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Pacific Gas and Electric Company

Diablo Canyon Power Plant P.O. Box 56 Avila Beach, CA 93424 805/545-6000 Warren H. Fujimoto Vice President-Diablo Canyon Operations and Plant Manager



January 13, 1995

PG&E Letter DCL-95-005

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20555

Docket No. 50-275, DPR-80 Docket No. 50-323, DPR-82 Diablo Canyon Units 1 and 2 Monthly Operating Report for December 1994

Gentlemen:

Enclosed are the monthly operating report forms for Diablo Canyon Units 1 and 2 for December 1994. This report is submitted in accordance with Section 6.9.1.7 of the Units 1 and 2 Technical Specifications.

Sincerely,

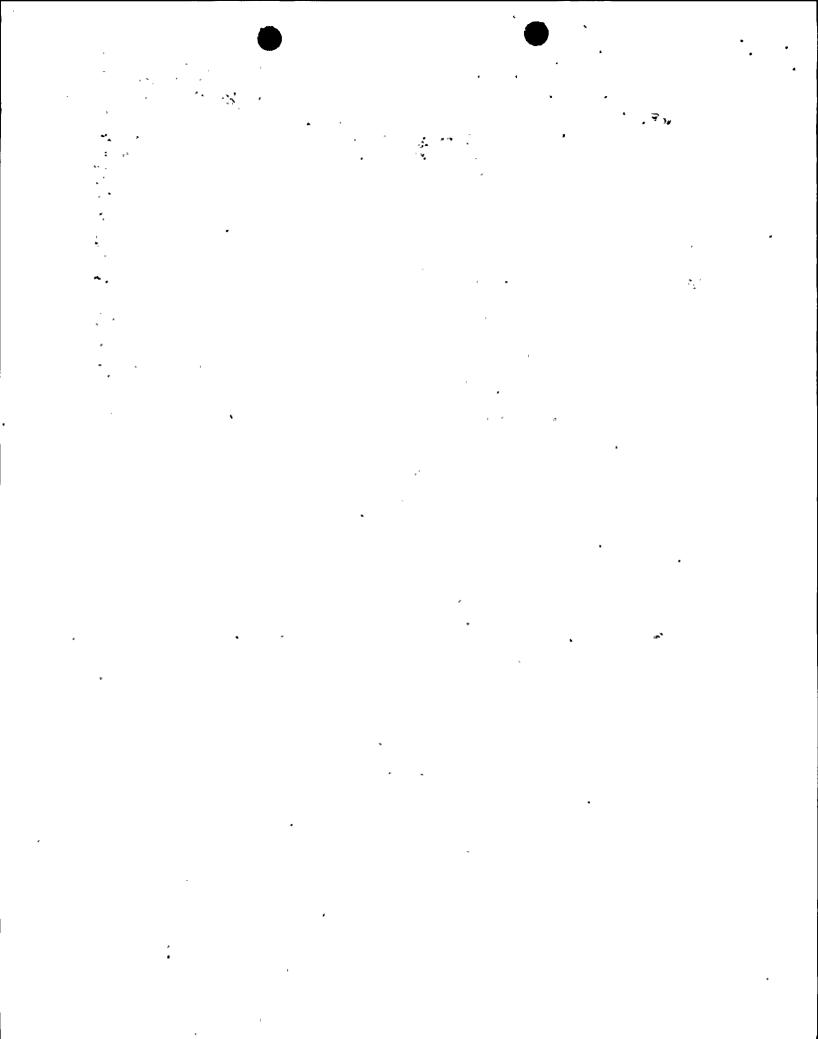
Warren H. Fujimoto

Enclosures

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cc: Mr. L. J. Callan, Regional Administrator
U.S. Nuclear Regulatory Commission, Region IV
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011-8064

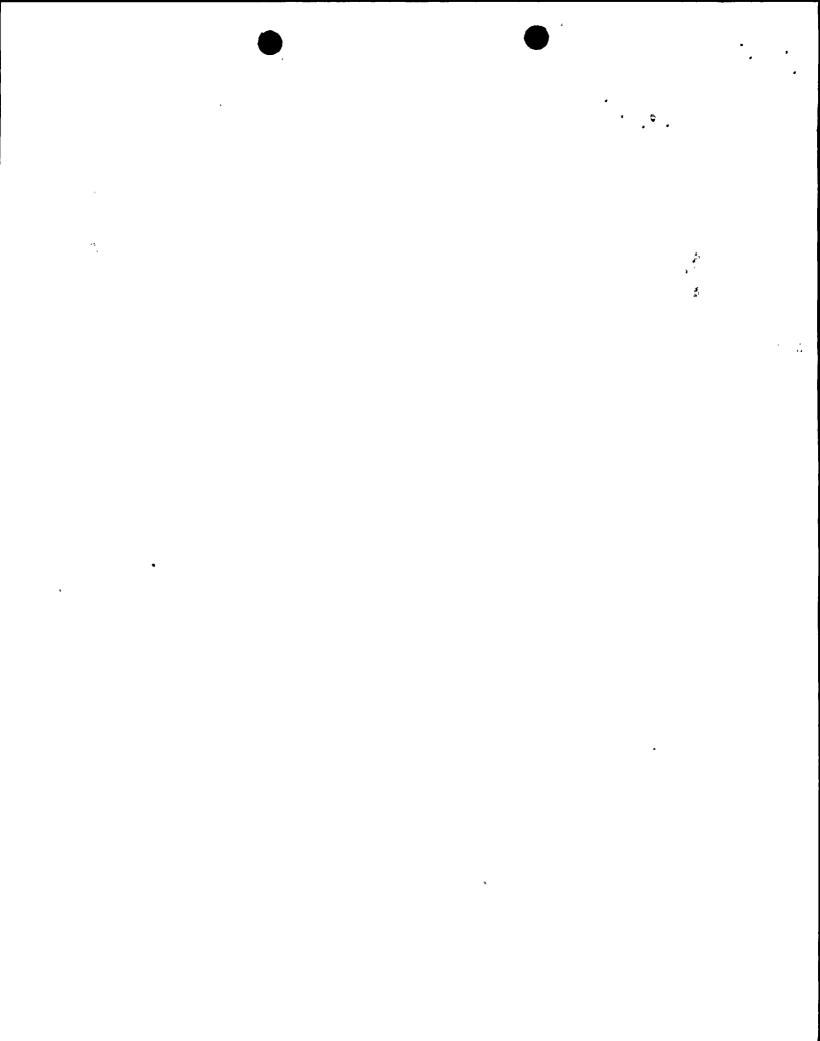
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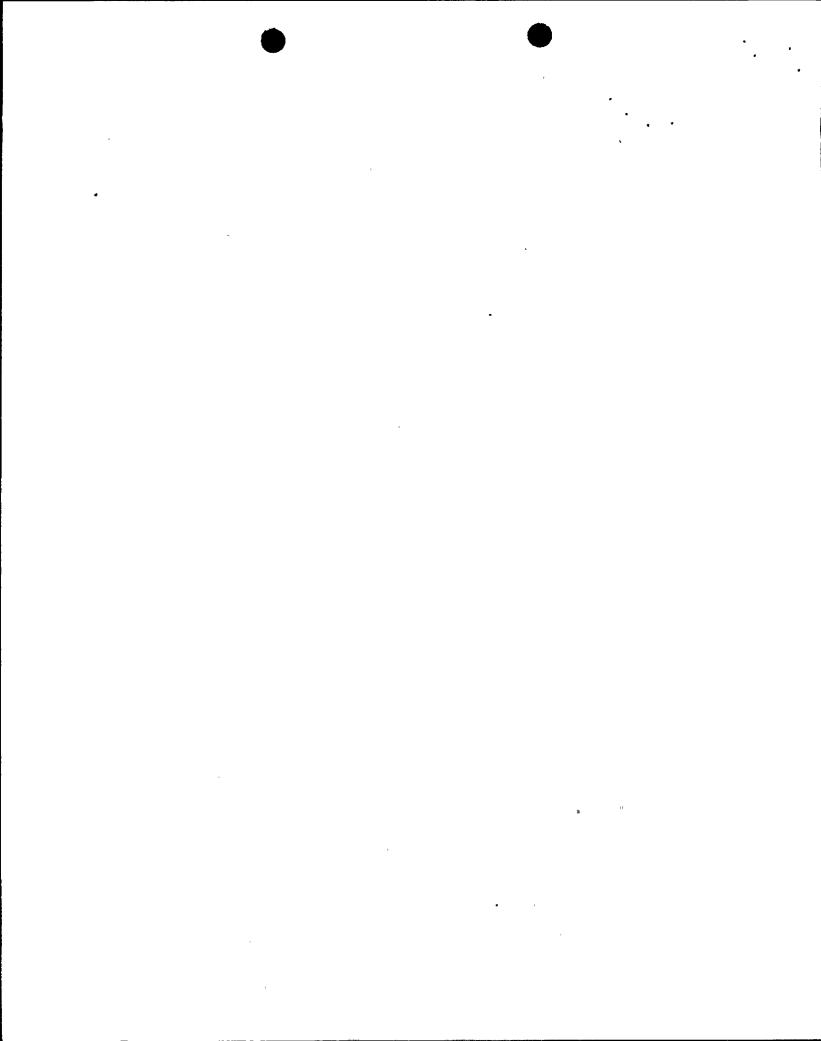
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MONTHLY NARRATIVE REPORT OF OPERATION AND MAJOR MAINTENANCE EXPERIENCE

This report describes the operating and major maintenance experience for the month of December 1994. This narrative report was prepared by the Plant Staff and is submitted in accordance with Section 6.9.1.7 of the Units 1 and 2 Technical Specifications.

Narrative of Daily Significant Plant Events

On December 1, 1994: Unit 1 and Unit 2 started the month in Mode 1 (Power Operation) at

100% power.

On December 8, 1994: A 10 CFR 50.72(b)(1)(v) one-hour, non-emergency report was made

due the loss of the normal emergency notification system (ENS) phone system for greater than one hour. Alternate means of communication was established with the NRC Operation Center during the period.

On December 14, 1994: A 10 CFR 50.72(b)(2)(ii) four-hour, non-emergency report was made

regarding an automatic reactor trip system (RTS) actuation for Units 1 and 2. The reactor trip was initiated by reactor coolant system pump undervoltage trip due to a western power pool transmission system voltage transient. Both Units 1 and 2 were stabilized in Mode 3 (Hot

Standby). For more information see LER 1-94-020.

On December 16, 1994: Unit 2 entered Mode 1, paralleled to the grid and increased power to

30%..

On December 17, 1994: Unit 2 increased power to 90%.

On December 18, 1994: Unit 1 entered Mode 1, paralleled to the grid and increased power to

30%. Unit 2 increased power to 100%.

On December 19, 1994: A 10 CFR 50.72(b)(2)(ii) four hour, non-emergency report was made

regarding a Unit 2 manual unit trip due to high circulating water differential pressure resulting from kelp buildup on the debris screens. For more information see LER 2-94-012. Unit 1 increased power to

50%.

On December 20, 1994: An Unusual Event (UE) was declared at 0245 PST due to detection of

an earthquake by plant personnel and seismic monitoring equipment. The earthquake was determined to have been a 5.0 Richter magnitude earthquake occurring approximately 50 miles to the Northeast of the plant site. The UE was terminated at 0349 PST with no adverse consequences resulting for Units 1 and 2. Unit 1 increased power to

100%.

On December 24, 1994: Unit 2 entered Mode 1 and paralleled to the grid.

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On December 25, 1994: Unit 2 increased power to 35%

On December 26, 1994: Unit 2 increased power to 64%.

On December 27, 1994: Unit 2 increased power to 100%.

On December 31, 1994: Unit 1 ended the month in Mode 1 (Power Operation) at 100% power.

Unit 2 ended the month in Mode 1 at 100% power.

Summary of Plant Operating Characteristics, Power Reductions and Unit Shutdowns

Unit 1 operated this month with a unit availability factor of 86.71% and a unit capacity factor (using MDC Net) of 83.03%. Unit 1 reduced power once by more than 20% for more than four hours this month due to an automatic reactor trip caused by a western power grid system voltage disturbance.

Unit 2 operated this month with a unit availability factor of 74.31% and a unit capacity factor (using MDC Net) of 66.42%. Unit 2 reduced power twice by more than 20% for more than four hours this month. Once due to an automatic reactor trip caused by a western power grid system voltage disturbance and once due to heavy Pacific Ocean swells resulting in heavy kelp loading of the seawater intake screens.

Summary of Significant Safety Related Maintenance

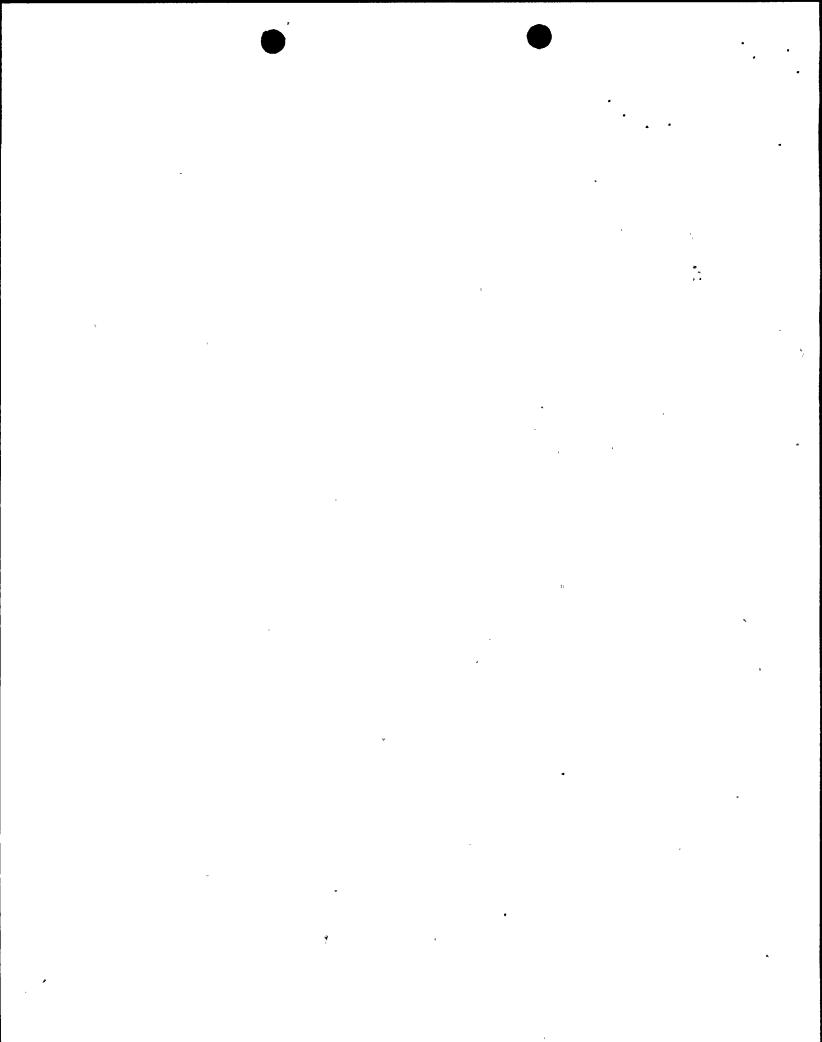
There was no significant safety-related maintenance for Unit 1 during the month of December.

There was no significant safety-related maintenance for Unit 2 during the month of December.

Actuation of Steam Generator Safety or Pressurizer Power Operated Relief Valves

Two of the Unit 2 lowest set steam generator safety valves operated to maintain pressure following the December 19, 1994, manual unit trip due to drift of the main steam power operated relief valves.

There were no challenges to the pressurizer power-operated relief valves.



OPERATING DATA REPORT

DOCKET NO.

50-275

UNIT

1

DATE

01/01/95 T. Eubank

COMPLETED BY

J. Stipicevich

TELEPHONE

(805) 545-4867/4877

OPERATING STATUS

1.	Unit Name:	Diablo Canyon Unit 1
2.	Reporting Period:	December 1994
3.	Licensed Thermal Power (MWt):	3338
4.	Nameplate Rating (Gross MWe):	1137
5.	Design Electrical Rating (Net MWe):	1086
6.	Maximum Dependable Capacity (Gross MWe):	1124
7.	Maximum Dependable Capacity (Net MWe):	1073.4
8.	If changes occur in capacity ratings (items	
	3 through 7) since last report, give reasons:	N/A
9.	Power level to which restricted, if any (Net MWe):	N/A
10.	Reasons for restrictions, if any:	N/A

		This Month	<u>YTD</u>	Cumulative
11.	Hours In Reporting Period	744.0	8760.0	84622.3
12.	Number Of Hours Reactor Was Critical	654.1	7041.0	71284.3
13.	Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14.	Hours Generator On-Line	645.1	6992.3	70334.4
15.	Unit Reserve Shutdown Hours	0.0	0.0	0.0
16.	Gross Thermal Energy Generated (MWH)	2074212	22842233	223181742
17.	Gross Electrical Energy Generated (MWH)	700800	7763400	75031932
18.	Net Electrical Energy Generated (MWH)	663098	7368876	71185429
19.	Unit Service Factor	86.71	79.82	83.12
20.	Unit Availability Factor	86.71	79.82	83.12
21.	Unit Capacity Factor (Using MDC Net)	83.03	78.37	78.37
22.	Unit Capacity Factor (Using DER Net)	82.07	77.46	77.46
23.	Unit Forced Outage Rate	13.29	5.21	3.11

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): None.

25. If Shut Down At End Of Report Period, Estimate Date of Startup: Not applicable.

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-275
UNIT	1
DATE	01/01/95
COMPLETED BY	T. Eubank
	J. Stipicevich
TELEPHONE	(805) 545-4867/4877

December 1994	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
	1	1081
	2	1085
	3	1082
	4	1086
	5	1081
	6	1082
	7	1085
	8	1077
	9	1085
	10	1085
	11	1082
	12	1081
	13	1081
	14	- 19
	15	- 40
	16	- 40
	17	- 49
	18	262
ø	19	571
	20	1009
	21	1080
	22	1076
	23	1077
	24	1081
	25	1073
	26 .	1082
	27	1077
	28	1077
	29	1077
	30	1081
	31	1078

The average monthly Electrical Power Level for December 1994 = 891.26 MWe-Net

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UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO.

50-275

UNIT

1

DATE 01/01/94

COMPLETED BY

D. D. Malone

TELEPHONE

(805) 545-4859

REPORT MONTH: December 1994

NO.	DATE	TYPE	DURATION (HOURS)	REASON ²	METHOD OF SHUTDOWN ³	LICENSEE EVENT REPORT		COMPONENT CODE ⁵	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
1	12/14/94	F	98.9	Н	3	1-94-020	N/A	N/A :	Western power pool trans- mission system voltage transient (for more inform- ation see LER 1-94-020)
								и	

Type: F-Forced

S-Scheduled

Reason:

2

A-Equipment Failure (Explain)

B-Maintenance or Test

C-Refueling

D-Regulatory Restriction

E-Operator Training & License Examination

F-Administrative

G-Operational Error (Explain)

H-Other (Explain)

3

Method:

1-Manual

2-Manual Scram

3-Automatic Scram

4-Continuation from previous month

5-Power reduction

6-Other

4

EIIS Systems List, Table 1

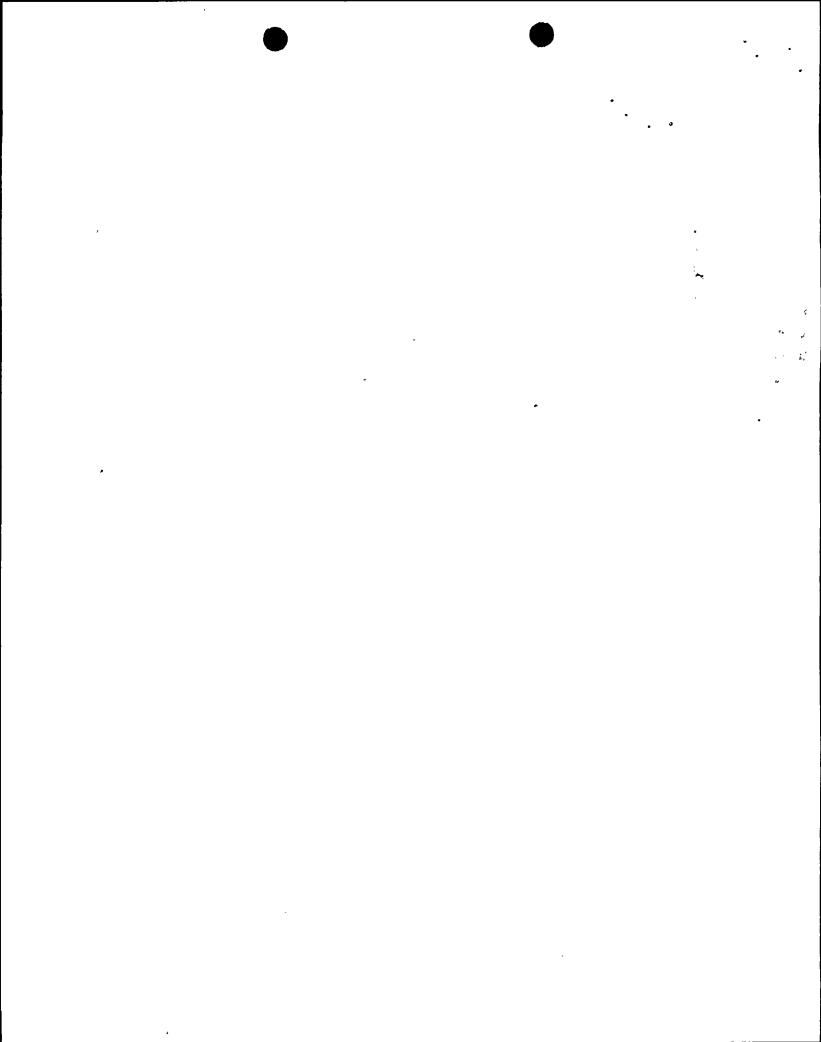
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IEEE Std. 803A-1983, "IEEE

Recommended Practice for Unique Identification in Power Plants and

Related Facilities - Table 2"





OPERATING DATA REPORT

DOCKET NO. 50-323 UNIT 2

DATE 01/01/95 COMPLETED BY T. Eubank

J. Stipicevich

TELEPHONE

(805) 545-4867/4877

OPERATING STATUS

1.	Unit Name:	Diablo Canyon Unit 2
2.	Reporting Period:	December 1994
	Licensed Thermal Power (MWt):	3411
4.	Nameplate Rating (Gross MWe):	1164
5.	Design Electrical Rating (Net MWe):	1119
6.	Maximum Dependable Capacity (Gross MWe):	1137
7.	Maximum Dependable Capacity (Net MWe):	1087
8.	If changes occur in capacity ratings (items	
	3 through 7) since last report, give reasons:	N/A
9.	Power level to which restricted, if any (Net MWe):	N/A
10.	Reasons for restrictions, if any:	N/A

		This Month	<u>YTD</u>	<u>Cumulative</u>
11.	Hours In Reporting Period	744.0	8760.0	77181.0
12.	Number Of Hours Reactor Was Critical	609.8	7560.2	65780.0
13.	Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14.	Hours Generator On-Line	552.9	7441.0	64769.8
15.	Unit Reserve Shutdown Hours	0.0	0.0	0.0
16.	Gross Thermal Energy Generated (MWH)	1719242	24810055	212642282
17.	Gross Electrical Energy Generated (MWH)	571900	8289904	70803003
18.	Net Electrical Energy Generated (MWH)	537162	7896101	67317914
19.	Unit Service Factor	74.31	84.94	83.92
20.	Unit Availability Factor	74.31	84.94	83.92
21.	Unit Capacity Factor (Using MDC Net)	66.42	82.92	80.36
22.	Unit Capacity Factor (Using DER Net)	64.52	80.55	77.95
23.	Unit Forced Outage Rate	25.69	6.19	4.28
24	Chartelesson Calcadalad Ossan Mart C Martle			

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): None.

25. If Shut Down At End Of Report Period, Estimate Date of Startup: Not applicable.

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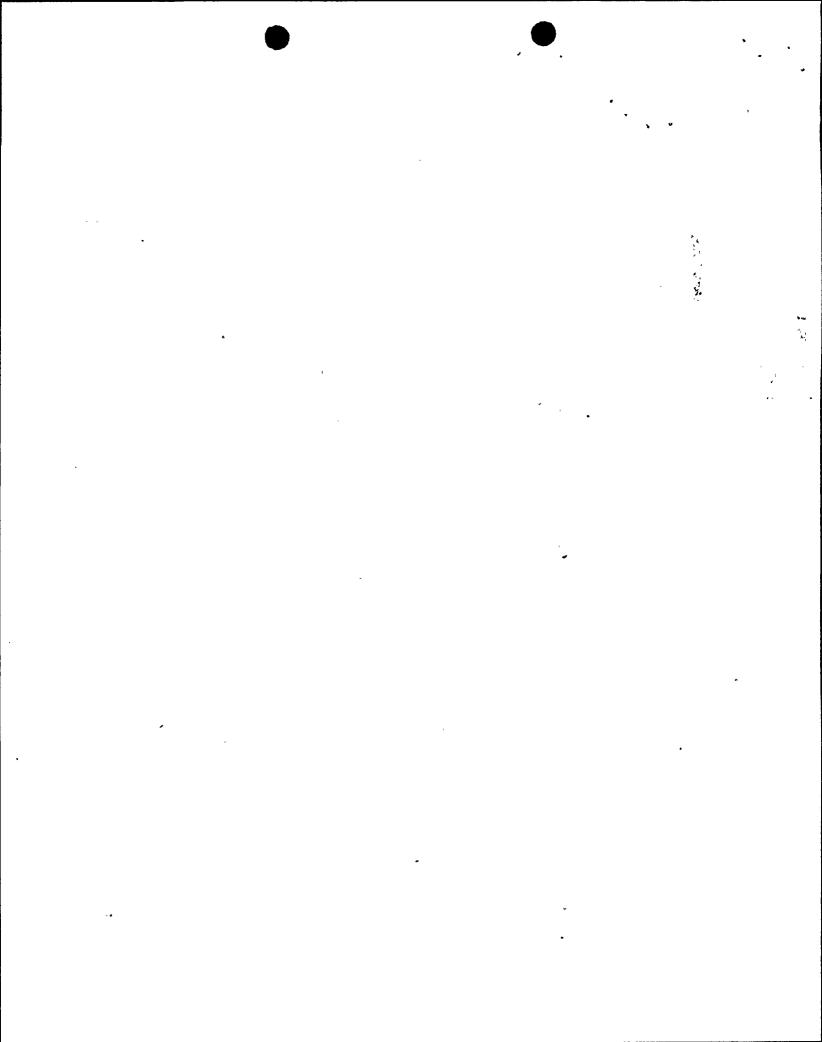
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AVERAGE DAILY UNIT POWER LEVEL

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December 1994	DAY	AVERAGE DAILY POWER LEVEL
		(MWe-Net)
	1	1096
		1096
	2 3	1096
	4	1101
	5	1096
	6	1096
	7	1101
	8	1096
	9	1101
	10	
		1097
•	. 11	1092
	12	1101
	13	1097
	14	- 16
	15	- 39
	16	107
	17	617
	18	1062
	19	421
	20	- 34 °
	21	- 38
	22	- 38
	23	- 38
	24	- 42
	25	235
	26	522
	27	1020
	28	1092
1	29	1095
	30	1097
	31	1092
	~ -	1072

The average monthly Electrical Power Level for December 1994 = 721.99 MWe-Net



UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO.

50-323

UNIT

DATE

01/01/95

COMPLETED BY TELEPHONE

D. D. Malone (805) 545-4859

REPORT MONTH: December 1994

NO.	DATE	TYPE	DURATION (HOURS)	REASON ²	METHOD OF SHUTDOWN?		SYSTEM CODE ¹	COMPONENT CODE ⁵	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
1	12/14/94	F	58.4	Н	3	1-94-020	N/A	N/A	Western power pool trans- mission system voltage transient (for more inform- ation see LER 1-94-020).
2	12/19/94	F	132.7	Н	2	2-94-012	NN	N/A	Manual unit trip due to high circulating water differential pressure caused by heavy seas (for more information see LER 2-94-012).
						· ·		•	

Type: F-Forced S-Scheduled

2 Reason:

A-Equipment Failure (Explain)

B-Maintenance or Test

C-Refueling

D-Regulatory Restriction

E-Operator Training & License Examination

F- Administrative

G-Operational Error (Explain)

H-Other (Explain)

3

Method:

1-Manual

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5-Power reduction

6-Other

EIIS Systems List, Table 1

5 IEEE Std. 803A-1983, "IEEE

Recommended Practice for Unique Identification in Power Plants and

Related Facilities - Table 2"

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REFUELING INFORMATION REQUEST

DOCKET NO.

50-275

UNIT

1

DATE

01/01/95

COMPLETED BY D. L. Farrer

D. D. Malone

TELEPHONE

(805) 545-4438/4859

1. Name of facility:

Diablo Canyon Unit 1

2. Scheduled date for next refueling shutdown:

September 9, 1995

3. Scheduled date for restart following refueling:

November 1, 1995

- 4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)? If no such review has taken place, when is it scheduled?
 - No. The PSRC reviewed and approved the cycle 7 core reload on March 28, 1994.
- 5. Scheduled date(s) for submitting proposed licensing action and supporting information: N/A
- 6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures: N/A
- 7. As of December 31, 1994, the number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool were:

(a) 193

(b) 464

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present 1324

Increase size by 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2006 (Loss of full core offload capability).

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REFUELING INFORMATION REQUEST

DOCKET NO.

50-323

UNIT

2

DATE

01/01/95

COMPLETED BY D. L. Farrer

D. Malan

D. D. Malone

TELEPHONE

(805) 545-4438/4859

1. Name of facility:

Diablo Canyon Unit 2

2. Scheduled date for next refueling shutdown:

March 9, 1996.

3. Scheduled date for restart following refueling:

May 5, 1996.

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)? If no such review has taken place, when is it scheduled?

No. The PSRC reviewed and approved the cycle 7 core reload on October 21, 1994.

- 5. Scheduled date(s) for submitting proposed licensing action and supporting information: N/A
- 6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures: N/A
- 7. As of December 31, 1994, the number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool were:

(a) 193

(b) 484

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present 1324

Increase size by 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2006 (Loss of full core offload capability).

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