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Docket No. 50-410

DEC 29 1972

Niagara Mohawk Power Corporation
ATTN: Mr. Thomas J. Brosnan
Vice President & Chief Engineer
300 Erie Boulevard West
Syracuse, New York 13202

Gentlemen:

In order that we may continue our review of your application for a license to construct the Nine Mile Point Nuclear Station Unit 2 additional information is required. The information requested is described in the enclosure and pertains to two areas. The first is relative to the main steam line isolation valve water seal system and the second is a generic problem pertaining to circuit and circuit breakers.

The questions in the enclosure are referenced by sections corresponding to the relevant sections of the Preliminary Safety Analysis Report and are numbered in seriatim with previous requests for information.

In order to maintain our licensing review schedule, we will need a completely adequate response to all enclosed questions by February 27, 1973. Please inform us within 7 days after receipt of this letter of your confirmation of the schedule date or the date you will be able to meet. If you cannot meet our specified date or if your reply is not fully responsive to our request, it is highly likely that the overall schedule for completing the licensing review for the project will have to be extended. Since reassignment of the staff's efforts will require completion of the new assignment prior to returning to this project, the extent of the extension will most likely be greater than the delay in your response.

Please contact us if you have any questions regarding the information requested.

Sincerely,

Original signed by
Robert A. Clark

Robert A. Clark, Chief
Gas Cooled Reactors Branch
Directorate of Licensing

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SURNAME ▶	ABournia:nb	RAClkqrk				LT
DATE ▶	12/27/72	12/27/72				

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Original signed by
Robert A. Clark

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DATE ▶						

cc: (Con't)

Chairman, Atomic Safety and
Licensing Board Panel
U. S. Atomic Energy Commission
Washington, D. C. 20545

Chairman, Atomic Safety and
Licensing Appeals Board
U. S. Atomic Energy Commission
Washington, D. C. 20545

OFFICE ▶						
SURNAME ▶						
DATE ▶						

REQUEST FOR ADDITIONAL INFORMATION

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION-2

DOCKET NO. 50-410

7.0 CONTROL AND INSTRUMENTATION

7.13 Two incidents have occurred at a nuclear power plant that indicate a deficiency in the control circuit design that warrants a review of the control circuits of each facility to assure that these types of deficiencies do not exist. Both incidents involved the inadvertent disabling of a component by racking out the circuit breaker for a different component. In one case, this caused the loss of capability to isolate secondary containment when this capability was required. In the second case, the racking out of a breaker for one pump disabled not only the pump being removed from service but also its redundant counterpart. Both of these occurrences resulted from the use of auxiliary contacts on the movable portion of the circuit breakers in the control circuits of other components. When the breaker is racked out, the control circuit employing these contacts is opened and may be rendered inoperable. As a result of these occurrences, we request that you perform a review of the control circuits of all safety related equipment for the plant to assure that disabling of one component does not, through incorporation in other interlocking or sequencing controls, render other components inoperable. All modes of test, operation, and failure must be considered.

Confirm that this problem is not prevalent in Nine Mile Point-2 facility.

10.0 AUXILIARY SYSTEMS

10.52 Describe how you will demonstrate seal integrity (to meet the requirements of 10 CFR 100) of the water seal system which includes the two isolation valves in the main steam line when a pressurizing medium is introduced into the elevated temperature steam line during an accident condition. Valve distortion due to thermal shock could be excessive and permit a greater leakage than if no water seal were used. If analytical means will be used to verify the design, describe the design criteria, methods of analysis and assumptions. If test methods will be used, provide the test procedures and acceptance criteria.