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(TEMPORARY FORM)

CONTROL NO: 43
FILE: _____

FROM: Niagara Mohawk Pwr Syracuse, NY RR Schneider		DATE OF DOC 12-30-75	DATE REC'D 1-5-76	LTR XXX	TWX	RPT	OTHER
TO: Mr Stolz		ORIG one signed	CC	OTHER	SENT NRC PDR <u>XX</u>		SENT LOCAL PDR <u>XX</u>
CLASS	UNCLASS XXXXXXXXXX	PROP INFO	INPUT	NO CYS REC'D 1	DOCKET NO: 50-410		

DESCRIPTION:
Ltr re their 10/3-75 ltr....furnishing info concerning mark II containment dynamic forcing functional report...which was forwarded to us by GE in late October of 1975....w/attachmss.....

ENCLOSURES:

PLANT NAME: Nine Mile Point #2

FOR ACTION/INFORMATION 1-7-76 ehf

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SALTZMAN
MELTZ

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MCDONALD
CHAPMAN
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| 1 - NSIC (EUCHANAN) | 1 - CONSULTANTS | 1 - G. ULRIKSON ORNL |
| 1 - ASLB | NEWMARK/BLUME/AGBABIAN | |
| 1 - Newton Anderson | | |
| 16 - ACRS HOLDING/SENT TO LA Smith | | |



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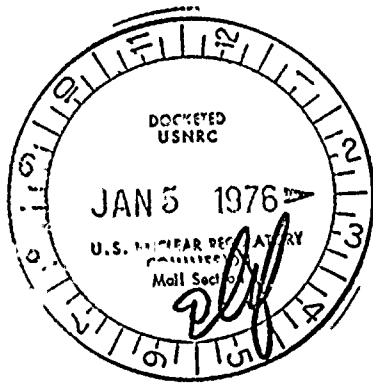
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NIAGARA MOHAWK POWER CORPORATION



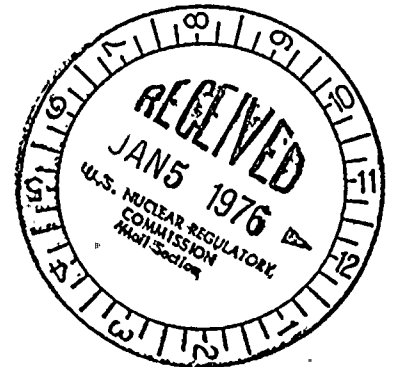
300 ERIE BOULEVARD WEST
SYRACUSE, N. Y. 13202



December 30, 1975

Director of Nuclear Reactor Regulation
Attention: Mr. J. F. Stolz, Chief
Branch 2-1
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Re: Nine Mile Point - Unit 2
Docket No. 50-410



Dear Mr. Stolz:

Our October 3, 1975, letter indicated that a report entitled "Mark II Containment Dynamic Forcing Functions Report" (DFFR) would be forwarded to you by the General Electric Company by late October, 1975. That report, consisting of NEDO-21061 and NEDE-21061P, was forwarded via an October 24, 1975 letter from Mr. I. F. Stuart, (General Electric), to Mr. R. S. Boyd, (Commission).

The report contains descriptions of phenomena to be used as a design basis for the Nine Mile Point Unit 2 containment. Specific load combinations for the Unit 2 containment are attached. This data supplements Table 5.2-1 of the DFFR. This completes the requirements for Part 1 of the Program and Schedule, enclosed with our June 20, 1975 letter.

The Nine Mile Point Unit 2 containment will be evaluated based on the DFFR and the attached table. The results of this evaluation will be submitted in June, 1976, in accordance with Part 3 of our Program and Schedule.



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Mr. J. F. Stolz

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December 30, 1975

In addition, Part 2 of our Program and Schedule indicated that information pertaining to pool temperature considerations would be provided by December, 1975. Due to additional supporting work required, however, we will be unable to meet that date. Portions of this information will be provided in June, 1976, with the balance included in the Final Safety Analysis Report.

Very truly yours,

NIAGARA MOHAWK POWER CORPORATION

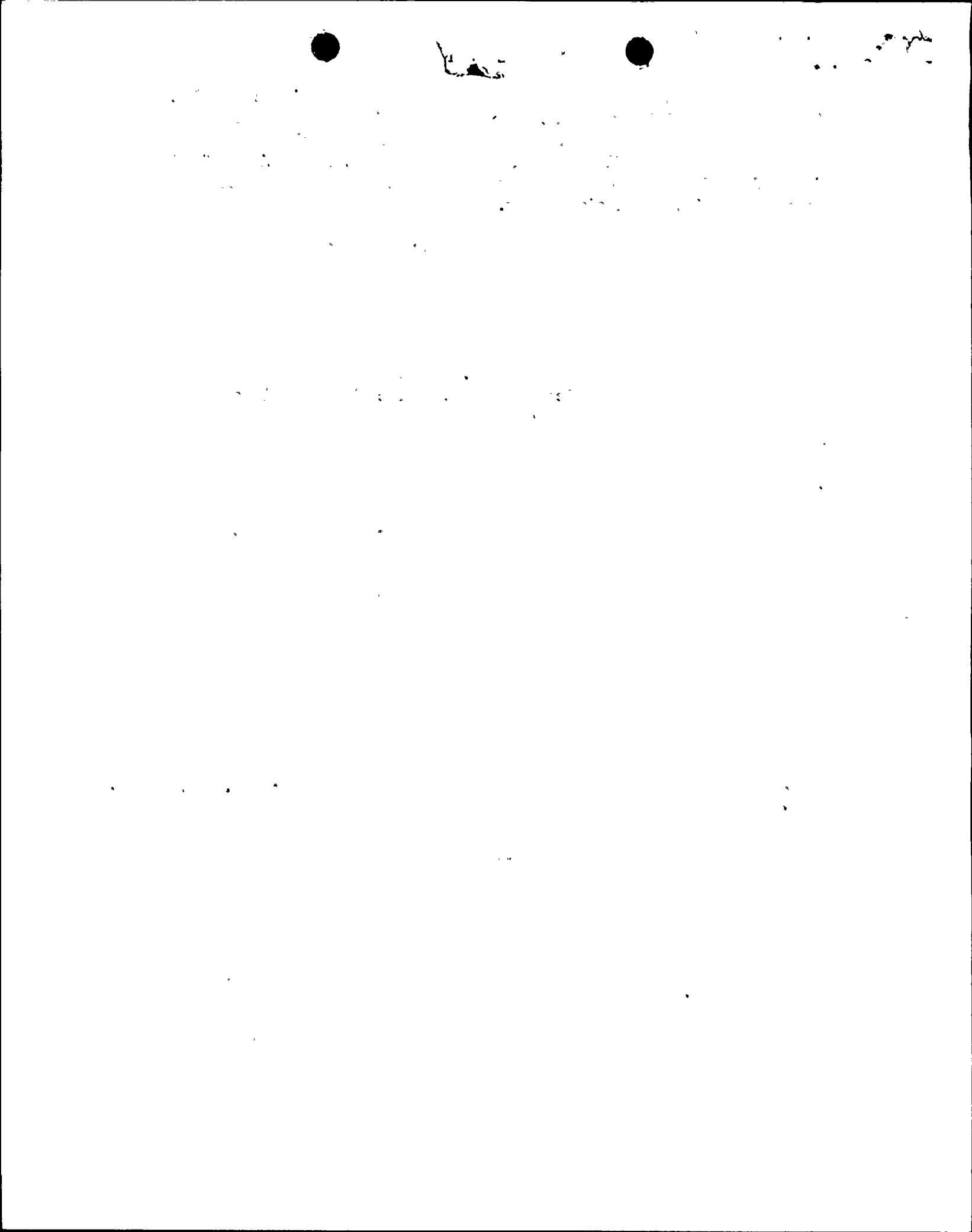


R. R. Schneider

Vice President - Electric Operations

plb

Attachment



NINE MILE POINT UNIT 2
TABLE 5.2-1

Load Combination for the Nine Mile Point 2
Containment Considering Hydrodynamic Loads

<u>EQN.</u>	<u>LOAD. COND.</u>	<u>D</u>	<u>L</u>	<u>F</u>	<u>P_O</u>	<u>T_O</u>	<u>R_O</u>	<u>E_O</u>	<u>E_{SS}</u>	<u>P_B</u>	<u>P_A</u>	<u>T_A</u>	<u>R_A</u>	<u>R_r</u>	<u>SRV</u>
1	Normal w/o Temp.	1.4	1.7	1.0	1.0	-	-	-	-	-	-	-	-	-	1.5
2	Normal w/ Temp.	1.0	1.3	1.0	1.0	1.0	1.0	-	-	-	-	-	-	-	1.3
3	Normal Sev. Env.	1.0	1.0	1.0	1.0	1.0	1.0	1.25	-	-	-	-	-	-	1.25
4	Abnormal	1.0	1.0	1.0	-	-	-	-	-	1.25	-	1.0	1.0	-	1.25
4a	Abnormal	1.0	1.0	1.0	-	-	-	-	-	-	1.25	1.0	1.0	-	1.0*
5	Abnormal Sev. Env.	1.0	1.0	1.0	-	-	-	1.1	-	1.1	-	1.0	1.0	-	1.1
5a	Abnormal Sev. Env.	1.0	1.0	1.0	-	-	-	1.1	-	-	1.1	1.1	1.1	-	1.0*
6	Normal Ext. Env.	1.0	1.0	1.0	1.0	1.0	1.0	-	1.0	-	-	-	-	-	1.0
7	Abnormal Ext. Env.	1.0	1.0	1.0	-	-	-	-	1.0	1.0	-	1.0	1.0	1.0	1.0
7a	Abnormal Ext. Env.	1.0	1.0	1.0	-	-	-	-	1.0	-	1.0	1.0	1.0	1.0	1.0*

Load Description

Sev. Env.	= Severe Environment	E _O	= Operating-Basis Earthquake
Ext. Env.	= Extreme Environment	E _{SS}	= Safe Shutdown Earthquake
D	= Dead Loads	P _B	= SBA or IBA Pressure Load
L	= Live Loads	P _A	= DBA (LOCA) Pressure Load
F	= Prestressing Loads	T _A	= Pipe Break Temperature Load
P _O	= Operating Pressure Loads	R _A	= Pipe Break Temperature Reaction Loads
T _O	= Operating Temperature Loads	R _r	= Reaction and jet forces associated with the pipe break
R _O	= Operating Pipe Reactions	SRV	= Safety/Relief Valve Loads

* In these combinations the SRV load is that associated with the discharge of one valve simultaneously with the LOCA to satisfy the single failure criterion.

