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TO:
MR B C RUSCHE

FROM: NIAGARA MOHAWK POWER CORP
SYRACUSE, NY
G K RHODE

DATE OF DOCUMENT
3-5-76
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3-8-76

LETTER
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DESCRIPTION

LTR RE OUR 2-27-76 LTR.....FURN CLARIFACATION ON CERTAIN ITEMS CONTAINED IN REFERENCE..... FURTHER ADVISING THAT THE CALCULATED LOAD RATATIONS ON THE TORUS SUPPORT COMPONENTS WERE BELOW 1.0.....

PLANT NAME: Nine Mile Pt # 1.

ENCLOSURE

SAFETY		FOR ACTION/INFORMATION		ENVIRO	
ASSIGNED AD :		ASSIGNED AD :			
<input checked="" type="checkbox"/> BRANCH CHIEF : (6)	LEAR	<input type="checkbox"/> BRANCH CHIEF :			
PROJECT MANAGER:		PROJECT MANAGER :			
<input checked="" type="checkbox"/> LIC. ASST. :	PARRISH	LIC. ASST. :			

INTERNAL DISTRIBUTION				
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<input checked="" type="checkbox"/> NRC PDR	HEINEMAN	TEDESCO	ERNST	
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<input checked="" type="checkbox"/> OELD		LAINAS	SPANGLER	
GOSSICK & STAFF	ENGINEERING	IPPOLITO		
MIPC	MACCARY		SITE TECH	
CASE	KNIGHT	OPERATING REACTORS	GAMMILL	
HANAUER	SIHWEIL	STELLO	STEPP	
HARLESS	PAWLICKI		HULMAN	
		OPERATING TECH		
PROJECT MANAGEMENT	REACTOR SAFETY	<input checked="" type="checkbox"/> EISENHUT	SITE ANALYSIS	
BOYD	ROSS	<input checked="" type="checkbox"/> SHAO	VOLLMER	
P. COLLINS	NOVAK	<input checked="" type="checkbox"/> BAER	BUNCH	
HOUSTON	ROSZTOCZY	<input checked="" type="checkbox"/> SCHWENCER	<input checked="" type="checkbox"/> J. COLLINS	
PETERSON	CHECK	<input checked="" type="checkbox"/> GRIMES	KREGER	
MELTZ				
HELTEMES	AT & I	SITE SAFETY & ENVIRO		
SKOVHOLT	SALTZMAN	ANALYSIS		
	RUTBERG	DENTON & MULLER		

EXTERNAL DISTRIBUTION			CONTROL NUMBER
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<input checked="" type="checkbox"/> NSIC	LA PDR		
ASLB	CONSULTANTS		
<input checked="" type="checkbox"/> ACRS 16 HOLDING/SENT			



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NIAGARA MOHAWK POWER CORPORATION

NIAGARA  MOHAWK

300 ERIE BOULEVARD, WEST
SYRACUSE, N. Y. 13202



March 5, 1976



NOTED File

Office of Nuclear Reactor Regulation
Attn: Mr. Benard C. Rusche, Director
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

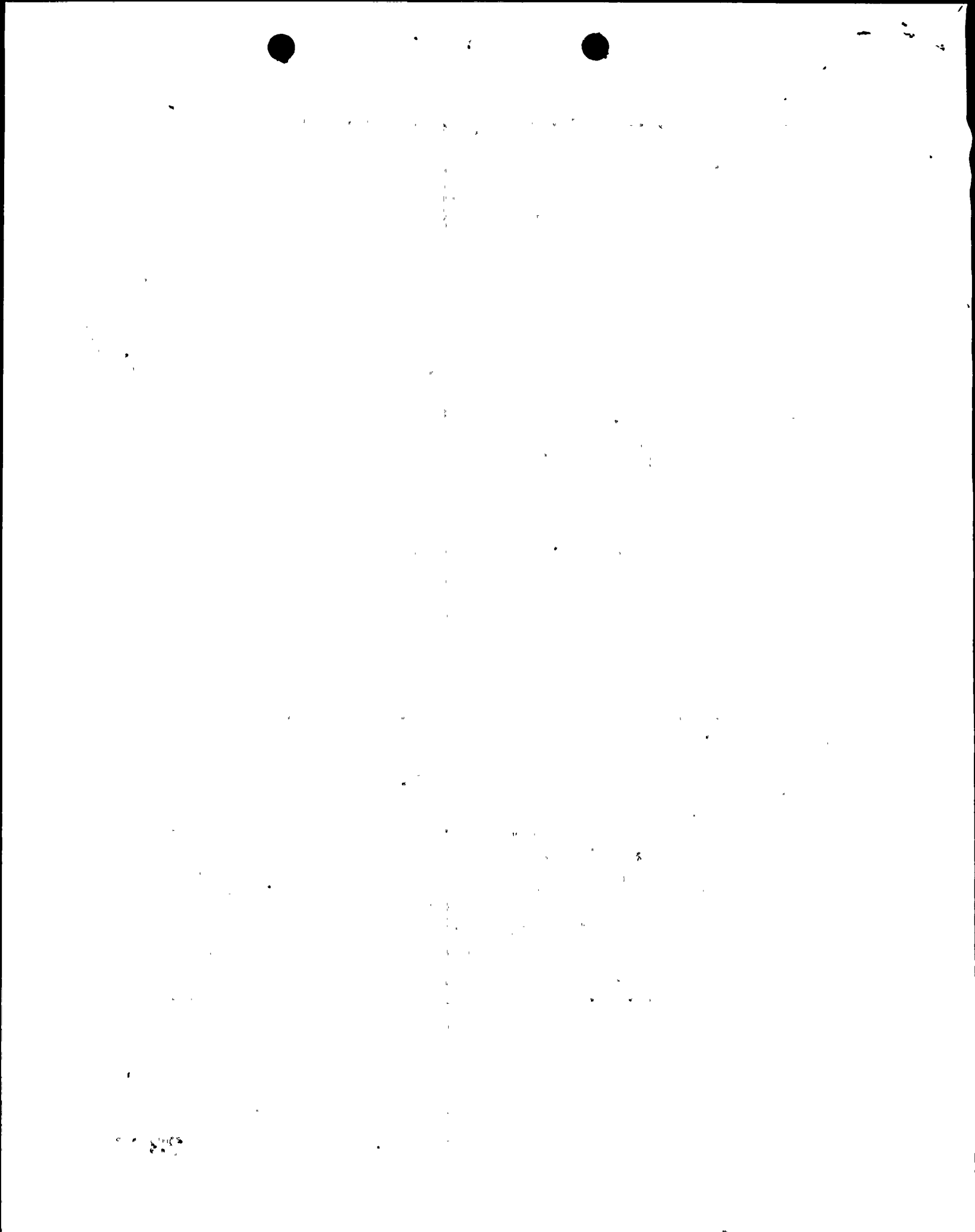
Re: Nine Mile Point Unit 1
Docket No. 50-220
DPR-63

Dear Mr. Rusche:

This letter is to acknowledge receipt of your February 27, 1976 communication and to provide clarification on certain items contained therein.

My February 27, 1976 letter to Mr. George Lear confirmed our commitment to implement a drywell pressure increase of one pound per square inch over the suppression chamber pressure at Nine Mile Point Unit 1 and we subsequently advised your staff on March 1 that this operating mode had indeed been achieved.

It should be noted that the calculated load ratios on the torus support components for Nine Mile Point Unit 1 were considerably below the 1.0 referenced in your letter, even before the drywell pressure increase. This new pressure adjustment should improve the safety margin at Nine Mile Point Unit 1 to at least a factor of two for the welds at the top of the torus support columns. Therefore, there are no plans for structural modifications to the containment system at this facility.



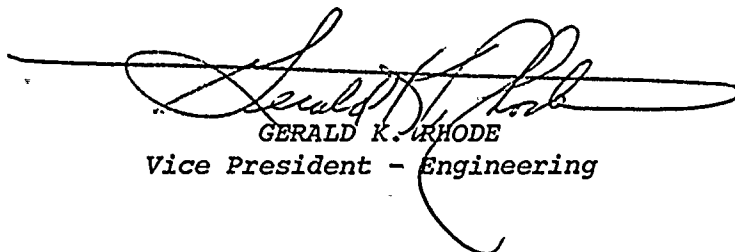
Mr. Benard C. Rusche
U. S. Nuclear Regulatory Commission

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It is our understanding that your directive of February 27, 1976 was intended to be complementary to the current Technical Specifications for Nine Mile Point Unit 1. Specifically, Specifications 3.3.1.b and 4.3.6.a provide for short operational periods during containment inerting after a shutdown and for monthly vacuum relief valve operability tests, respectively, when maintenance of the differential pressure would not be possible: The Technical Specifications will continue to be followed in these cases.

Very truly yours,

NIAGARA MOHAWK POWER CORPORATION



GERALD K. RHODE
Vice President - Engineering

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