50-220

UNITED STATES
ATOMIC ENERGY COMMISSION
DIVISION OF COMPLIANCE
REGION I
970 BROAD STREET
NEWARK, NEW JERSEY 07102

201 645-

MAY 08 1972

J. B. Henderson, Chief, Reactor Construction
Branch, Regulatory Operations, HQ

NIAGARA MOHAWK POWER CORPORATION
(NINE MILE POINT 2)
CO REPORT NO. 72-01

The subject inspection report is forwarded for information.

The meeting agenda included subjects contained in PI 3900 in addition to a quality assurance orientation to apprise the applicant of the requirements, purpose, and details of the initial QA program inspection.

The application for construction permit was submitted in March 1972. The quality assurance program inspection is scheduled in the week of June 12, 1972.

E. M. Howard
E. M. Howard
Senior Reactor Inspector

Enclosure:
Subject Inspection Report

cc: E. G. Case, RS (3)
R. S. Boyd, RL (2)
R. C. DeYoung, RL (2)
D. J. Skovholt, RL (3)
H. R. Denton, RL (2)
L. Kornblith, Jr., RO
R. H. Engelken, RO
RO Files
DR Central Files

U. S. ATOMIC ENERGY COMMISSION
REGULATORY OPERATIONS
REGION I

CO Inspection Report No. 72-01

Subject: Niagara Mohawk Power Corporation

Nine Mile Point 2 License No.

Location: Scriba, New York Priority

Category A-1

Type of Licensee: Power 1100 MWe BWR

Type of Inspection: Announced, Initial Management Meeting

Dates of Inspection: April 10, 1972

Dates of Previous Inspection: None

Principal Inspector: Not Currently Assigned Date

Accompanying Inspectors: E. M. Howard 5-5-72
E. M. Howard, Senior Reactor Inspector Date

Wm. M. Hayward May 4, 1972
W. M. Hayward, Reactor Inspector Date

Other Accompanying Personnel: NONE Date

Reviewed By: R. F. Heishman 5-5-72
R. F. Heishman, Reactor Inspector Date

Proprietary Information: NONE

SECTION I

Enforcement Action

None

Licensee Action on Previously Identified Enforcement Matters

None

Unresolved Items

None

Status of Previously Reported Unresolved Items

None

Persons Contacted

The following corporate officers and technical staff personnel were present at the corporate level meeting held in the NMPC offices in Syracuse, New York.

F. J. Schneider, Vice President - Operations, NMPC
T. J. Brosnan, Vice President - Chief Engineer, NMPC
P. A. Burt, General Superintendent- Nuclear Generation, NMPC
D. P. Dise, Manager of Quality Assurance, NMPC
T. G. Bassett, QA Supervisor, NMP 2, NMPC
G. K. Rhode, Manager System Planning, NMPC
J. L. Hilke, Project Manager, NMP-2, NMPC
S. F. Manno, Assistant Project Manager, NMP 2, NMPC
T. J. Perkins, Superintendent - Operations, NMP 1, NMPC
J. P. O'Reilly, Director, RO:I

Management Interview

The following subjects were discussed with the NMPC management and contract personnel on April 10, 1972.

A. A functional description of the regulatory organization, the RL - RO relationship and the RO organization.

B. A description of the RO inspection responsibilities and methods of communication with the licensee with particular emphasis on the management interview following each inspection.

C. A definition of the inspector-licensee relationship and the requirement for clear channels of communication.

D. A description of the conclusions which the regulatory staff must reach before an operating license can be issued, the role of RO in reaching these conclusions and the continuing role of RO after the facility is licensed and operational.

E. A discussion of enforcement procedures available to RO. An explanation of the new reporting system.

F. An explanation of the type and frequency of inspection, the information needs of the inspector and the necessity for pertinent records being readily available.

G. An explanation of the need of RO for ready access to licensee and contractor personnel in order to obtain the information required by RO.

H. An explanation of the purpose of the management interview at the conclusion of each inspection.

I. A description of concurrent inspection efforts at vendor facilities and use of consultants to assist in areas of specialized concern.

J. An explanation of the necessity for complete documentation of deviations, including the method of resolution and approvals.

K. An explanation of the necessity for advising RO promptly of significant discrepancies and a general discussion of a regulatory reporting requirement.

L. An announcement of the date of the initial QA inspection as being in the week of May 29, 1972.

M. An explanation of the requirement for the applicant to prepare a QA program as defined by Appendix B, Title 10, Code of Federal Regulations, Part 50, using the transparency slide program for guidance and examples.

N. An explanation of the purpose of the first inspection, being to objectively evaluate the QA program and proposed methods of implementation.

O. An explanation of the emphasis in the regulation on QA and QC records relative to type and availability.

P. An explanation of the scope of the inspection.

No other subjects were discussed in the meeting, nor were there any additional subjects inspected.

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MAY 04 1972

Niagara Mohawk Power Corporation
Attention: Mr. F. J. Schneider
Vice President, Operations
300 Erie Boulevard
Syracuse, New York

Gentlemen:

This refers to the meeting which was held at our request at the Niagara Mohawk Power Corporation corporate office, Syracuse, New York, on April 10, 1972. This meeting, which included senior members of my staff and management representatives from the Niagara Mohawk Power Corporation, concerned activities relating to your request for an AEC Construction Permit for your proposed nuclear facility, Nine Mile Point Unit No. 2.

The matters discussed at this meeting primarily related to your planned systems for preparing and implementing your quality assurance program for Nine Mile Point Unit No. 2 in accordance with the requirements of Appendix B to 10 CFR 50. In addition, we discussed, in general terms, our construction inspection program and the specifics of our pending quality assurance inspection which we plan to conduct on or about July 12, 1972.

It is our view that these discussions were helpful, in that our understanding of your plans and systems has been enhanced significantly. Furthermore, we believe that the discussions relating to details of our construction inspection program should prove to be mutually beneficial to our organizations.

No reply to this letter is necessary; however, should you have any questions concerning this meeting, we will be pleased to discuss them with you.

Very truly yours,

James P. O'Reilly
Director

bcc: L. Kornblith, Jr., CO
R. H. Engelken, CO
P. A. Morris, DRL
J. B. Henderson, CO
CO Files
DR Central Files
PDR
Local PDR
NSIC
DTIE

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201 645-3944

APR 27 1972

J. G. Keppler, Chief, Reactor Testing & Operations Br.
Division of Compliance, HQ

CO INQUIRY REPORT NO. 50-220/72-02
NIAGARA MOHAWK POWER CORPORATION
NINE MILE POINT 1 - BWR
EQUIPMENT FAILURE - LOOSE BAFFLES IN TORUS

The subject inquiry report is forwarded for your information.

This inspection was made in response to a written request from DRL, and the licensee stated that a report would be submitted within the two weeks as requested in DRL's letter. This will be prior to reactor startup.

We plan to review the licensee's corrective action during the next routine inspection which is currently scheduled for the week of May 15, 1972.

R. T. Carlson
Senior Reactor Inspector

Enclosure:
Subject Inquiry Report

cc: E. G. Case, DRS (3)
R. S. Boyd, DRL (2)
R. C. DeYoung, DRL (2)
D. J. Skovholt, DRL (3)
H. R. Denton, DRL (2)
L. Kornblith, CO
R. H. Engelken, CO
Regional Directors, CO
CO Files
DR Central Files

CO Inquiry Report No. 50-220/72-02

Subject: Niagara Mohawk Power Corporation

License No.: DPR-17

Facility: Nine Mile Point 1 - BWR

Title: Equipment Failure - Loose Baffles in Torus

Prepared by: F. S. Cantrell, Jr., Reactor Inspector — Date: 4/27/72

A. Date and Manner AEC was Informed:

By telephone call from the Station Superintendent, Mr. T. J. Perkins, on April 27, 1972.

B. Description of Particular Event or Circumstance:

The water in the torus was drained to permit an inspection of the torus and the baffles. Seventeen of the thirty-six baffles located within one foot of the six downcomers on the discharge lines from the auto depressurization relief valves, were found loose and laying on the floor of the torus. One baffle about ten feet from a downcomer was found loose on one end. The 3/8" bolts that hold the baffles in position had broken. (The baffles are 12 inch channels, 7'9" to 24'9" long and are mounted vertically.) No scouring was observed on the torus below the downcomer.

C. Action by Licensee:

The licensee is evaluating his findings and will make a written report to DRL by May 5, 1972.