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SEP 17 1975

Docket No. 50-220

Niagara Mohawk Power Corporation  
 ATTN: Mr. Gerald K. Rhode  
 Vice President - Engineering  
 300 Erie Boulevard West  
 Syracuse, New York 13202

Gentlemen:

A preliminary review of your application for license amendment dated August 18, 1975 indicated that we will require additional information. The requested information is contained in the enclosure.

In order that we may complete our review on a timely basis and accommodate your earliest anticipated Cycle 4 startup date of November 1, 1975, please submit your written response to the items contained in the enclosure within 14 days following receipt of this letter.

If you have any questions regarding the requested information, we would be pleased to meet with you.

Sincerely,

*151*

George Lear, Chief  
 Operating Reactors Branch #3  
 Division of Reactor Licensing

Enclosure:  
 Requested Information on Core  
 Cycle 4 Analysis for Nine Mile  
 Point Unit 1

cc: See next page

*appd  
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*W*

OFFICE >	ORB#3	TR <i>DF</i>	TR <i>RW</i>	ORB#3	
SURNAME >	JGuibert <i>kmf</i>	DFieno	RWoods	Glear <i>GL</i>	
DATE >	9/17/75	9/17/75	9/17/75	9/17/75	

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Niagara Mohawk Power Corporation - 2 -

SEP 17 1975

cc:

Arvin E. Upton, Esquire  
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Dr. William Seymour, Staff Coordinator  
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New York State Department of Commerce  
112 State Street  
Albany, New York 12207

Oswego City Library  
120 E. Second Street  
Oswego, New York 13126

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ENCLOSURE  
REQUEST FOR ADDITIONAL INFORMATION  
CORE CYCLE 4 ANALYSIS FOR NINE MILE POINT UNIT 1

1. Provide the value for the delayed neutron fraction applicable to the beginning and the end of Cycle 4.
2. Provide the design conservatism factors for scram reactivity, Doppler coefficient, and void coefficient (include both values used in Table 6-1).
3. Provide a statement verifying that the Neutron Effective Void model was used in the calculation of the void coefficient used for Cycle 4.
4. Provide the control rod location used in the development of Figures 6-12 and 6-13 of NEDO-20772, "GE-BWR Reload-5 Licensing Submittal for NMP-1 Nuclear Power Station, Unit 1", for the Rod Withdrawal Error transient analysis.
5. Provide the time in Cycle 4 to which the curves in Figures 6-2 through 6-5, NEDO-20772, correspond.
6. The analysis of the fuel loading error accident (page 6-7 of NEDO-20772) shows that a localized MCPR of 1.05 would occur in a misplaced bundle. Provide an explanation of the consequences of operating with a MCPR less than the Safety Limit of 1.06. Include the radiological consequences, if any.
7. The GETAB transient analysis initial condition parameters, Table 4-4 of NEDO-20772, indicate that the initial condition MCPR values used in the transient analyses are lower than the operating limits derived from the analyses. Provide a discussion of how this affects the conservatism of the analyses. Include an explanation of the relationship between initial MCPR and  $\Delta$ MCPR.
8. Provide an explanation of the difference in the GETAB transient analysis initial condition parameters as presented in Table 4 of your June 30, 1975 submittal and Table 4-4 of NEDO-20772.
9. For the Rod Withdrawal Error transient analysis, provide a curve of APRM channel reading (% of initial level) versus control rod position (ft. withdrawn) for the case where no LPRM's are bypassed.
10. Provide an explanation for the difference in the void fraction values in Tables 5-1 and 6-1, NEDO-20772.

