

FROM: Niagara Mohawk Power Corp
Syracuse, N.Y.
M.H. Pratt

9-24-69

9-26-69

LTR. MEMO: REPORT: OTHER:

X

TO: Morris

ORIG.: CC: OTHER:

1

ACTION NECESSARY CONCURRENCE DATE ANSWERED:
NO ACTION NECESSARY COMMENT BY:

CLASSIF: U POST OFFICE REG. NO:

FILE CODE: 50-220

DESCRIPTION: (Must Be Unclassified)
Ltr re: our 9-2-69 ltr of understanding
that all action recommended by ACRS
have been carried out....& listing
status of items to be carried out...

REFERRED TO	DATE	RECEIVED BY	DATE
Schemel W/6 cys for action	9-26-69		
<u>DISTRIBUTION:</u>			
Regar file cy AEC PDR			
Compliance (2) OGC			
H. Price & Staff Skovholt			
Dube/Levina D. Thompson			
Boyd DTIE (Laughlin)			
NSIC (Buchanan)			
		3048	

ENCLOSURES:

REMARKS:

ACRS DIVISION
Do Not Remove

FEL

1942

...

...

...

...

...

...

...

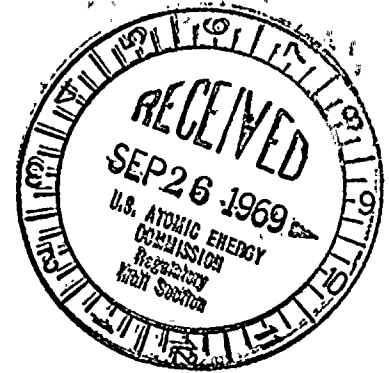
...

NIAGARA MOHAWK POWER CORPORATION

NIAGARA  MOHAWK

300 ERIE BOULEVARD WEST
SYRACUSE, N.Y. 13202

September 24, 1969



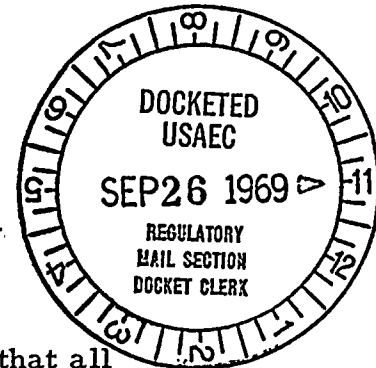
Dr. Peter A. Morris, Director
Division of Reactor Licensing
United States Atomic Energy Commission
Washington, D. C. 20545

Dear Dr. Morris:

Regulatory

File Cy. 7

Re: Docket No. 50-220



Your letter of September 2 states that you understand that all actions recommended by the ACRS have been carried out, where necessary, before power operation, or are being carried out as recommended. These actions as recommended by the ACRS have been carefully reviewed by our key personnel and we find that your understanding is correct. For your further specific information at this time the current status of these items is individually set forth below:

Corrosion Surveillance Program

Corrosion test specimens have been installed in three different locations on the reactor. Surveillance requirements for these specimens have been included in Section 4.2.2 of the Station's Technical Specifications.

Periodic Inspection of the High Pressure Coolant System

A program for periodic inspection of the reactor coolant system during the first five years of operation is described in the Station's Technical Specifications (Table 4.2.6). This program was modified shortly after the ACRS review to include plans for in-service inspection of the main steam lines beyond the second isolation valve. Throughout the first five-year period, inspection experience will be carefully evaluated and pertinent industry-wide development efforts such as those presently sponsored by the Edison Electric Institute will be closely followed to improve means for assuring continued pressure vessel integrity at Nine Mile Point. If review of the present inspection program in the light of the

2000

1000

1000

1000

1000

1000

1000

1000

1000

1000

continuous evaluation of operating experience over about five years of operation and the development work noted above should result in developing a significantly improved inspection technique, it will be implemented to the degree practical at Nine Mile Point and the program specifications will be revised accordingly at that time.

Primary System Leak Detection Studies

The recommendations of the ACRS have been incorporated in the Station's Technical Specifications (page 48). A report of progress in this area will be made to AEC Staff within a year after start of power operation.

Effects of Radiolysis

Studies of the potential for radiolytic decomposition of water following a design basis accident have been under way for some time and preliminary results are available for review with your Staff at any time. However, completion of our evaluation, including specific recommendations for the Nine Mile Point Nuclear Station, await the results of test data being obtained at the ORNL Test Loop and at the KRB reactor facility for the equilibrium value of hydrogen at BWR conditions and for the rate at which hydrogen would be evolved following an accident. Assuming these tests progress satisfactorily in the next few months, we anticipate submittal of our completed evaluations within a year.

Vibration Monitoring

Transducer type instrumentation is presently being studied for possible implementation at Nine Mile Point to detect vibration or the presence of loose parts in the reactor system. It is anticipated that a hardware package will be available for installation by the first refueling outage.

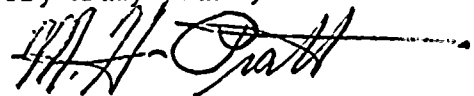
Safety Review and Audit Board

The current membership of the Safety Review and Audit Board has been increased to include three experts from outside the Niagara Mohawk organization.

Steam Line Radioactivity Monitors

Utilization and improvement of these monitors to provide early signs of gross fuel failures will be specifically studied after the Station begins power operation.

Very truly yours,



M. H. Pratt
Vice President

