December 2, 1997

Mr. Gregory M. Rueger, Senior Vice President
and General Manager
Pacific Gas and Electric Company
Nuclear Power Generation N9B
P. O. Box 770000
San Francisco, California 94177

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - WCAP-14707, "MODEL 51 STEAM GENERATOR LIMITED TUBE SUPPORT PLATE DISPLACEMENT ANALYSIS FOR DENTED OR PACKED TUBE TO TUBE SUPPORT PLATE CREVICES" (TAC NOS. M99011 AND M99012)

Dear Mr. Rueger:

By letter dated October 4, 1996, Pacific Gas and Electric Company (PG&E) submitted Westinghouse Electric Corporation (Westinghouse) technical reports WCAP-14707 (proprietary) and WCAP-14708 (nonproprietary), "Model 51 Steam Generator Limited Tube Support Plate Displacement Analysis For Dented or Packed Tube To Tube Support Plate Crevices." These reports describe the results of work performed to assess the potential for tube support plate (TSP) displacement in a postulated main steam line break event for a generic Model 51 steam generator (SG) with dented or packed tube-to-TSP crevices with TSP corrosion. The NRC has performed an initial review, which focused on the results from Section 4.0, "Axial Pull Force Tests for TSP Displacement and Leakage Tests with Dented or Packed TSP Intersections," and the connection between these results and the conclusion drawn in Section 2.0, "Summary and Conclusions." In order to complete our review the staff requires the additional information described in the enclosure. Please submit your response within 30 days from receipt of this letter.

Sincerely,

Original Signed By
Steven D. Bloom, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-275
and 50-323

Enclosure: Request for Additional Information

cc w/encl: See next page

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cc w/encl:
NRC Resident Inspector
Diablo Canyon Nuclear Power Plant
c/o U.S. Nuclear Regulatory Commission
P. O. Box 369
Avila Beach, California 93424

Dr. Richard Ferguson, Energy Chair
Sierra Club California
1100 11th Street, Suite 311
Sacramento, California 95814

Ms. Nancy Culver
San Luis Obispo
Mothers for Peace
P. O. Box 164
Pismo Beach, California 93448

Chairman
San Luis Obispo County Board of
Supervisors
Room 370
County Government Center
San Luis Obispo, California 93408

Mr. Truman Burns
Mr. Robert Kinosian
California Public Utilities Commission
505 Van Ness, Room 4102
San Francisco, California 94102

Mr. Steve Hsu
Radiologic Health Branch
State Department of Health Services
Post Office Box 942732
Sacramento, California 94232

Diablo Canyon Independent Safety
Committee
ATTN: Robert R. Wellington, Esq.
Legal Counsel
857 Cass Street, Suite D
Monterey, California 93940

Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
Harris Tower & Pavillion
611 Ryan Plaza Drive, Suite 400
Arlington, Texas 76011-8064

Christopher J. Warner, Esq.
Pacific Gas & Electric Company
Post Office Box 7442
San Francisco, California 94120

Mr. Robert P. Powers
Vice President and Plant Manager
Diablo Canyon Nuclear Power Plant
P. O. Box 56
Avila Beach, California 93424

Telegram-Tribune
ATTN: Managing Editor
1321 Johnson Avenue
P.O. Box 112
San Luis Obispo, California 93406

ENCLOSURE

REQUEST FOR ADDITIONAL INFORMATION

PACIFIC GAS AND ELECTRIC COMPANY

DIABLO CANYON POWER PLANT UNITS 1 AND 2

DOCKET NOS. 50-275 AND 50-323

1. Provide References 1, 2, 3 and 4 for staff review.

2. Provide the technical basis for concluding that the Dampierre-1 test results (i.e., pull force and leak rate testing, before and after chemical cleaning) are applicable to all TSP intersections in all Model 51 steam generators despite potential differences in the TSP crevice conditions given differences in TSP location, SG service life, secondary side water chemistry, chemical cleaning process, etc.

3. Provide the technical basis for concluding the presence of "some denting" indicates all TSP intersections have "packed crevices developing high forces resisting SLB displacement and resulting in negligible leakage (p.2-1)."

4. Discuss how one can directly verify, on a site-specific basis, that the TSP crevice conditions can be relied upon to ensure limited TSP displacement.

5. Discuss the methods used to ensure that all relevant worldwide data have been obtained.

6. In accordance with the staff's approach to risk informed regulation, changes in licensing basis are to be accompanied by an assessment of the associated potential for changes in risk. In view of future proposed changes to the design basis, address the implications for tube integrity under severe accident conditions. Specifically, address the effect due to defects left in service.