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10 CFR 52.99(c)(1)

Southern Nuclear Operating Company
Vogtle Electric Generating Plant Unit 4
ITAAC Closure Notification on Completion of ITAAC 2.3.13.05.i [Index Number 462]

Ladies and Gentlemen:

In accordance with 10 CFR 52.99(c)(1), the purpose of this letter is to notify the Nuclear Regulatory Commission (NRC) of the completion of Vogtle Electric Generating Plant (VEGP) Unit 4 Inspections, Tests, Analyses, and Acceptance Criteria (ITAAC) Item 2.3.13.05.i [Index Number 462] for verifying that an inspection was performed and concludes that the seismic Category I equipment and valves identified in VEGP Unit 4 Combined License (COL) Appendix C, Table 2.3.13-1 are located on the Nuclear Island. The closure process for this ITAAC is based on the guidance described in NEI 08-01, "Industry Guideline for the ITAAC Closure Process under 10 CFR Part 52," which was endorsed by the NRC in Regulatory Guide 1.215.

This letter contains no new NRC regulatory commitments. Southern Nuclear Operating Company (SNC) requests NRC staff confirmation of this determination and publication of the required notice in the Federal Register per 10 CFR 52.99.

If there are any questions, please contact David Woods at 706-848-6903.

Respectfully submitted,

Michael J. Yox
Regulatory Affairs Director Vogtle 3&4

Enclosure: Vogtle Electric Generating Plant (VEGP) Unit 4
Completion of ITAAC 2.3.13.05.i [Index Number 462]

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U.S. Nuclear Regulatory Commission

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**Southern Nuclear Operating Company
ND-16-2770
Enclosure**

**Vogtle Electric Generating Plant (VEGP) Unit 4
Completion of ITAAC 2.3.13.05.i [Index Number 462]**

ITAAC Statement

Design Commitment:

5. The seismic Category I equipment identified in Table 2.3.13-1 can withstand seismic design basis loads without loss of its safety function.

Inspections, Tests, Analysis:

- i) Inspection will be performed to verify that the seismic Category I equipment and valves identified in Table 2.3.13-1 are located on the Nuclear Island.

Acceptance Criteria:

- i) The seismic Category I equipment identified in Table 2.3.13-1 is located on the Nuclear Island.

ITAAC Determination Basis

Multiple ITAAC are performed to demonstrate that the equipment identified in VEGP Unit 4 Combined License (COL) Appendix C, Table 2.3.13-1 can withstand seismic design basis loads without loss of safety function.

This ITAAC requires an inspection be performed to verify that the Primary Sampling System (PSS) equipment (valves) identified in Table 2.3.13-1 (Attachment A) are located on the Nuclear Island, which is a Seismic Category I structure. Subsequent ITAAC 2.3.13.05.ii (Index Number 463) will verify that the equipment is seismically qualified and ITAAC 2.3.13.05.iii (Index Number 464) will confirm that the equipment is installed and seismically bounded for the as-built location (i.e., Nuclear Island). Completion of these multiple ITAAC will confirm the equipment can withstand seismic design basis loads without loss of its safety function.

The piping isometric drawings (References 4-11) which are issued for construction for each valve were visually inspected and each valve listed in Attachment A was verified to be located on the nuclear island per design. The location of the piping isometric drawings which contains the valves was compared to the Nuclear Island General Arrangement Plan at Elevation 117' 6", SV4-1040-P2-001 (Reference 3) and were verified to be located within the bounds of the column lines shown on the Nuclear Island General Arrangement Plan, thereby confirming that the valves in Attachment A are located on the Nuclear Island.

The results of the inspections are documented in the Inspection Report (Reference 1) and conclude that the seismic Category I equipment identified in Table 2.3.13-1 is located on the Nuclear Island.

ITAAC Finding Review

In accordance with plant procedures for ITAAC completion, Southern Nuclear Operating Company (SNC) performed a review of all ITAAC findings pertaining to the subject ITAAC and associated corrective actions. This review found that there are no relevant ITAAC findings associated with this ITAAC. The ITAAC completion review document number is included in the Vogtle Unit 4 ITAAC Completion Package for ITAAC 2.3.13.05.i (Reference 2) and available for NRC inspection.

ITAAC Completion Statement

Based on the above information, SNC hereby notifies the NRC that ITAAC 2.3.13.05.i was performed for VEGP Unit 4 and that the prescribed acceptance criteria are met.

Systems, structures, and components verified as part of this ITAAC are being maintained in their as-designed, ITAAC compliant condition in accordance with approved plant programs and procedures.

References (available for NRC inspection)

1. SV4-PSS-ITR-001, Revision 0, "Inspection Report Confirming Primary Sampling System (PSS) Seismic Category I Equipment is Located on the Nuclear Island, ITAAC 2.3.13.05.i"
2. SVP_SV0_004517, Attachment 1, "Submittal of Inspections, Test, Analyses and Acceptance Criteria (ITAAC) Completion Package for Unit 4 ITAAC 2.3.13.05.i [COL Index Number 462] (PSS System Seismic Category I Equipment Location)"
3. SV4-1040-P2-001, Revision 0, "Nuclear Island General Arrangement Plan at Elevation 117'-6"
4. SV4-PSS-PLW-710, Revision 0, "Primary Sampling System Cont Bldg Rm 11400/Aux Bldg Rm 12341 Cont. Atmosphere Sample Isolation"
5. SV4-PSS-PLW-662, Revision 0, "Primary Sampling System Cont Bldg Rm 11400/Aux Bldg Rm 12341 CV Penetration C04 Isolation"
6. SV4-PSS-PLW-66E, Revision 0, "Primary Sampling System Cont Bldg Rm 11400/Aux Bldg Rm 12341 CV Penetration C0I Isolation"
7. SV4-PSS-PLW-687, Revision 0, "Primary Sampling System Auxiliary Building Room 12454 Liquid Sample A to SCR (PSS-MS-02)"
8. SV4-PSS-PLW-518, Revision 0, "Primary Sampling System Auxiliary Building Room 12454 Liquid Sample B to SCR (PSS-MS-02)"
9. SV4-PSS-PLW-530, Revision 2, "Primary Sampling System Auxiliary Building Rooms 12341/12454 Containment Atmosphere Sample Return"

10. SV4-PSS-PLW-630, Revision 0, "Primary Sampling System Containment Bldg Room 11400 Containment Sampling Return Line"
11. SV4-PSS-PLW-520, Revision 0, "Primary Sampling System Auxiliary Building Rooms 12341/12454 Containment Atmosphere Sample Supply"

Attachment A

SYSTEM: Primary Sampling System (PSS)

Excerpt from COL Appendix C Table 2.3.13-1*

Equipment Name*	Tag No.*	Seismic Cat. I*	Isometric Drawing
Containment Air Sample Containment Isolation Valve Inside Reactor Containment (IRC)	PSS-PL-V008	Yes	SV4-PSS-PLW-710 (Reference 4)
Liquid Sample Line Containment Isolation Valve IRC	PSS-PL-V010A	Yes	SV4-PSS-PLW-662 (Reference 5)
Liquid Sample Line Containment Isolation Valve IRC	PSS-PL-V010B	Yes	SV4-PSS-PLW-66E (Reference 6)
Liquid Sample Line Containment Isolation Valve Outside Reactor Containment (ORC)	PSS-PL-V011A	Yes	SV4-PSS-PLW-687 (Reference 7)
Liquid Sample Line Containment Isolation Valve ORC	PSS-PL-V011B	Yes	SV4-PSS-PLW-518 (Reference 8)
Sample Return Line Containment Isolation Valve ORC	PSS-PL-V023	Yes	SV4-PSS-PLW-530 (Reference 9)
Sample Return Containment Isolation Valve IRC	PSS-PL-V024	Yes	SV4-PSS-PLW-630 (Reference 10)
Air Sample Line Containment Isolation Valve ORC	PSS-PL-V046	Yes	SV4-PSS-PLW-520 (Reference 11)