UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 27, 2017

Mr. Peter A. Gardner
Site Vice President
Monticello Nuclear Generating Plant
Northern States Power Company - Minnesota (NSPM)
2807 West County Road 75
Monticello, MN 55362-9637

SUBJECT: MONTICELLO NUCLEAR GENERATING PLANT - ISSUANCE OF AMENDMENT RE: REVISION TO TECHNICAL SPECIFICATION SURVEILLANCE REQUIREMENT 3.8.4.2 (CAC NO. MF7576)

Dear Mr. Gardner:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No.192 to Renewed Facility Operating License No. DPR-22 for the Monticello Nuclear Generating Plant. The amendment consists of changes to the technical specifications (TSs) in response to your application dated April 4, 2016.

The amendment revises TSs surveillance requirement (SR) associated with TS 3.8.4, "DC [direct current] Sources – Operating." Specifically, the amendment revises SR 3.8.4.2 by increasing the 125 Volt DC battery charger test output current to 75 amperes (amps) from the current test level of 50 amps, and removes the second (alternate) method specified to perform the SR.

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

Robert F. Kuntz, Senior Project Manager
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-263

Enclosures:
1. Amendment No192 to DPR-22
2. Safety Evaluation

cc w/encls: Distribution via ListServ

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-263

MONTICELLO NUCLEAR GENERATING PLANT

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No.192
License No. DPR-22

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Northern States Power Company (NSPM, the licensee), dated April 4, 2016, as supplemented by letters dated October 3 and November 22, 2017, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Renewed Facility Operating License No. DPR-22 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 192, are hereby incorporated in the license. NSPM shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Joel S. Wiebe, Acting Chief
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Operating
License No. DPR-22 and
Technical Specifications

Date of Issuance: February 27, 2017

ATTACHMENT TO LICENSE AMENDMENT NO. 192

MONTICELLO NUCLEAR GENERATING PLANT

RENEWED FACILITY OPERATING LICENSE NO. DPR-22

DOCKET NO. 50-263

Replace the following page of Renewed Facility Operating License DPR-22 with the attached revised page. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

REMOVE	INSERT
3	3

Replace the following pages of Appendix A, Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE	INSERT
3.8.4-2	3.8.4-2

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Required Action and associated Completion Time not met.	C.1 Be in MODE 3. AND C.2 Be in MODE 4.	12 hours 36 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.8.4.1	Verify battery terminal voltage is greater than or equal to the minimum established float voltage.	7 days
SR 3.8.4.2	Verify each required battery charger supplies the following: • ≥ 150 amps for 250 VDC Div 1 • ≥ 110 amps for 250 VDC Div 2 • ≥ 75 amps for 125 VDC subsystems, at greater than or equal to the minimum established float voltage for ≥ 4 hours.	24 months

2. Pursuant to the Act and 10 CFR Part 70, NSPM to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operations, as described in the Final Safety Analysis Report, as supplemented and amended, and the licensee's filings dated August 16, 1974 (those portions dealing with handling of reactor fuel);

3. Pursuant to the Act and 10 CFR Parts 30, 40 and 70, NSPM to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

4. Pursuant to the Act and 10 CFR Parts 30, 40 and 70, NSPM to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

5. Pursuant to the Act and 10 CFR Parts 30 and 70, NSPM to possess, but not separate, such byproduct and special nuclear material as may be produced by operation of the facility.

C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission, now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

1. Maximum Power Level

NSPM is authorized to operate the facility at steady state reactor core power levels not in excess of 2004 megawatts (thermal).

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.192, are hereby incorporated in the license. NSPM shall operate the facility in accordance with the Technical Specifications.

3. Physical Protection

NSPM shall implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO.192 TO

RENEWED FACILITY OPERATING LICENSE NO. DPR-22

NORTHERN STATES POWER COMPANY

MONTICELLO NUCLEAR GENERATING PLANT

DOCKET NO. 50-263

1.0 INTRODUCTION

By application dated April 4, 2016 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML16099A097), as supplemented by letters dated October 3 and November 22, 2016 (ADAMS Accession No. ML16277A486 and ML16327A465, respectively), Northern States Power Company (NSPM, the licensee), requested changes to the technical specifications (TSs) for Monticello Nuclear Generating Plant. The supplemental letters dated October 3 and November 22, 2016, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the U.S. Nuclear Regulatory Commission (NRC/Commission) staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on June 7, 2016 (81 FR 36621).

The proposed changes would revise TS Surveillance Requirement (SR) associated with TS 3.8.4, "DC [direct current] Sources – Operating." Specifically, the proposed license amendment request (LAR) would revise SR 3.8.4.2 by increasing the 125 Volt (V) DC battery charger test output current to 75 amperes (amps) from the current test level of 50 amps, and remove the second (alternate) method specified to perform the SR.

2.0 REGULATORY EVALUATION

Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50, Section 50.36(c), TSs requires, in part, that TSs include items in five specific categories related to station operation. These categories are: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation (LCO); (3) SRs; (4) design features; and (5) administrative controls. The proposed changes in the LAR relate to SRs to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the LCO will be met.

The construction permit for MNGP was issued by the Atomic Energy Commission (AEC) on June 19, 1967. AEC issued a provisional operating license (OL) on September 8, 1970, allowing commercial operation. The NRC issued full-term OL issued on January 9, 1981. The

plant general design criteria (GDC) are discussed in Chapter 1.2 "Principal Design Criteria," and in Appendix E "Plant Comparative Evaluation with the Proposed AEC 70 Design Criteria," of the MNGP Updated Safety Analysis Report (USAR). The AEC published the final rule that added 10 CFR Part 50, Appendix A, "General Design Criteria for Nuclear Power Plants," in the *Federal Register* (36 FR 3255) on February 20, 1971, with the rule effective on May 21, 1971. In accordance with an NRC staff requirements memorandum from S. J. Chilk to J. M. Taylor, "SECY-92-223 - Resolution of Deviations Identified during the Systematic Evaluation Program," dated September 18, 1992 (ADAMS Accession No. ML003763736), the Commission decided not to apply the 10 CFR Part 50, Appendix A, GDC to the plants with construction permits issued prior to May 21, 1971.

Based on the above, the GDC which constitute the licensing bases for MNGP are those described in Chapter 1.2 and Appendix E of the USAR. The NRC staff identified the following AEC proposed criteria as being applicable to the proposed amendment:

Criterion 24, "Emergency Power for Protection System," which states: In the event of loss of all offsite power, sufficient alternate sources of power shall be provided to permit the required functioning of the protective systems.

Criterion 39, "Emergency Power for Engineered Safety Features," which states: Alternate power systems shall be provided and designed with adequate independency, redundancy, capacity, and testability to permit the functioning required of the engineered safety features. As a minimum, the on-site power system and the off-site power system shall each, independently, provide this capacity assuming a failure of a single active component in each power system.

Criterion 41, "Engineered Safety Features Performance Capability," which states: Engineered safety features such as emergency core cooling and containment heat removal systems shall provide sufficient performance capability to accommodate partial loss of installed capacity and still fulfill the required safety function. As a minimum, each engineered safety feature shall provide this required safety function assuming a failure of a single active component.

The NRC staff also considered the following guidance documents during this review:

NUREG-1433, Revision 4 "Standard Technical Specifications for General Electric Boiling Water Reactor/4 (BWR/4) plants" (ADAMS Accession Nos. ML12104A192 and ML12104A193), Section 3.8.4, "DC Sources – Operating."

NUREG-0800, "Standard Review Plan [SRP] for Review of Safety Analysis Reports for Nuclear Power Plants," Section 8.3.2, "DC Power Systems (Onsite)," (ADAMS Accession No. ML100740391). SRP, Section 8.3.2, states that the capacity of the battery charger should be based on an evaluation of the largest combined demands of the various continuous steady-state loads plus charging capacity to restore the battery from the design minimum charge state to the fully charged state within the time stated in the design basis, regardless of the status of the plant when these demands occur.

According to above guidance of SRP, Section 8.3.2, the time to recharge a battery should be based on the design basis of the plant. Since, the design basis provided in the MNGP USAR did not specify the time to recharge a battery, the NRC staff used the guidance provided in the following industry standards:

- Institute of Electrical and Electronics Engineers (IEEE) Standard 946-1985 "IEEE Recommended Practice for the Design of Safety-Related DC Auxiliary Power Systems for Nuclear Power Generating Stations." This standard recommends 8-12 hours to recharge a battery to approximately 95 percent of capacity.

- IEEE Standard 946-2004 "IEEE Recommended Practice for the Design of Safety-Related DC Auxiliary Power Systems for Generating Stations." In 2004, this IEEE standard revised its recommendation to recharge a battery within 8-24 hours.

3.0 TECHNICAL EVALUATION

3.1 Description of the Monticello DC electrical power system

The DC electrical power system provides the ac [alternating current] emergency power system with control power. It also provides both motive and control power to selected safety-related equipment. The DC subsystems provide electrical power to inverters, which in turn power the ac instrument loads. The DC electrical power system is designed to have sufficient independence and redundancy to perform its safety functions, assuming a single failure. The Division 1 and Division 2 250 VDC [volts direct current] electrical power subsystems provide power to the associated uninterruptible ac power supply. The Division 1 electrical power subsystem also provides power to support the reactor core isolation cooling (RCIC) system motor-operated valves (MOVs), the turbine pumps, and other noncritical loads. The Division 2 electrical power subsystem supplies power for the high pressure coolant injection (HPCI) system MOVs, the HPCI auxiliary oil pumps, and the control room ventilation system control circuits. Each 250 VDC electrical power subsystem consists of two-in-series 125 VDC batteries, two normally inservice 125 VDC chargers, a spare 125 VDC charger, and all of the control equipment and interconnecting cabling to the associated distribution panel. The inservice and spare chargers are supplied from the associated ac load group.

Division 1 and Division 2 125 VDC electrical power subsystems provide control power to the associated 4.16 kilovolt essential bus and for each of the two 480 VAC essential load centers. Each 125 VDC electrical power subsystem consists of a battery, a charger, and all the control equipment and interconnecting cabling up to the associated distribution panels. The inservice chargers are supplied from the associated ac load group. The design includes a spare charger that can be used for either the Division 1 or Division 2 125 VDC electrical power subsystem. However, the spare charger is supplied from the Division 2 ac load group.

During normal operation, the DC loads are powered from the battery chargers with the batteries floating on the system. In case of loss of normal power to the battery charger, the DC loads are automatically powered from the station batteries.

Section 8.5.11 of USAR states that the 250 VDC batteries are sized to provide adequate voltage at the terminals of connected loads for the duration of a 4-hour station blackout (SBO) event. The demands placed on the battery by a SBO event envelope the demands which would be placed on the batteries by any design basis event. The battery chargers are sized to charge the batteries while supplying the normal continuous DC loads.

Section 8.5.2.1 of USAR states that the each 125 VDC battery is sized to provide adequate voltage at the terminals of connected loads for the duration of a 4-hour SBO event, and is

capable of meeting power requirements during a design basis event. Each 125 VDC battery charger is sized to charge the battery while supplying the normal continuous DC loads.

3.2 Proposed SR changes

The LAR proposed changing SR 3.8.4.2 are as follows:

Current SR 3.8.4.2

Surveillance: Verify each required battery charger supplies the following:

- ≥ 150 amps for 250 VDC Division 1
- ≥ 110 amps for 250 VDC Division 2
- ≥ 50 amps for 125 VDC subsystem

at greater than or equal to the minimum established float voltage for ≥ 4 hours.

OR

Verify each required battery charger can recharge the battery to the fully charged state within 24 hours for 250 VDC subsystems and 8 hours for 125 VDC subsystems while supplying the largest combined demands of the various continuous steady state loads, after a battery discharge to the bounding design basis event discharge state.

Frequency: 24 months

Proposed SR 3.8.4.2

Surveillance: Verify each required battery charger supplies the following:

- ≥ 150 amps for 250 VDC Division 1
- ≥ 110 amps for 250 VDC Division 2
- ≥ 75 amps for 125 VDC subsystem

at greater than or equal to the minimum established float voltage for ≥ 4 hours.

Frequency: 24 months

The proposed change increases the 125 VDC battery charger test output current to ≥ 75 amps from the current test level of ≥ 50 amps. The licensee determined that the current test level of ≥ 50 amps is nonconservative. Also, the change would remove second (alternative) testing method for meeting the SR because the licensee determined that it has not been utilized, nor will the alternative be used in the future.

3.3 NRC staff's review

Enclosure 2 of the LAR provided MNGP Calculation 91-006, Revision 4, "125 VDC Battery Charger Sizing," for supporting the proposed changes. In the calculation, the licensee provided the following acceptance criteria for sizing or determining the adequacy of the battery charger.

(1) Normal Operation: USAR, Section 8.5, states that each 125 VDC charger is capable of carrying the normal load and the same time supplying additional charging current to keep the batteries in a fully charged condition. This criteria is consistent with the IEEE 946-1985 recommendation that the charger output current capability should be greater than the continuous loads plus the largest combination of noncontinuous loads likely to occur simultaneously during normal plant operation.

(2) Time to recharge a discharged battery: The LAR stated that there are currently no explicit licensing basis criteria in place for the recharge time after design basis discharge. The licensee chose to calculate the recharge time using the IEEE standard 946 methodology, with system load on the battery considered to be largest continuous load at the end of station blackout.

According to IEEE 946-1985, the battery charger should be sized so that time for re-charging a battery (to approximately 95 percent of capacity) should be between 8 to12 hours. This recommendation was revised in IEEE 946-2004, according to which the time for recharging a battery should be between 8 to 24 hours.

Normal Operation

MNGP Calculation 91-006, "125 VDC Battery Charger Sizing," states that the maximum DC loads under normal plant conditions for Divisions I and II, are 28 amps and 29 amps respectively. The maximum steady-state DC loads after an SBO discharge are 61.54 amps (approximately 62 amps) and 57.95 amps (approximately 58 amps). Therefore, the licensee proposed to change SR for 125 VDC charger to be able to supply maximum current from ≥ 50 amps to ≥ 75 amps, which envelope the maximum steady-state load of 62 amps.

IEEE 946-1985 recommends that battery charger sizing consider the largest combination of continuous and non-continuous loads likely to occur simultaneously during normal plant operation. Therefore, The NRC staff's request for additional information (RAI) (ADAMS Accession No. ML16266A081) RAI 2 requested the licensee provide the noncontinuous loads and their ampere rating considered for the battery charger sizing based on normal operation criteria. The licensee's letter dated October 3, 2016, stated that each division will only have 4.40 amps of non-continuous loads. Based on this information, the staff finds that the proposed SR limit of ≥ 75 amps limit has sufficient margin to verify that the charger will be capable to supply largest combination of continuous and non-continuous loads likely to occur simultaneously during normal plant operation, and keep the connected divisional battery in a fully charged condition.

During review of the LAR, the NRC staff noted that according to MNGP USAR, Section 8.5.2.2, the 125 V battery chargers have a DC output rating of 80 amps for main chargers and 50 amps for common standby battery chargers. The staff noted that common standby battery charger may not be of adequate capacity considering the revised proposed ampere requirement of main chargers. In response to the staff's RAI 4, the licensee in its letter dated October 3, 2016,

clarified that the USAR was incorrect, and that as-left current limit setting for all three 125 VDC battery chargers (D10, D20 (main chargers), and D40 (standby charger)) is 78 to 80 amperes. In its letter dated November 22, 2016, the licensee provided a mark-up copy of the USAR according to which the nominal output current setting of each charger has been corrected as 80 amps.

Time to recharge a discharged battery: In the MNGP Calculation 91-006, "125 VDC Battery Charger Sizing," the licensee calculated that based on minimum charger of 75 amps, the charger will be able to: (1) Recharge Division I battery within 16 hours for after a design basis SBO event, and 11.6 hours after a full rated battery discharge; and (2) Recharge Division II battery within 18 hours for after a design basis SBO event, and 11.9 hours after a full rated battery discharge.

In the RAI 3, the NRC staff requested the licensee explain why the USAR does not provide a basis for a battery charger capability of recharging a discharged battery within a certain time, consistent with TS bases and the LAR calculations for recharging a discharged battery. In response dated October 3, 2016, the licensee stated:

> The USAR describes the Design Basis for station battery charger sizing which, rather than providing for a particular recharge time under specific load conditions, assures the capacity to provide a float charge to the batteries while carrying certain system loads. Specifically, "Each 125 VDC [battery] charger is capable of carrying the normal 125 VDC load and at the same time supplying additional charging current to keep the batteries in a fully charged condition." The 24 hour recharge time referenced in this "Background" section of the TS Bases was not excerpted from the charger sizing Design Basis in the USAR, but was rather inadvertently incorporated from the standard wording from NUREG-1433, Revision 3, during Monticello's transition to the improved Standard Technical Specifications (ITS). A correction to the TS Bases was included for information as part of the LAR (Enclosure 1, Attachment 2). The battery charger sizing calculation included with the LAR compares the calculated recharge time with industry recommendations. This comparison is solely a prudent engineering practice and the battery charger sizing Design Basis remains as described in the USAR.

Although, according to the USAR, the recharge time is not part of current licensing basis, the NRC staff finds that the recharge times calculated in MNGP Calculation 91-006, "125 VDC Battery Charger Sizing," meet the industry recommended recharge time of 8 hours to 24 hours as recommended in IEEE 946-2004.

In RAI 5, the NRC staff requested the licensee provide the following additional information: Explain how testing the battery at ≥ 75 amps at greater than or equal to the minimum established float voltage for ≥ 4 hours provides verification of the designed capability of the battery charger; provide the charger specifications regarding its allowable output voltage and current limit setting ranges; and briefly explain how the SR is performed for the first option, and the actual output voltage and current limit settings used during performance of SR 3.8.4.2.

In response to RAI 5, the licensee in the letter dated October 3, 2016, provided following additional information supporting the LAR change:

The battery charger capacity test demonstrates the equipment's ability to provide the output current and voltage required to meet the DC subsystem design requirements. The test is performed with the charger supplying a load bank while maintaining voltage at or above the required float voltage. For the 4-hour test duration, the output current is verified to remain above the minimum value considered in the battery charger sizing calculation and the charger output voltage is verified to remain equal to or greater than the battery cell manufacturer's minimum required float voltage. The voltage and output current design requirements are lower than the rated capacity of the battery charger.

The 125 VDC normal and standby battery chargers are made by C&D (model ARR130K100FMOD). The specifications for this charger model are:

DC Output Voltage
Nominal Rating: 132 VDC
Adjustable Range: 121.4-142.6 VDC

Current Output
Nominal Rating: 100 amps
Adjustable Range: Up to 125 percent of nominal (115 percent max for continuous operation)

To perform the battery charger capacity test for SR 3.8.4.2, a resistive load bank is connected to the charger output and programmed to control load at 76 to 78 amps. No adjustments are made to the battery charger settings for the surveillance which remain at the normal as left current limit setting of 78 to 80 amps and the normal float voltage setting of 129.5 VDC to 130.5 VDC. For the duration of the test, the charger output current and voltage are monitored to verify that load current remains above the design requirement value of 75 amps and that float voltage remains above the cell manufacturer's minimum required float voltage value of 127.6 VDC.

Based on above discussion and additional information by the licensee, the NRC staff finds that the proposed change to SR 3.8.4.2, to increase the 125 VDC battery charger test output current to ≥ 75 amps from the current test level of ≥ 50 amps, is acceptable.

In the LAR, the licensee also proposed to remove the following second (alternate) method of performing SR 3.8.4.2: "Verify each required battery charger can recharge the battery to the fully charged state within 24 hours for 250 VDC subsystems and 8 hours for 125 VDC subsystems while supplying the largest combined demands of the various continuous steady state loads, after a battery discharge to the bounding design basis event discharge state."

In the LAR, the licensee stated that the second option under SR 3.8.4.2 was added during the MNGP Improved TS (ITS) conversion process (based on standard TS) but has not been utilized, and it has been determined that it will not be utilized in the future.

The NRC staff finds that according to USAR, the licensee did not commit to verify that a battery charger can recharge a discharged battery within a certain time, although according to the calculation supplied in the LAR that 125 V battery charger can recharge a discharged battery within 24 hours, and meets the industry guideline (IEEE 946-2004) of maximum 24 hours. Also, the second method of performing the SR 3.8.4.2 (alternative) is an option rather than

requirement. The staff finds that according to NUREG-1433, Revision 4 "Standard Technical Specifications for General Electric Boiling Water Reactor/4 (BWR/4) plants," Section 3.8.4, "DC Sources – Operating." SR 3.8.4.2, one of two methods is adequate to verify that a battery charger can serve its purpose. Therefore, the staff finds deleting the second method (option) of performing SR 3.8.4.2 on the battery chargers is acceptable. The staff has determined that SR 3.8.4.2 will continue to meet 10 CFR 50.36(c)(3) following the changes.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Minnesota State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or change the surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (81 FR 36621). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: V. Goel, NRR

Date of issuance: Febuary 27, 2017

MONTICELLO NUCLEAR GENERATING PLANT - ISSUANCE OF AMENDMENT RE: REVISION TO TECHNICAL SPECIFICATION SURVEILLANCE REQUIREMENT 3.8.4.2 (CAC NO. MF7576) DATED FEBRUARY 27, 2017

DISTRIBUTION:

PUBLIC LPL3 r/f	RidsAcrs_MailCTR Resource
RidsNrrDssStsb Resource	RidsNrrDorlDpr Resource
RidsNrrDorlLpl3 Resource	RidsNrrPMMonticello Resource
RidsNrrLASRohrer Resource	RidsNrrDeEeeb
RidsRgn3MailCenter Resource	VGoel, NRR

ADAMS Accession No.: ML17013A435 *comments via email

OFFICE	DORL/LPL3/PM	DORL/LPL3/LA	EEEB/BC	STSB/BC
NAME	RKuntz	SRohrer	JZimmerman	AKlein*
DATE	01/19/17	01/18/17	12/20/17	1/24/17
OFFICE	OGC – NLO	DORL/LPL3/BC(A)	DORL/LPL3/PM	
NAME	VHoang	JWiebe	RKuntz	
DATE	2/6/17	2/27/17	2/27/17	

OFFICIAL RECORD COPY