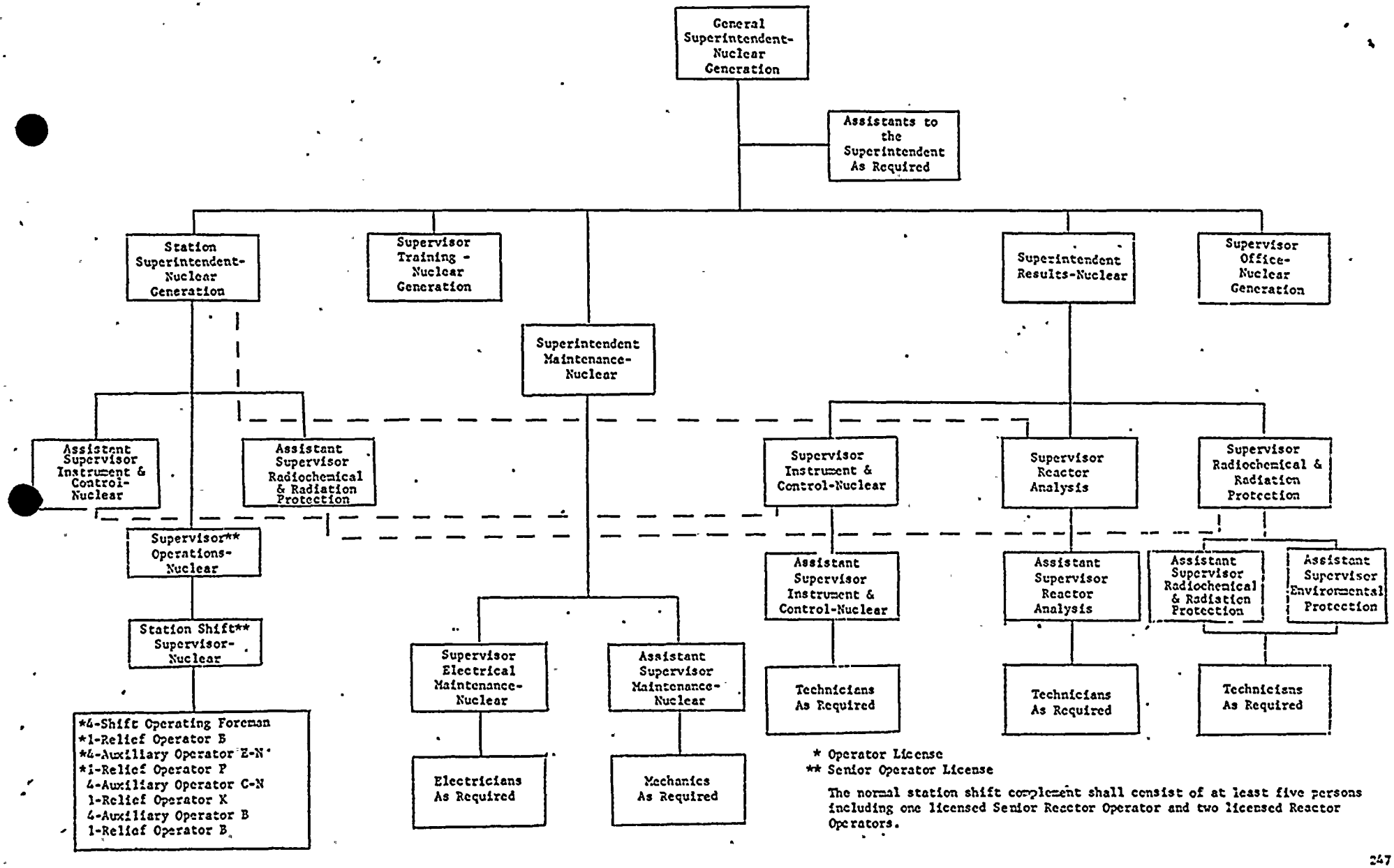


FIGURE 6.2-2

NINE MILE POINT NUCLEAR SITE

OPERATION ORGANIZATION



* Operator License

** Senior Operator License

The normal station shift complement shall consist of at least five persons including one licensed Senior Reactor Operator and two licensed Reactor Operators.

6.3 Facility Staff Qualifications

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions.

6.4 Training

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.5 Review and Audit

6.5.1 Site Operations Review Committee (SORC)

Function

6.5.1.1 The Site Operations Review Committee shall function to advise the General Superintendent Nuclear Generation on all matters related to nuclear safety.

Composition

6.5.1.2 The Site Operations Review Committee shall be composed of the:

Chairman:	General Superintendent Nuclear Generation
Member:	Station Superintendent-Nuclear Generation
Member:	Superintendent Results-Nuclear
Member:	Supervisor Reactor Analysis
Member:	Superintendent Maintenance-Nuclear
Member:	Supervisor Instrument & Control-Nuclear
Member:	Supervisor Radiochemical & Radiation Protection

Alternates

6.5.1.3 Alternate members shall be appointed in writing by the SORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in SORC activities at any one time.

Meeting Frequency

6.5.1.4 The SORC shall meet at least once per calendar month and as convened by the SORC Chairman.

Quorum

6.5.1.5 A quorum of the SORC shall consist of the Chairman and four members including alternates.

Responsibilities

6.5.1.6 The Site Operations Review Committee shall be responsible for:

a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the General Superintendent Nuclear Generation to affect nuclear safety.

b. Review of all proposed tests and experiments that affect nuclear safety.

c. Review of all proposed changes to the Technical Specifications.

d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.

e. Investigation of all violations of the Technical Specifications and shall prepare and forward a report covering evaluation and recommendations to prevent recurrence to the Vice President - Electric Production and to the Chairman of the Safety Review and Audit Board.

Responsibilities (cont'd)

f. Review of facility operations to detect potential safety hazards.

g. Performance of special reviews and investigations and reports thereon as requested by the Chairman of the Safety Review and Audit Board.

h. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Chairman of the Safety Review and Audit Board.

i. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the Safety Review and Audit Board.

Authority

6.5.1.7 The Site Operations Review Committee shall:

a. Recommend to the General Superintendent Nuclear Generation written approval or disapproval of items considered under 6.5.1.6 (a) through (d) above.

b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6 (a) through (e) above constitutes an unreviewed safety question.

c. Provide immediate written notification to the Vice President - Electric Production and the Safety Review and Audit Board of disagreement between the SORC and the General Superintendent Nuclear Generation; however, the General Superintendent Nuclear Generation shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

Records

6.5.1.8 The Site Operations Review Committee shall maintain written minutes of each meeting and copies shall be provided to the Vice President - Electric Production and Chairman of the Safety Review and Audit Board.

6.5.2 Safety Review and Audit Board (SRAB)

Function

6.5.2.1 The Safety Review and Audit Board shall function to provide independent review and audit of designated activities in the areas of:

a. nuclear power plant operations

b. nuclear engineering

c. chemistry and radiochemistry

d. metallurgy

e. instrumentation and control

f. radiological safety

g. mechanical and electrical engineering

h. quality assurance practices

i. (other appropriate fields associated with the unique characteristics of the nuclear power plant)

Composition

6.5.2.2 The Safety Review and Audit Board shall be composed of the:

Chairman:	Staff Engineer or Manager
Member:	General Superintendent Nuclear Generation
Member:	Staff Engineer - Nuclear
Member:	Staff Engineer - Mechanical or Electrical
Member:	Staff Engineer - Environmental
Member:	Consultant (See 6.5.2.4)

Audits (cont'd)

d. The performance of all activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per two years.

e. The Facility Emergency Plan and implementing procedures at least once per two years.

f. The Facility Security Plan and implementing procedures at least once per two years.

g. Any other area of facility operation considered appropriate by the SRAB, the Vice President - Electric Production or the Vice President - Engineering.

Authority

6.5.2.9 The SRAB shall report to and advise the Vice President - Electric Production and Vice President - Engineering on those areas of responsibility specified in Section 6.5.2.7 and 6.5.2.8.

Records

6.5.2.10 Records of SRAB activities shall be prepared, approved and distributed as indicated below:

a. Minutes of each SRAB meeting shall be prepared, approved and forwarded to the Vice President - Electric Production and Vice President - Engineering within 30 days following each meeting.

b. Reports of reviews encompassed by Section 6.5.2.7 e,f,g and h above, shall be prepared, approved and forwarded to the Vice President - Electric Production and Vice President - Engineering within 14 days following completion of the review.

c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Vice President - Electric Production and Vice President - Engineering and to the management positions responsible for the areas audited within 90 days after completion of the audit.

6.6 Reportable Occurrence Action

6.6.1 The following actions shall be taken in the event of a REPORTABLE OCCURRENCE:

a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.

b. Each Reportable Occurrence Report submitted to the Commission shall be reviewed by the SORC and submitted to the SRAB and the Vice President - Electric Production.

6.7 Safety Limit Violation

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.

b. The Safety Limit violation shall be reported to the Commission, the Vice President-Electric Production and to the SRAB immediately.

c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.

d. The Safety Limit Violation Report shall be submitted to the Commission, the SRAB and the Vice President - Electric Production within 10 days of the violation.

6.8 Procedures

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Sections 5.1 and 5.3 of ANSI N18.7-1972 and Appendix "A" of USAEC Regulatory Guide 1.33 except as provided in 6.8.2 and 6.8.3 below.

ATTACHMENT B

Niagara Mohawk Power Corporation

License No. DPR-63

Docket No. 50-220

Supporting Information

The proposed changes are being made to:

1) eliminate the James A. FitzPatrick Nuclear Power Plant from the Nine Mile Point Nuclear Site Organization (Figure 6.22)

2) to eliminate representation of the Power Authority of the State of New York from the Site Operations Review Committee and Safety Review and Audit Board and

3) to reflect changes in the Niagara Mohawk onsite and offsite organizations.

These proposed changes reflect the changes in operating responsibility for the James A. FitzPatrick Nuclear Power Plant. These changes also reflect current Niagara Mohawk onsite and offsite organization for facility management. It is expected that the new organization will improve facility management and provide for more effective reporting. The Site Operations Review Committee and the Safety Review and Audit Board will retain adequate membership to provide the expertise required by ANSI N18.7-1972.