



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 6, 2017

Mr. Peter P. Sena, III
President and Chief Nuclear Officer
PSEG Nuclear LLC - N09
P.O. Box 236
Hancocks Bridge, NJ 08038

**SUBJECT: HOPE CREEK GENERATING STATION AND SALEM NUCLEAR
GENERATING STATION, UNIT NOS. 1 AND 2 – ISSUANCE OF
AMENDMENTS RE: CHANGES TO THE TECHNICAL SPECIFICATIONS
TO REMOVE CERTAIN TRAINING PROGRAM REQUIREMENTS REDUNDANT
TO, OR SUPERSEDED BY, THE REGULATIONS IN 10 CFR PART 55 AND
10 CFR 50.120 (CAC NOS. MF8477, MF8478, AND MF8479)**

Dear Mr. Sena:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment Nos. 201, 317, and 298 to Renewed Facility Operating License Nos. NPF-57, DPR-70, and DPR-75 for the Hope Creek Generating Station and the Salem Nuclear Generating Station, Unit Nos. 1 and 2, respectively. These amendments consist of changes to the technical specifications (TSs) in response to your application dated October 17, 2016, as supplemented by letter dated December 19, 2016.

The amendments revise the TSs by removing certain training program requirements. Specifically, the amendments remove TS requirements that are redundant to, or superseded by, the requirements contained in Title 10 of the *Code of Federal Regulations* (10 CFR) Part 55 and 10 CFR 50.120.

P. Sena, III

- 2 -

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, appearing to read "Carleen J. Parker". The signature is written in a cursive style with a long horizontal line extending to the right.

Carleen J. Parker, Project Manager
Plant Licensing Branch I
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-354, 50-272, and 50-311

Enclosures:

1. Amendment No. 201 to Renewed NPF-57
2. Amendment No. 317 to Renewed DPR-70
3. Amendment No. 298 to Renewed DPR-75
4. Safety Evaluation

cc w/enclosures: Distribution via Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PSEG NUCLEAR LLC

DOCKET NO. 50-354

HOPE CREEK GENERATING STATION

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 201
Renewed License No. NPF-57

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by PSEG Nuclear LLC, dated October 17, 2016, as supplemented by letter dated December 19, 2016, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-57 is hereby amended to read as follows:

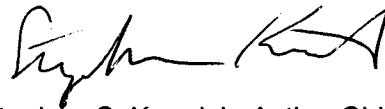
Enclosure 1

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 201, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Stephen S. Koenick, Acting Chief
Plant Licensing Branch I
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed License
and Technical Specifications

Date of Issuance: February 6, 2017

ATTACHMENT TO LICENSE AMENDMENT NO. 201

HOPE CREEK GENERATING STATION

RENEWED FACILITY OPERATING LICENSE NO. NPF-57

DOCKET NO. 50-354

Replace the following page of the Renewed Facility Operating License with the revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove
3

Insert
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Replace the following page of the Appendix A Technical Specifications with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove
6-6

Insert
6.6

reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;

- (4) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (5) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (6) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility. Mechanical disassembly of the GE14i isotope test assemblies containing Cobalt-60 is not considered separation.
- (7) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Part 30, to intentionally produce, possess, receive, transfer, and use Cobalt-60.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

PSEG Nuclear LLC is authorized to operate the facility at reactor core power levels not in excess of 3840 megawatts thermal (100 percent rated power) in accordance with the conditions specified herein.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 201, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

ADMINISTRATIVE CONTROLS

6.2.3 SHIFT TECHNICAL ADVISOR

6.2.3.1 The Shift Technical Advisor shall provide advisory technical support to the Senior Nuclear Shift Supervisor in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to safe operation of the unit. The Shift Technical Advisor shall have a bachelor's degree or equivalent in a scientific or engineering discipline and shall have received specific training in the response and analysis of the unit for transients and accidents, and in unit design and layout, including the capabilities of instrumentation and controls in the control room.

6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1981 for comparable positions, except for the individual designated as the Radiation Protection Manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, the individual designated as the operations manager who shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1981 except as modified by Specification 6.3.3, and the licensed operators who shall comply with the requirements of 10 CFR Part 55.

6.3.2 The Operations Manager or Assistant Operations Manager shall hold a senior reactor operator license. The Senior Nuclear Shift Supervisors, and Nuclear Shift Supervisors, shall hold a senior reactor operator license. The Nuclear Control Operators shall hold a reactor operator license.

6.3.3 The Operations Manager shall meet one of the following:

- (1) Hold a senior reactor operator license, or
- (2) Have held a senior reactor operator license for this or a similar unit (BWR), or
- (3) Have been certified at an appropriate simulator for equivalent senior operator knowledge.

6.4 TRAINING

6.4.1 DELETED

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Fire Protection Manager and shall meet or exceed the requirements of the SRP (NUREG-0800) Section 13.2.2.II.6, 10 CFR 50 Appendix R and Branch Technical Position CMEB 9.5.1, Section C.3.d.

6.5 REVIEW AND AUDIT (THIS SECTION DELETED)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PSEG NUCLEAR LLC

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 317
Renewed License No. DPR-70

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by PSEG Nuclear LLC, acting on behalf of itself and Exelon Generation Company, LLC (the licensees), dated October 17, 2016, as supplemented by letter dated December 19, 2016, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR, Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-70 is hereby amended to read as follows:

Enclosure 2

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 317, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications, and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Stephen S. Koenick, Acting Chief
Plant Licensing Branch I
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed License
and Technical Specifications

Date of Issuance: February 6, 2017

ATTACHMENT TO LICENSE AMENDMENT NO. 317
SALEM NUCLEAR GENERATING STATION, UNIT NO. 1
RENEWED FACILITY OPERATING LICENSE NO. DPR-70
DOCKET NO. 50-272

Replace the following page of Renewed Facility Operating License No. DPR-70 with the attached revised page as indicated. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove
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Insert
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Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages as indicated. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove
XVIII
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Insert
XVIII
6-7

instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

- (5) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (6) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

PSEG Nuclear LLC is authorized to operate the facility at a steady state reactor core power level not in excess of 3459 megawatts (one hundred percent of rated core power).

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 317, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications, and the Environmental Protection Plan.

(3) Deleted Per Amendment 22, 11-20-79

(4) Less than Four Loop Operation

PSEG Nuclear LLC shall not operate the reactor at power levels above P-7 (as defined in Table 3.3-1 of Specification 3.3.1.1 of Appendix A to this renewed license) with less than four (4) reactor coolant loops in operation until safety analyses for less than four loop operation have been submitted by the licensees and approval for less than four loop operation at power levels above P-7 has been granted by the Commission by Amendment of this renewed license.

(5) PSEG Nuclear LLC shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety

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ADMINISTRATIVE CONTROLS

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ADMINISTRATIVE CONTROLS

6.2.3 SHIFT TECHNICAL ADVISOR

6.2.3.1 The Shift Technical Advisor shall serve in an advisory capacity to the Senior Nuclear Shift Supervisor on matters pertaining to the engineering aspects assuring safe operation of the unit.

6.2.3.2 The Shift Technical Advisor shall have a Bachelor's Degree or equivalent in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents.

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the individual designated as the Radiation Protection Manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, the individual designated as the Operations Manager who shall meet or exceed the minimum qualifications of ANSI N18.1-1971 except as modified by Specification 6.3.3, and the licensed operators who shall comply with the requirements of 10CFR55.

6.3.2 The Operations Manager or Assistant Operations Manager shall hold an SRO license. The Senior Nuclear Shift Supervisors and Nuclear Shift Supervisors shall each hold a senior reactor operator license. The Nuclear Control Operators shall hold reactor operator licenses.

6.3.3 The Operations Manager shall meet one of the following:

- 1) Hold an SRO license, or
- 2) Have held an SRO license for a similar unit (PWR), or
- 3) Have been certified at an appropriate simulator for equivalent senior operator knowledge.

6.4 DELETED



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PSEG NUCLEAR LLC

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 298
Renewed License No. DPR-75

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by PSEG Nuclear LLC, acting on behalf of itself and Exelon Generation Company, LLC (the licensees), dated October 17, 2016, as supplemented by letter dated December 19, 2016, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-75 is hereby amended to read as follows:

Enclosure 3

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 298, and the environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Stephen S. Koenick, Acting Chief
Plant Licensing Branch I
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed License
and Technical Specifications

Date of Issuance: February 6, 2017

ATTACHMENT TO LICENSE AMENDMENT NO. 298
SALEM NUCLEAR GENERATING STATION, UNIT NO. 2
RENEWED FACILITY OPERATING LICENSE NO. DPR-75
DOCKET NO. 50-311

Replace the following page of Renewed Facility Operating License No. DPR-75 with the attached revised page as indicated. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove
3

Insert
3

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages as indicated. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove
XVIII
6-7

Insert
XVIII
6-7

- (4) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use at any time any byproduct, source or special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration and as fission detectors in amounts as required;
 - (5) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
 - (6) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

PSEG Nuclear LLC is authorized to operate the facility at steady state reactor core power levels not in excess of 3459 megawatts (thermal).
 - (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 298, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

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ADMINISTRATIVE CONTROLS

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ADMINISTRATIVE CONTROLS

6.2.3 SHIFT TECHNICAL ADVISOR

6.2.3.1 The Shift Technical Advisor shall serve in an advisory capacity to the Senior Nuclear Shift Supervisor on matters pertaining to the engineering aspects assuring safe operation of the unit.

6.2.3.2 The Shift Technical Advisor shall have a Bachelor's Degree or equivalent in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents.

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the individual designated as the Radiation Protection Manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, the individual designated as the Operations Manager who shall meet or exceed the minimum qualifications of ANSI N18.1-1971 except as modified by Specification 6.3.3, and the licensed operators who shall comply with the requirements of 10CFR55.

6.3.2 The Operations Manager or Assistant Operations Manager shall hold an SRO license. The Senior Nuclear Shift Supervisors and Nuclear Shift Supervisors shall each hold a senior reactor operator license. The Nuclear Control Operators shall hold reactor operator licenses.

6.3.3 The Operations Manager shall meet one of the following:

- 1) Hold an SRO license, or
- 2) Have held an SRO license for a similar unit (PWR), or
- 3) Have been certified at an appropriate simulator for equivalent senior operator knowledge.

6.4 DELETED



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NOS. 201, 317, AND 298 TO
RENEWED FACILITY OPERATING LICENSE NOS. NPF-57, DPR-70, AND DPR-75
PSEG NUCLEAR LLC
HOPE CREEK GENERATING STATION
PSEG NUCLEAR LLC
EXELON GENERATION COMPANY, LLC
SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2
DOCKET NOS. 50-354, 50-272, AND 50-311

1.0 INTRODUCTION

By letter dated October 17, 2016,¹ as supplemented by letter dated December 19, 2016,² PSEG Nuclear LLC (PSEG or the licensee) submitted a license amendment request (LAR) for changes to the Hope Creek Generating Station (Hope Creek) and Salem Nuclear Generating Station (Salem), Unit Nos. 1 and 2, Technical Specifications (TSs). The requested changes would revise the TSs by removing certain training program requirements. Specifically, the amendments would remove TS requirements that are redundant to, or superseded by, the requirements contained in Title 10 of the *Code of Federal Regulations* (10 CFR) Part 55 and 10 CFR 50.120.

The current TS requirements specify that the retraining and replacement training programs shall be maintained under the direction of the Director – Nuclear Training, and that the program shall comply with the requirements of 10 CFR Part 55 for licensed operators. Additionally, the current TS requirements specify that the retraining and replacement training programs meet or exceed the requirements and recommendations of Section 5.5 of ANSI/ANS 3.1-1981, “American National Standard for Selection, Qualification, and Training of Personnel for Nuclear Power Plants,” for Hope Creek and Section 5.5 of ANSI N18.1-1971, “Selection and Training of Nuclear Power Plant Personnel,” for Salem, Unit Nos. 1 and 2. The proposed revision would delete these requirements, which are contained in Hope Creek TS Section 6.4.1 and Salem, Unit Nos. 1 and 2, TS Section 6.4, “Training.”

The supplement dated December 19, 2016, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the U.S. Nuclear Regulatory Commission (NRC or the Commission) staff’s original proposed no

¹ Agencywide Documents Access and Management System (ADAMS) Accession No. ML16291A318

² ADAMS Accession No. ML16354A866

significant hazards consideration determination as published in the *Federal Register* on November 22, 2016 (81 FR 83877).

2.0 REGULATORY EVALUATION

The NRC staff reviewed the proposed TS changes against the regulatory requirements and guidance listed below.

2.1 Applicable TS Regulations

The Commission's regulatory requirements related to the content of the TSs are set forth in 10 CFR 50.36, "Technical specifications." This regulation requires that the TSs include items in the following categories: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. The regulation does not specify the particular requirements to be included in plant TSs.

In accordance with 10 CFR 50.36(c)(5), administrative controls are the provisions relating to organization and management, procedures, recordkeeping, review and audit, and reporting necessary to assure operation of the facility in a safe manner. At Hope Creek and Salem, administrative controls are located in Section 6.0 of the TSs. Salem, Unit Nos. 1 and 2, TS Section 6.4, and Hope Creek TS Section 6.4.1, provide the requirements that must be met for a retraining and replacement training program.

2.2 Applicable Training Requirement Regulations and Guidance

NRC regulation 10 CFR Part 55 contains the regulatory requirements for licensed operators at nuclear power plants, including requirements for their training and qualification. Regulatory Guide 1.8, "Qualification and Training of Personnel for Nuclear Power Plants,"³ contains specific guidance for complying with those portions of the Commission's regulations regarding the training and qualification of nuclear power plant personnel.

On March 20, 1985, the NRC issued a "Commission Policy Statement on Training and Qualification of Nuclear Power Plant Personnel" (50 FR 11147), that recognized industry's commitment to accredited training programs. In the policy statement, the NRC endorsed the training accreditation program managed by the Institute of Nuclear Power Operations (INPO) because it encompassed the elements of a systems approach to training (SAT). The five elements of an SAT were later codified in 10 CFR Part 55 as:

- (1) Systematic analysis of the jobs to be performed.
- (2) Learning objectives derived from the analysis which describe desired performance after training.
- (3) Training design and implementation based on the learning objectives.
- (4) Evaluation of trainee mastery of the objectives during training.
- (5) Evaluation and revision of the training based on the performance of trained personnel in the job setting.

³ ADAMS Accession No. ML003706932

In Generic Letter 87-07, "Information Transmittal of Final Rulemaking for Revisions to Operator Licensing – 10 CFR 55 and Conforming Amendments,"⁴ dated March 19, 1987, and in NUREG-1262, "Answers to Question at Public Meetings Regarding Implementation of Title 10, Code of Federal Regulations, Part 55 on Operators' Licenses," published November 1987, the NRC indicated it would accept a facility's licensed operator training program if the facility certified in writing that the program was accredited and based on an SAT process. Facility licensees were advised to submit a request to the NRC for an administrative change to their licensing documents to revise or delete, as appropriate, the requirements that had been superseded.

On January 18, 2001, the NRC published NRC Regulatory Issue Summary (RIS) 2001-01, "Eligibility of Operator License Applicants,"⁵ to familiarize licensees with the NRC's current guidelines for the qualification and training of licensed operator applicants. The RIS also encouraged licensees to review their requirements with regard to licensed operator training and experience and update their documentation (including TSs) to enhance consistency and minimize confusion.

On April 26, 1993 (58 FR 21904), the NRC published its final rule, 10 CFR 50.120, "Training and qualification of nuclear power plant personnel." The rule requires nuclear power plant licensees to establish, implement, and maintain SAT-based training programs for nine non-licensed positions. Accreditation of these nine training programs is an acceptable means of meeting the requirements of 10 CFR 50.120.

2.3 Quality Assurance Program Requirements

In accordance with 10 CFR 50.34(b)(6)(ii), the Final Safety Analyses Report (FSAR) shall include managerial and administrative controls to be used to assure safe operation. Appendix B to 10 CFR Part 50, "Quality Assurance Criteria for Nuclear Power Plants and Fuel Reprocessing Plants," sets forth the requirements for such controls. Implementation of and changes to the quality assurance program described in the FSAR are controlled in accordance with the requirements of 10 CFR 50.54(a).

3.0 TECHNICAL EVALUATION

3.1 Proposed TS Changes

The proposed changes would remove TS requirements that are redundant to, or superseded by, the requirements contained in 10 CFR Part 55 and 10 CFR 50.120.

Salem, Unit Nos. 1 and 2, TS Section 6.4 currently states:

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall:

⁴ ADAMS Accession No. ML031150669

⁵ ADAMS Accession No. ML003733003

- 1) be coordinated by each functional level manager (Department Head) at the facility and maintained under the direction of the Director – Nuclear Training
- 2) meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 for all affected positions except licensed operators and
- 3) comply with the requirements of 10CFR55 for licensed operators.

Administrative TS Section 6.4 would be deleted in its entirety from Salem, Unit Nos. 1 and 2. The TS Index for Salem, Unit Nos. 1 and 2, would be revised to reflect this.

Hope Creek TS 6.4.1 currently states:

- 6.4.1 A retraining and replacement training program for the unit staff shall:
- (1) be maintained under the direction of the Director – Nuclear Training, (2) shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI/ANS 3.1-1981 for all affected positions except licensed operators, and
 - (3) comply with the requirements of 10 CFR Part 55 for licensed operators.

Administrative TS Section 6.4.1 would be deleted in its entirety from Hope Creek. No changes to the Hope Creek TS Index are needed.

3.2 NRC Staff Evaluation

3.2.1 Retraining and Replacement Training Program – Licensed Operators

The proposed TS change will delete the redundant licensed operator qualification requirements that are specified in the administrative controls section of the Hope Creek and Salem, Unit Nos. 1 and 2, TSs. The current TSs require Hope Creek and Salem to comply with 10 CFR Part 55 for licensed operators. The regulations in 10 CFR Part 55 provide the necessary requirements for the training and qualification of licensed operators. Hope Creek and Salem, Unit Nos. 1 and 2, are required to comply with the requirements contained in 10 CFR Part 55, unless an exemption is granted by the NRC, regardless of whether the requirements are contained in the Hope Creek and Salem, Unit Nos. 1 and 2, TSs. Therefore, the requirements for licensed operators that are located in the administrative controls section of the Hope Creek and Salem, Unit Nos. 1 and 2, TSs are redundant to the requirements located in 10 CFR Part 55.

Based on the information above, the NRC staff has determined that the licensed operator training requirements may be deleted from the administrative controls section of the Hope Creek and Salem, Unit Nos. 1 and 2, TSs.

3.2.2 Retraining and Replacement Training Program – Non-Licensed Operators

In the LAR, the licensee states that the Hope Creek and Salem, Unit Nos. 1 and 2, training programs for the nine non-licensed operator categories listed in 10 CFR 50.120 are based on the SAT process. In addition, the Hope Creek and Salem, Unit Nos. 1 and 2, non-licensed staff training programs are accredited by the National Academy for Nuclear Training. Therefore, the training programs are in compliance with 10 CFR 50.120.

For Hope Creek, this request would remove the requirements for conducting retraining for individuals identified in Section 5.5 of ANSI/ANS-3.1-1981.⁶ Section 5.5 of ANSI/ANS-3.1-1981 states, in part, that, "Mechanisms shall be established to ensure that individuals in the operating organization performing safety related functions remain cognizant of changes to the facility, procedures, governmental regulations, and quality assurance requirements...." Quality Assurance personnel are listed in Section 4.4.5 of ANSI/ANS-3.1-1981 as requiring training consistent with other professional technical personnel, which are listed in 10 CFR 50.120. The quality assurance personnel category is not specified in 10 CFR 50.120 as requiring a SAT-based training program; therefore, 10 CFR 50.120 is not fully redundant to the Hope Creek TS administrative controls Section 6.4.1. In its letter dated December 19, 2016, the licensee stated that the PSEG Quality Assurance Program (QAP) provides training requirements for personnel performing or verifying activities in the scope of the QAP, which are more comprehensive than the training required by the current Hope Creek TS Section 6.4.1. PSEG provides a list of currently required qualification and training criteria for quality assurance personnel in its letter. Implementation of, and any changes to, the QAP are controlled in accordance with the requirements contained in 10 CFR 50.54(a). Therefore, the requirements in Hope Creek TS Section 6.4.1 for non-licensed personnel are redundant to the requirements contained in 10 CFR 50.54(a) and 10 CFR 50.120.

The Salem, Unit Nos. 1 and 2, TSs reference Section 5.5 of ANSI N18.1-1971 for all affected positions except licensed operators. ANSI N18.1-1971 does not include a category of personnel beyond the nine non-licensed categories in 10 CFR 50.120; therefore, the requirements in Salem TS Section 6.4 for non-licensed personnel are redundant to the requirements contained in 10 CFR 50.120.

Based on the information above, the NRC staff has determined that the retraining and training replacement program requirements for non-licensed staff may be deleted from the administrative controls section of the Hope Creek and Salem, Unit Nos. 1 and 2, TSs.

3.2.3 Retraining and Replacement Training Program – Organization

The proposed TS change will delete the organizational requirement that the retraining and replacement training program be maintained under the Director – Nuclear Training and coordinated by each functional level manager (Salem, Unit Nos. 1 and 2, only) from the administrative controls section of the Hope Creek and Salem, Unit Nos. 1 and 2, TSs. The current TSs require Hope Creek and Salem, Units 1 and 2, to comply with TS Section 6.2.1, "Onsite and Offsite Organizations," which requires the licensee to establish lines of authority and responsibility from the highest management levels through intermediate levels which includes all operating organization positions. Therefore, the requirement specifying that the retraining and replacement training program shall be maintained under the Director – Nuclear Training and coordinated by each functional level manager (Salem, Unit Nos. 1 and 2, only) is redundant to the requirements specified in TS Section 6.2.1.

Based on the information above, the NRC staff has determined that the retraining and training replacement program organizational requirements may be deleted from the administrative controls section of the Hope Creek and Salem, Unit Nos. 1 and 2, TSs.

⁶ Salem's TSs reference ANSI N18.1-1971

3.2.4 NRC Staff Conclusion

The NRC staff concludes that deleting the retraining and replacement training program requirements for licensed operators and non-licensed staff from the administrative controls section of the TSs is consistent with 10 CFR Part 55, 10 CFR 50.120 (and 10 CFR 50.54(a) for Hope Creek), and TS Section 6.2.1, does not adversely affect nuclear safety or plant operation, and provides adequate requirements to assure an acceptable and competent operating staff. In addition, the NRC staff concludes the TSs, as revised, continue to meet the requirements of 10 CFR 50.36.

Deleting administrative controls TS Section 6.4 from Salem, Unit Nos. 1 and 2, and Section 6.4.1 from Hope Creek eliminates any confusion regarding standards and expectations for training by removing the redundant or superseded requirements. Therefore, the NRC staff finds the proposed changes acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments relate to changes in record keeping, reporting, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: T. Kolb

Date: February 6, 2017

SUBJECT: HOPE CREEK GENERATING STATION AND SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2 – ISSUANCE OF AMENDMENTS RE: CHANGES TO THE TECHNICAL SPECIFICATIONS TO REMOVE CERTAIN TRAINING PROGRAM REQUIREMENTS REDUNDANT TO, OR SUPERSEDED BY, THE REGULATIONS IN 10 CFR PART 55 AND 10 CFR 50.120 (CAC NOS. MF8477, MF8478, AND MF8479) DATED FEBRUARY 6, 2017

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