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DOCTYPE: LETTER NOTARIZED: NO
SUBJECT:
FORWARDING REPT OF OPERATING STATISTICS AND SHUTDOWN EXPERIENCE FOR JANUARY 1978 AND A NARRATIVE REPT OF OPERATING EXPERIENCE FOR THE MONTH.

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PLANT NAME: NINE MILE PT - UNIT 1

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MONTHLY OPERATING REPORT FOR GRAY BOOK PREPARATION.
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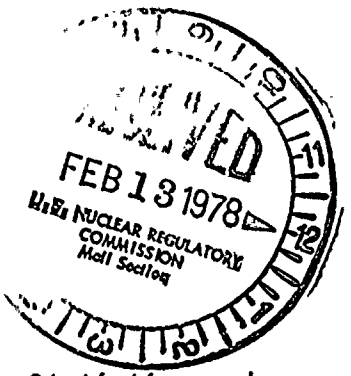
NIAGARA MOHAWK POWER CORPORATION/300 ERIE BOULEVARD WEST, SYRACUSE, N.Y. 13202/TELEPHONE (315) 474-1511

February 7, 1978

Director
Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555



RE: Docket No. 50-220
DPR-63

Gentlemen:

Submitted herewith is the Report of Operating Statistics and Shutdown Experience for January 1978 for the Nine Mile Point Nuclear Station Unit #1. Also included is a narrative report of operating experience for the month.

Very truly yours,

R.R. Schneider
Vice President -
Nuclear Generation

mtm
Enc.

xc: Director Office of I&E (10 copies)
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A003
1/1

Jan. 22 Continued with reactor startup. At 0400, a steam leak was discovered on the steam seal regulator blocking valve. Startup continued to rated pressure and scram test of the control rods was performed. Unit was then brought to a hot standby condition to repair the steam seal regulator blocking valve. At 1030 repairs were completed and rod withdrawals started. The mode switch was placed in run at 1338 and the Turbo-generator synchronized to the system at 1510. Continued rod withdrawals to approx. 200 mwe. At 1813, #13 feedwater pump was placed in service. At 1823, high vibration was experienced in the feedwater system. At 1830, #13 feedwater pump was removed from service.

Jan. 23 Investigation showed a sheared pin on the control cylinder snubber of the south flow control valve of #13 feedwater pump. Repairs were completed at 0500 and #13 feedwater pump was placed in service at 0620. At 0630, vibration was again experienced in the feedwater system and #13 feedwater pump was removed from service at 0650. Load was maintained at 240 mwe while maintenance inspected the north flow control valve. It was found that the disc had separated from the stem. Repairs continued for the remainder of the day.

Jan. 24 Repairs to #13 feedwater pump's north flow control valve continued during the day. Unit load at approx. 238 mwe and 775 mwt.

Jan. 25 Completed repairs to the north flow control valve. #13 feedwater pump placed in service at 0615. Continued to increase load during the day.

Jan. 26 Unit load at 510 mwe and 1522 mwt. Continued load increases within PCIOMR limits.

Jan. 27 Continued load increases to 590 mwe. Maintaining load due to problems with condensate demineralizers dP. At 1348 increased load to 603 mwe.

Jan. 28 Reduced load to 490 mwe to change a condensate demineralizer. At 0500, load increased to 585 mwe. Continued with load increases to 602 mwe.

Jan. 29 Reduced load to 500 mwe at 2241 to change a condensate demineralizer.

Jan. 30 At 0144, started to increase load to 580 mwe. Continued to increase load to 614 mwe.

Jan. 31 Continued steady state operation at approx. 615 mwe.

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION UNIT #1

NARRATIVE OF OPERATING EXPERIENCE
JANUARY 1978

Jan. 1 — At 0000 after a control rod pattern change, unit load was 1375 mwt, 445 mwe. Unit load increased during the remainder of the day as PCIOMR limits allowed.

Jan. 2 — Continued to increase load within PCIOMR limits.

Jan. 3 — Continued to increase load via recirculation flow increases. At 1450 unit load was 625 mwe. At 2202 reduced load to approx. 485 mwe due to icing up of the lake water intake. At 2305 increased load to 525 mwe following the flow reversal.

Jan. 4 — Continued load increase to 560 mwe. At 0505 reduced load to return circulating water flow to normal. At 0535, the unit was returned to normal flow and load increased during the remainder of the day.

Jan. 5 — Continued to increase load. At 1500 obtained maximum recirculation flow, unit at 625 mwe.

Jan. 6 - Jan. 12
Continued steady state operation at approx. 625 mwe, 1840 mwt.

Jan. 13 — Load was reduced at 2205 to adjust rod pattern. Unit load approx. 515 mwe, 1515 mwt.

Jan. 14 — Had problems with the TIP machines held load at approx. 1569 mwt. At 0610 started to increase load as PCIOMR limits allowed.

Jan. 15 — Unit load at approx. 1785 mwt, 608 mwe. Continued load increases during the remainder of the day.

Jan. 16 - Jan. 19
Continued steady state operation at 1830 mwt and 624 mwe.

Jan. 20 — Unit load was 1828 mwt and 624 mwe. At 1000 reactor level control became erratic. At 1035 started to reduce load due to a malfunction of #13 feedwater pump flow control valves. At 1058, the reactor was manually scrammed due to continuing rapid reactor water level oscellations.

Investigation showed a broken stem on #13 feedwater pump's south flow control valve. (Also, some rods showed long scram response times.)

Jan. 21 — The rubber diaphrams on the scram pilot valves for 11 CRD's were replaced.

OPERATING DATA REPORT

DOCKET NO. 50-220
DATE 2/8/78
COMPLETED BY T.J. Perkins
TELEPHONE (315)343-2110 ext.1312

OPERATING STATUS

1. Unit Name: Nine Mile Point Unit #1
2. Reporting Period: 01/01/78 - 01/31/78
3. Licensed Thermal Power (MWt): 1850
4. Nameplate Rating (Gross MWe): 640
5. Design Electrical Rating (Net MWe): 620
6. Maximum Dependable Capacity (Gross MWe): 630
7. Maximum Dependable Capacity (Net MWe): 610
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.0	744.0	72,336.0
12. Number Of Hours Reactor Was Critical	709.7	709.7	51,778.6
13. Reactor Reserve Shutdown Hours	0	0	1,204.0
14. Hours Generator On-Line	691.8	691.8	49,382.0
15. Unit Reserve Shutdown Hours	0	0	1,204.0
16. Gross Thermal Energy Generated (MWH)	1,158,620.0	1,158,620.0	79,612,480.0
17. Gross Electrical Energy Generated (MWH)	392,900.0	392,900.0	26,174,612.0
18. Net Electrical Energy Generated (MWH)	379,622.0	379,622.0	25,356,545.0
19. Unit Service Factor	93.0	93.0	68.0
20. Unit Availability Factor	93.0	93.0	70.0
21. Unit Capacity Factor (Using MDC Net)	84.0	84.0	58.0
22. Unit Capacity Factor (Using DER Net)	82.0	82.0	57.0
23. Unit Forced Outage Rate	7.0	7.0	11.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
May 28, 1978 Snubber Inspection

25. If Shut Down At End Of Report Period, Estimated Date of Startup:
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH JANUARY 1978

DOCKET NO. 50-220
UNIT NAME Nine Mile Point Unit #1
DATE 2/7/78
COMPLETED BY T.J. Perkins
TELEPHONE (315)343-2110 ext.1312

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
0	1/3/78	F	1	H	1				Intake Tunnel Ice Reverse Flow - 485 MWe
0	1/4/78	S	.45	H	1				Intake To Normal - 550 MWe
1		F	52.2	A	2				#13 Feedwater Pump F.C.V. Failure.
	1/23/78	F	35.7	A	1				Held Load At ≃ 240 MWe To Repair #13 Feedwater Pump FCV.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

INSTRUCTIONS

This report should describe all plant shutdowns during the report period. In addition, it should be the source of explanation of significant dips in average power levels. Each significant reduction in power level (greater than 20% reduction in average daily power level for the preceding 24 hours) should be noted, even though the unit may not have been shut down completely[1]. For such reductions in power level, the duration should be listed as zero, the method of reduction should be listed as 4 (Other), and the Cause and Corrective Action to Prevent Recurrence column should explain. The Cause and Corrective Action to Prevent Recurrence column should be used to provide any needed explanation to fully describe the circumstances of the outage or power reduction.

NUMBER. This column should indicate the sequential number assigned to each shutdown or significant reduction in power for that calendar year. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported. Until a unit has achieved its first power generation, no number should be assigned to each entry.

DATE. This column should indicate the date of the start of each shutdown or significant power reduction. Report as year, month, and day. August 14, 1977 would be reported as 770814. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported.

TYPE. Use "F" or "S" to indicate either "Forced" or "Scheduled," respectively, for each shutdown or significant power reduction. Forced shutdowns include those required to be initiated by no later than the weekend following discovery of an off-normal condition. It is recognized that some judgment is required in categorizing shutdowns in this way. In general, a forced shutdown is one that would not have been completed in the absence of the condition for which corrective action was taken.

DURATION. Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

REASON. Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER. Categorize by number designation in accordance with the table appearing on the report form. If category 4 must be used, supply brief comments.

LICENSEE EVENT REPORT #. Reference the applicable reportable occurrence pertaining to the outage or power reduction. Enter the first four parts (event year, sequential report number, occurrence code and report type) of the five part designation as described in Item 17 of Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). This information may not be immediately evident for all such shutdowns, of course; since further investigation may be required to ascertain whether or not a reportable occurrence was involved.) If the outage or power reduction will not result in a reportable occurrence, the positive indication of this lack of correlation should be noted as not applicable (N/A).

SYSTEM CODE. The system in which the outage or power reduction originated should be noted by the two digit code of Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161).

Systems that do not fit any existing code should be designated XX. The code ZZ should be used for those events where a system is not applicable.

COMPONENT CODE. Select the most appropriate component from Exhibit I - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161), using the following critieria:

A. If a component failed, use the component directly involved.

B. If not a component failure, use the related component: e.g., wrong valve operated through error; list valve as component.

C. If a chain of failures occurs, the first component to malfunction should be listed. The sequence of events, including the other components which fail, should be described under the Cause and Corrective Action to Prevent Recurrence column.

Components that do not fit any existing code should be designated XXXXXX. The code ZZZZZZ should be used for events where a component designation is not applicable.

CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE. Use the column in a narrative fashion to amplify or explain the circumstances of the shutdown or power reduction. The column should include the specific cause for each shutdown or significant power reduction and the immediate and contemplated long term corrective action taken, if appropriate. This column should also be used for a description of the major safety-related corrective maintenance performed during the outage or power reduction including an identification of the critical path activity and a report of any single release of radioactivity or single radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

[1]Note that this differs from the Edison Electric Institute (EEI) definitions of "Forced Partial Outage" and "Scheduled Partial Outage." For these terms, EEI uses a change of 30 MW as the break point. For larger power reactors, 30 MW is too small a change to warrant explanation.

NINE MILE POINT NUCLEAR STATION
NIAGARA MOHAWK POWER CORPORATION

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-220
UNIT Nine Mile Point #1
DATE 2/8/78
COMPLETED BY T.J. Perkins
TELEPHONE (315)343-2110 ext.1312

MONTH JANUARY

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	471	17	611
2	553	18	610
3	584	19	610
4	585	20	607
5	608	21	-
6	608	22	192
7	609	23	215
8	609	24	218
9	610	25	346
10	609	26	537
11	609	27	579
12	608	28	573
13	602	29	577
14	551	30	582
15	598	31	598
16	609		

REMARKS: