



UNITED STATES
ATOMIC ENERGY COMMISSION
DIVISION OF COMPLIANCE
REGION I
970 BROAD STREET
NEWARK, NEW JERSEY 07102

3944

AREA CODE: 201-
TELEPHONE: 645-

IN REPLY REFER TO:

JAN 28 1972

Niagara Mohawk Power Corporation
Attention: Mr. F. J. Schneider
Vice President - Operations
300 Erie Boulevard West
Syracuse, New York 13202

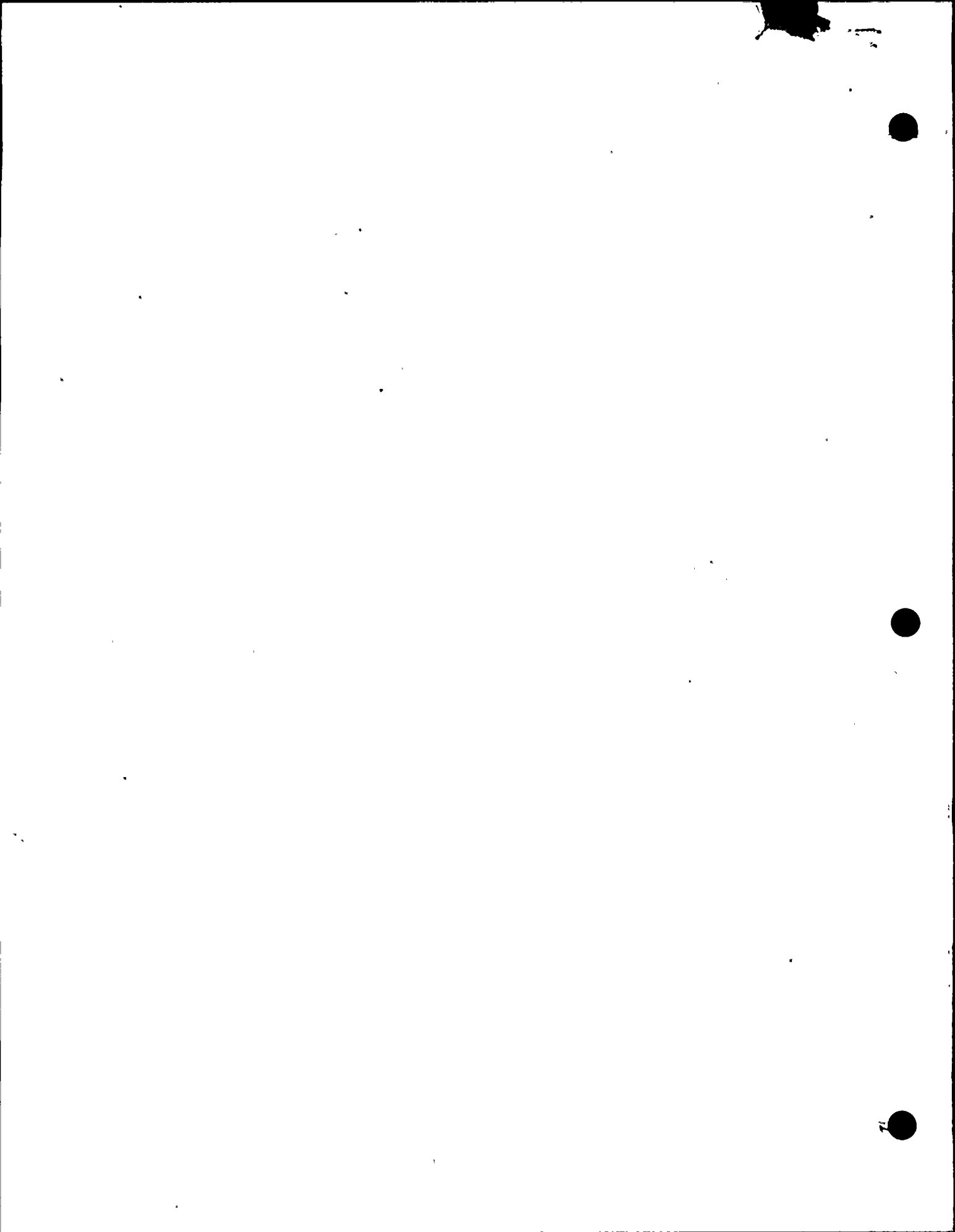
Docket No. 50-220

Gentlemen:

This refers to the inspection conducted by Mr. F. S. Cantrell of this office on December 14 - 16, 1971 of operations authorized by AEC License No. DPR-17 and to the discussion of our findings held by Mr. Cantrell with Mr. Burt of your staff at the conclusion of the inspection.

Areas examined during this inspection included administrative controls; the power ascension program; the results of the test program for operation at 1850 MWT; the testing performed to determine stack sample line losses; the calculations of release rates; the evaluation of the significance of a tubing crack in the main steam flow restrictors; the preventive maintenance program; the installation of a strong motion seismograph; and the evaluation of an emergency drill. Within these areas, the inspection consisted of selective examinations of procedures and representative records, interviews with plant personnel, and observations by the inspector.

It is our understanding, based on our inspector's findings and the referenced discussions, that your quality assurance program for the operation of your Nine Mile Point facility has not been fully implemented; however, we understand that detailed procedures for implementation of your quality assurance program have been prepared. It is requested that you provide us within 30 days with copies of these procedures and the date the quality assurance program will be fully implemented. In your response to this request, please include appropriate cross references to show how each of the criteria listed in 10 CFR Part 50, Appendix B, is satisfied. This information is requested only to facilitate our inspection effort; therefore, any documents sent to us pursuant to this request will not be placed in the Public Document Room and will be returned to you upon completion of our review.



No items of noncompliance were identified in the areas examined during the inspection.

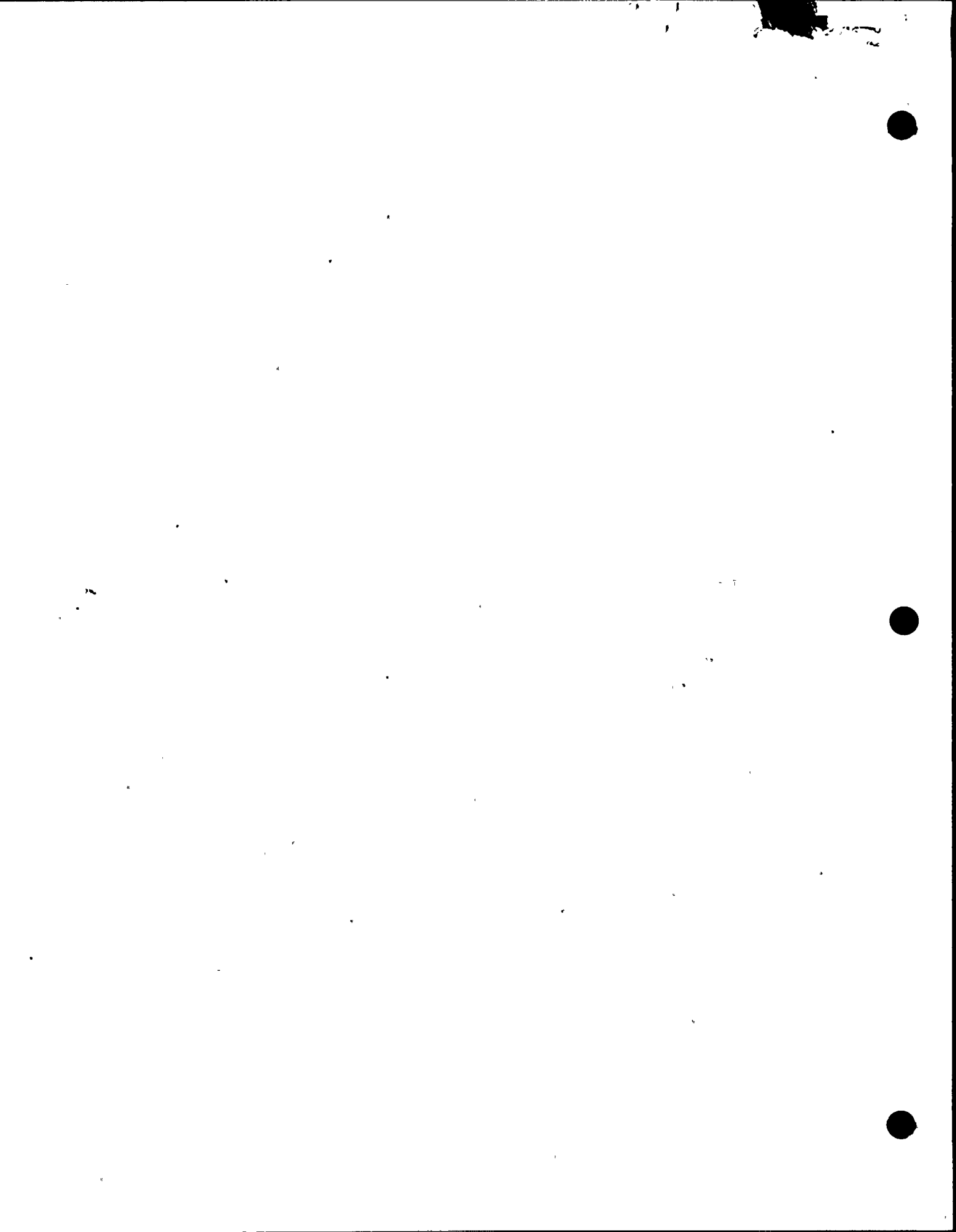
Should you have any questions concerning this inspection, we will be pleased to discuss them with you.

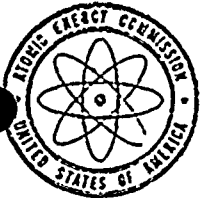
Very truly yours,

James P. O'Reilly
Director

cc: Mr. P. A. Burt, Station Superintendent

bcc: A. Giambusso, CO
L. Kornblith, CO
R. H. Engelken, CO
P. A. Morris, DRL
J. G. Keppler, CO
CO Files
✓ BK Central Files
PDR
Local PDR
NSIC
T. Laughlin, DTIE





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JAN 5 1972

R. T. Carlson, Acting Chief, Reactor Testing & Operations Br.
Division of Compliance, HQ

CO INQUIRY REPORT NO. (50-220)71-07
NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT 1 - BWR
PREMATURE OPENING OF RELIEF VALVES FOLLOWING A REACTOR SCRAM

The subject inquiry report is forwarded for your information.

The licensee stated that water may have been carried over into the main steam lines and caused a water hammer that caused the relief valves to relieve momentarily. Reactor level indication is only recorded as a maximum +3 feet (full scale); therefore, the maximum reactor level attained is not known.

Even though the Station Operations Review Committee approved the return to power the same day, NMP has not completed their analysis of this event. Mr. Burt stated that he did not consider this event an abnormal occurrence as defined in the Technical Specifications, however, he agreed to submit a written report to DRL by January 20, 1972. We will review this subject during the next inspection.

G. L. Madsen

G. L. Madsen
Acting Sr. Reactor Inspector

Enclosure:
Subject Inquiry Report

cc: E. G. Case, DRS (3)
R. S. Boyd, DRL (2)
R. C. DeYoung, DRL (2)
D. J. Skovholt, DRL (3)
H. R. Denton, DRS (2)
A. Giambusso, CO
L. Kornblith, CO
R. H. Engelken, CO
Regional Directors, CO
CO Files
DR Central Files ✓



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CO Inquiry Report No. 50-220/71-07

Subject: Niagara Mohawk Power Corporation

License No.: DPR-17

Facility: Nine Mile Point 1 - BWR

Title: Premature Opening of Relief Valves Following a Reactor Scram

Prepared by: *F. S. Cantrell, Jr.*
F. S. Cantrell, Jr., Reactor Inspector

1/5/72
Date

A. Date & Manner AEC was Informed:

By telephone call from Mr. P. A. Burt, Station Superintendent, on January 3, 1972. Additional information was provided in phone conversations with Mr. Burt on January 4 and 5, 1972.

B. Description of Particular Event or Circumstance:

While performing a routine monthly surveillance test on the reactor level instrumentation on December 31, 1971, the mechanic's wrench slipped and hit a companion instrument, apparently causing coincident high level signals that tripped the turbine. The turbine anticipatory trip initiated a reactor scram. Reactor pressure increased to 1065 psi at +3 seconds, decayed to 850 psi at +32 seconds, and increased to 1115 psi at +120 seconds. Reactor level decreased to -2.8' at +11 seconds (0 in normal level). The feed water controller increased makeup flow and the level increased to +3' (full scale on the recorder) at +63 seconds. The high reactor level initiated closure of the main steam isolation valves. At this point two of the electromatic releif valves (RV) relieved to the torus for 4 seconds. The RV are set to relieve between 1135 and 1145 psi.

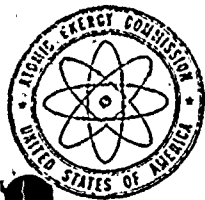
C. Action by Licensee:

A calibration check of the feedwater controls did not identify any reason for overfilling the system. The Station Operating Review Committee reviewed this occurrence and approved the return to power. The plant was brought back on line at 11:55 pm on December 31, 1971. The licensee is making a detailed analysis to determine the significance of this event



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NOV 9 1971

Niagara Mohawk Power Corporation
ATTN: Mr. F. J. Schneider,
Vice President - Operations
300 Erie Boulevard West
Syracuse, New York 13202

Docket No. 50-220

Gentlemen:

This refers to the inspection conducted by Mr. T. Young, Jr. of this office on October 5 and 6, 1971 of operations authorized by AEC License No. DPR-17 and to the discussions of our findings held by Mr. Young with Messrs. Burt and Pasternak of your staff and Mr. Atkinson of General Electric at the conclusion of the inspection.

Areas examined during this inspection included refueling operations, fuel sipping, fuel inspection, fuel bundle reconstitution, fuel accountability and fuel handling. Within these areas, the inspection consisted of selective examinations of procedures and representative records, interviews with plant personnel, and observations by our inspector.

No items of noncompliance were identified in the areas examined during the inspection.

No reply to this letter is necessary; however, should you have any questions concerning this inspection, we will be pleased to discuss them with you.

Very truly yours,

James P. O'Reilly
Director

cc: P. A. Burt, Station Superintendent

bcc: A. Giambusso, CO
L. Kornblith, CO
R. H. Engelken, CO
J. G. Keppler, CO (5)
DR Central Files

