



UNITED STATES
ATOMIC ENERGY COMMISSION
DIVISION OF COMPLIANCE
REGION I
970 BROAD STREET
NEWARK, NEW JERSEY 07102

201 645-3944

MAR 17 1972

J. G. Keppler, Chief, Reactor Testing & Operations Br.
Division of Compliance, HQ

CO INQUIRY REPORT NO. 50-220/72-01
NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT 1 - BWR
EQUIPMENT FAILURE - REACTOR PRESSURE CONTROLLED BY USE OF RELIEF VALVES

The subject inquiry report is forwarded for your information.

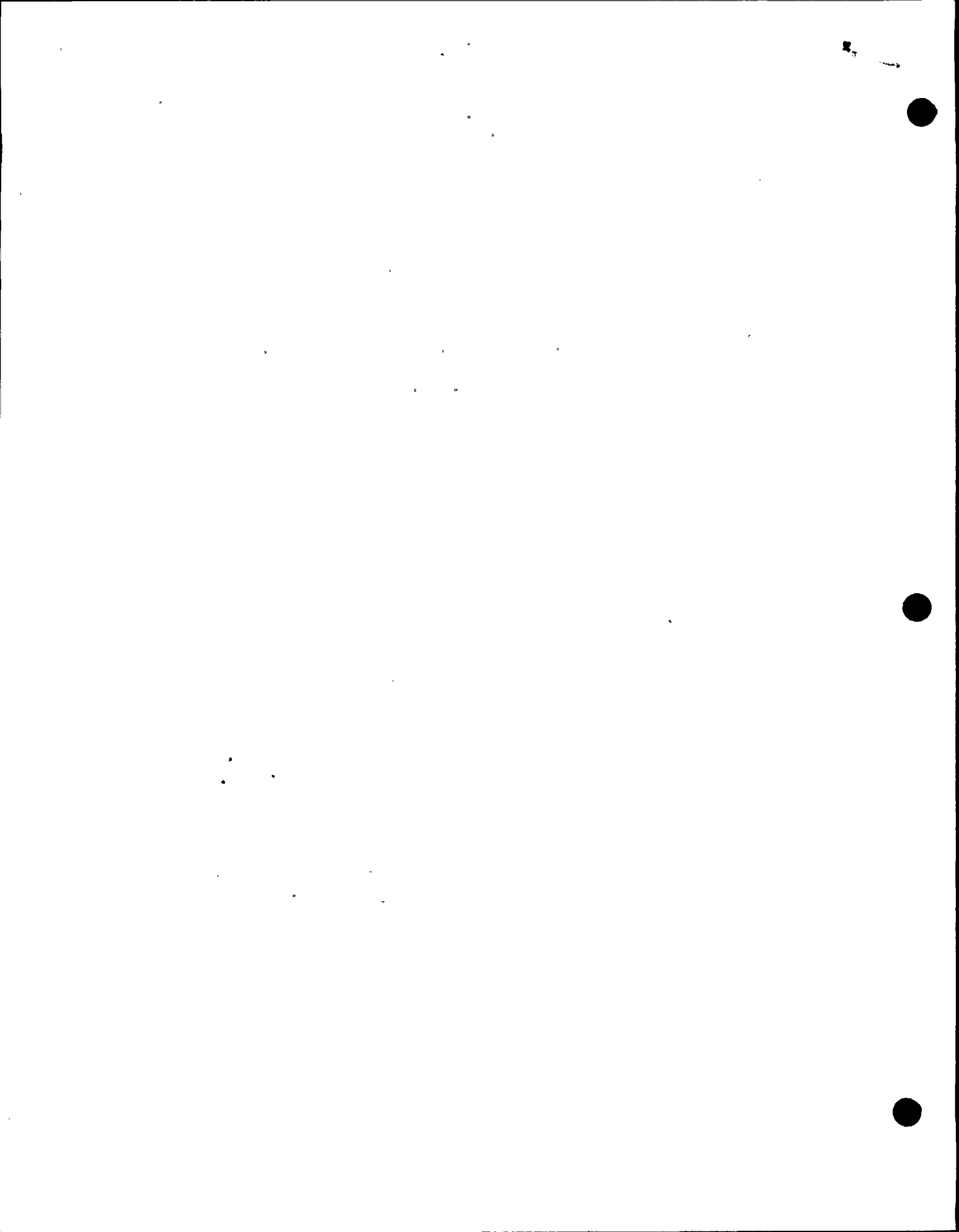
The original report of the failure was reported as a straight forward reactor scram due to the feedwater controller locking out on loss of instrument power. As a result of subsequent questions about the specific cause of the problem with the instrument power supply, the other details were provided. This is the second time in the past three months that the inspector is aware of that the relief valves have operated. (Inquiry Report 220/71-07)

This event and the need for full disclosure of details will be reviewed during the next inspection.

R. T. Carlson for
R. T. Carlson
Reactor Inspector

Enclosure:
Subject Inquiry Report

cc: E. G. Case, DRS (3)
R. S. Boyd, DRL (2)
R. C. DeYoung, DRL (2)
D. J. Skovholt, DRL (3)
H. R. Denton, DRL (2)
L. Kornblith, CO
R. H. Engelken, CO
Regional Directors, CO
CO Files
DR Central Files



CO INQUIRY REPORT NO. 50-220/72-01

Subject: Niagara Mohawk Power Corporation

License No.: DPR-16

Facility: Nine Mile Point 1 - BWR

Title: Equipment Failure - Reactor Pressure Controlled by Use of Relief Valves

Prepared by: F. S. Cantrell, Reactor Inspector

3/17/72
Date

A. Date & Manner AEC was Informed:

By telephone call from Mr. P. A. Burt, General Superintendent, on March 9, 1972, and a telephone call to Mr. T. J. Perkins, Station Superintendent, on March 17, 1972:

B. Description of Particular Event or Circumstance:

The reactor scrambled on February 28, 1972 as a result of a spurious transfer of the instrument power supply motor generator set to the station batteries. The feedwater controller locked out and caused a turbine trip as a result of a reactor high level. The turbine trip caused a reactor scram. The reactor level rose above the emergency condenser inlet and reactor pressure was controlled by operation of the electromatic relief valves for a period of approximately 15 minutes. When reactor level was lowered sufficiently, the emergency condensers were used to remove decay heat and control pressure.

C. Action by Licensee:

The specific cause of the transfer to station batteries has not been determined. The licensee agreed to submit a written report to DRL by March 24, 1972.

23

