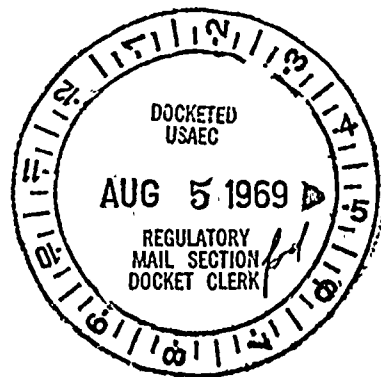
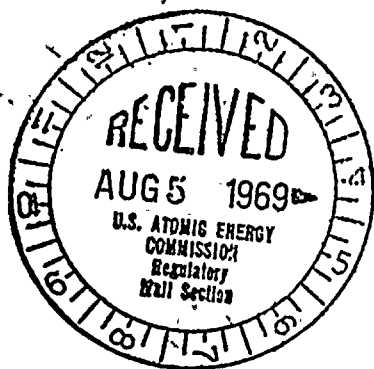


NIAGARA MOHAWK POWER CORPORATION

NIAGARA  MOHAWK

300 ERIE BOULEVARD WEST
SYRACUSE, N.Y. 13202



August 1, 1969

Regulatory

File Cy.

DOCKET NO. 50-220

Dr. Peter A. Morris
Director
Division of Reactor License
Atomic Energy Commission
Washington, D. C. 20545

Dear Dr. Morris:

During our meeting at Bethesda on July 17, 1969, we discussed and reviewed the matter of examination and assurance of adequate quality for the specific service application of certain components for the Nine Mile Point Nuclear Station. For your information relative to these matters we are proceeding as follows:

1. We will continue the process currently under way of verifying the non-destructive testing history for the installed piping, valves and pumps within the reactor coolant pressure boundary. We will document in sufficient detail to permit review and evaluation as to acceptable procedures and schedules on a case-by-case basis before operation in excess of 5 MW_{th} with the reactor head installed. This documentation will be made available for inspection by A. E. C. representatives.

For your information the 15 reactor safety valves have been removed from the vessel head and they, along with 8 spare valves for this vessel, have been returned to the manufacturer where they will be dismantled, tested and examined.

2. We will proceed with documentation of the non-destructive inspection methods used and the acceptance standards specified and applied for piping, valves and pumps of systems outside the reactor coolant pressure boundary. We plan to complete this information within one year from the start of fuel loading.

A report that describes the non-destructive inspection methods used and acceptance standards specified and applied for systems outside the reactor coolant pressure boundary will be submitted within one year from



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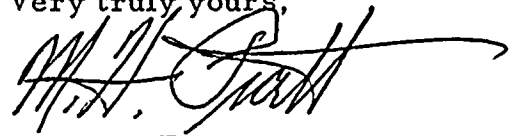
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the start of fuel loading. Results of the inspections and documentation of the inspection methods and acceptance standards will be available for inspection by A. E. C. representatives.

3. It shall be demonstrated before operation in excess of 5 MW_{th} with the reactor head installed that low pressure leakage from any one of the inboard or outboard Main Steam Line isolation valves does not exceed 5 per cent of the allowable operational leak rate described in the Technical Specifications.

4. We have taken steps to assure that both cold and hot vibration tests of selected reactor internal components similar to those planned for the Oyster Creek reactor can be performed at the Nine Mile Point reactor should this be necessary to meet the commitment described in Volume I, Page IV-87 of the FSAR.

Very truly yours,



M. H. Pratt
Vice President

MHP/jcl

