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TO: Mr B C Rusche

FROM: Niagara Mohawk Pwr Corp
Syracuse, NY
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DESCRIPTION

Ltr re our 1-14-77 ltr....furnishing schedule for response to questions concerning containment dynamic loads.....

1p

ENCLOSURE

ACKNOWLEDGED

PLANT NAME: Nine Mile Pt #2

DO NOT REMOVE

SAFETY

FOR ACTION/INFORMATION

ENVIRO 2-11-77 ehf

ASSIGNED AD:	<i>Vassallo</i>	ASSIGNED AD:	
BRANCH CHIEF:	<i>Varga</i>	BRANCH CHIEF:	
PROJECT MANAGER:	<i>Kane</i>	PROJECT MANAGER:	
LIC. ASST. :	<i>Service</i>	LIC. ASST. :	

INTERNAL DISTRIBUTION

REG-ELLE	SYSTEMS SAFETY	PLANT SYSTEMS	SITE SAFETY & SYSTEMS
NRC PDR	HEINEMAN	TEDESCO	ENVIRO ANALYSTS
I & E (2)	SCHROEDER	BENAROYA	DENTON & MULLER
OELD		LAINAS	
GOSSICK & STAFF	ENGINEERING	IPPOLITO	ENVIRO TECH.
MIPC	MACARRY	KIRKWOOD	ERNST
CASE	KNIGHT		BALLARD
HANAUER	SIHWEIL	OPERATING REACTORS	SPANGLER
HARLESS	PAWLICKI	STELLO	
			SITE TECH.
PROJECT MANAGEMENT	REACTOR SAFETY	OPERATING TECH.	GAMMILL
BOYD	ROSS	EISENHUT	STAPP
P. COLLINS	NOVAK	SHAO	HULMAN
HOUSTON	ROSZTOCZY	BAER	
PETERSON	CHECK	BUTLER	SITE ANALYSIS
MELTZ		GRIMES	VOLLMER
HELTEMES	AT & I		BUNCH
SKOVHOLT	SALTZMAN		J. COLLINS
	RUTBERG		KREGER

EXTERNAL DISTRIBUTION

CONTROL NUMBER

LPDR: <i>Oswego, NY</i>	NAT. LAB:	BROOKHAVEN NAT. LAB.
TIC:	REG V. IE	ULRIKSON (ORNL)
NSIC:	LA PDR	
ASLB:	CONSULTANTS:	
ACRS CYS HOLDING/SENT		

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EXCEPT BY AUTHORITY OF THE
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Regulatory Docket File

NIAGARA MOHAWK POWER CORPORATION

NIAGARA  MOHAWK

300 ERIE BOULEVARD, WEST
SYRACUSE, N. Y. 13202

February 7, 1977

Director of Nuclear Reactor Regulation
Attn: Mr. Benard C. Rusche, Director
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555



Re: Nine Mile Point Unit 2
Docket No. 50-410

Dear Mr. Rusche:

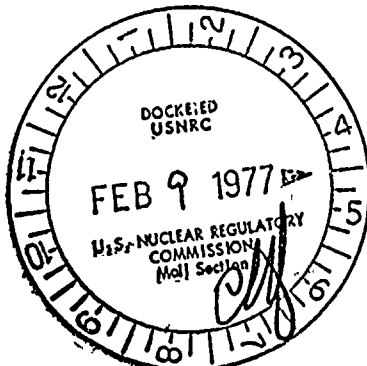
Mr. Varga's January 14, 1977 letter requested a schedule for response to certain questions regarding the Nine Mile Point Unit 2 Containment dynamic loads. We have reviewed this matter with our Architect Engineer and Nuclear Steam Supply Vendor. Based on this review, we propose to reschedule Revision 1 of the "Plant Design Assessment for Safety Relief Valve and Loss of Coolant Accident Loads."

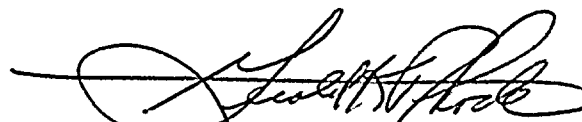
Revision 1 was previously scheduled to be submitted in June, 1977, as indicated in our September 1, 1976 letter to Mr. Vassallo. Revision 1 is now scheduled to be submitted in November, 1977, and will incorporate the information requested in the January 14, 1977 letter and Revision 3 to the "Mark II Containment Dynamic Forcing Functions Report."

Please note that requests 020.62(1) and 020.63(2) are addressed in Section 10 of the Unit 2 "Plant Design Assessment for Safety Relief Valve and Loss of Coolant Accident Loads" Report which was submitted June 30, 1976.

Very truly yours,

NIAGARA MOHAWK POWER CORPORATION




GERALD K. RHODE
Vice President - Engineering

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NLR/sz

