

PACIFIC GAS AND ELECTRIC COMPANY  
NUCLEAR POWER GENERATION  
DIABLO CANYON POWER PLANT  
CORE OPERATING LIMITS REPORT

DCPP  
FOR  
INFORMATION  
ONLY

NUMBER COLR 1-7  
REVISION 0  
PAGE 1 OF 10  
UNIT

TITLE: COLR FOR DIABLO CANYON UNIT 1 CYCLE 7

1

APPROVED: *A. Mikulush* 4/28/94 4/28/94  
DATE EFFECTIVE DATE

PROCEDURE CLASSIFICATION: QUALITY RELATED

## 1.0 CORE OPERATING LIMITS REPORT

This Core Operating Limits Report (COLR) for Diablo Canyon Unit 1 Cycle 7 has been prepared in accordance with the requirements of Technical Specification (TS) 6.9.1.8.

The Technical Specifications affected by this report are listed below:

- 3/4.1.3.5 - Shutdown Rod Insertion Limit
- 3/4.1.3.6 - Control Rod Insertion Limits
- 3/4.2.1 - Axial Flux Difference
- 3/4.2.2 - Heat Flux Hot Channel Factor -  $F_0(z)$
- 3/4.2.3 - RCS Flow Rate and Nuclear Enthalpy Rise Hot Channel Factor

## 2.0 OPERATING LIMITS

The cycle-specific parameter limits for the specifications listed in Section 1.0 are presented in the following subsections. These limits have been developed using the NRC-approved methodologies specified in Technical Specification 6.9.1.8.

### 2.1 Shutdown Rod Insertion Limit (TS 3/4.1.3.5)

2.1.1 The shutdown rods shall be withdrawn to at least 225 steps.

### 2.2 Control Rod Insertion Limits (TS 3/4.1.3.6)

2.2.1 The control rod banks shall be limited in physical insertion as shown in Figure 1.

### 2.3 Axial Flux Difference (TS 3/4.2.1)

2.3.1 The AXIAL FLUX DIFFERENCE (AFD) Limits are provided in Figure 2.

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TITLE: COLR FOR DIABLO CANYON UNIT 1 CYCLE 7

2.4 Heat Flux Hot Channel Factor -  $F_Q(z)$  (TS 3/4.2.2)

2.4.1

$$F_Q(z) < \frac{F_{RTP}}{P} * K(z) \quad \text{for } P > 0.5$$

$$F_Q(z) \leq \frac{F_{RTP}}{0.5} * K(z) \quad \text{for } P \leq 0.5$$

where:  $P = \frac{\text{THERMAL POWER}}{\text{RATED THERMAL POWER}}$

$$F_{RTP} = 2.45$$

$K(z)$  is provided in Figure 6.

2.4.2 The  $W(z)$  curves for Specification 4.2.2.2.c Relaxed Axial Offset Control (RAOC) operation, provided in Figures 3 through 5, are sufficient to determine the RAOC  $W(z)$  versus core height for Cycle 7 burnups through the end of full power reactivity plus a power coastdown of up to 1000 MWD/MTU.

SECRET  
CONFIDENTIAL

TITLE: COLR FOR DIABLO CANYON UNIT 1 CYCLE 7

UNIT

1

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2.5 RCS Flow Rate and Nuclear Enthalpy Rise Hot Channel Factor  
(Specification 3/4.2.3)

$$R = \frac{F_{\Delta H}^N}{F_{\Delta H}^{RTP} * [1 + PF_{\Delta H} * (1-P)]}$$

where:  $P = \frac{\text{THERMAL POWER}}{\text{RATED THERMAL POWER}}$

$$F_{\Delta H}^{RTP} = 1.59$$

$$PF_{\Delta H} = 0.3$$

SECRET  
CONFIDENTIAL

TITLE: COLR FOR DIABLO CANYON UNIT 1 CYCLE 7

ONLY

UNIT

1

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### 3.0 FIGURES

3.1 Figure 1, Rod Bank Insertion Limits Versus Rated Thermal Power

3.2 Figure 2, AFD Limits as a Function of Rated Thermal Power

3.3 Figure 3,  $W(z)$  at 150 MWD/MTU

3.4 Figure 4,  $W(z)$  at 10,000 MWD/MTU

3.5 Figure 5,  $W(z)$  at 18,000 MWD/MTU

3.6 Figure 6,  $K(z)$  - Normalized  $F_Q(z)$  as a Function of Core Height

### 4.0 RECORDS

None

### 5.0 REFERENCES

Westinghouse Reload Safety Evaluation for Diablo Canyon Power Plant Unit 1  
Cycle 7, Revision 1, dated April 1994.

### 6.0 SPONSOR

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TITLE: COLR FOR DIABLO CANYON UNIT 1 CYCLE 7

UNIT

1

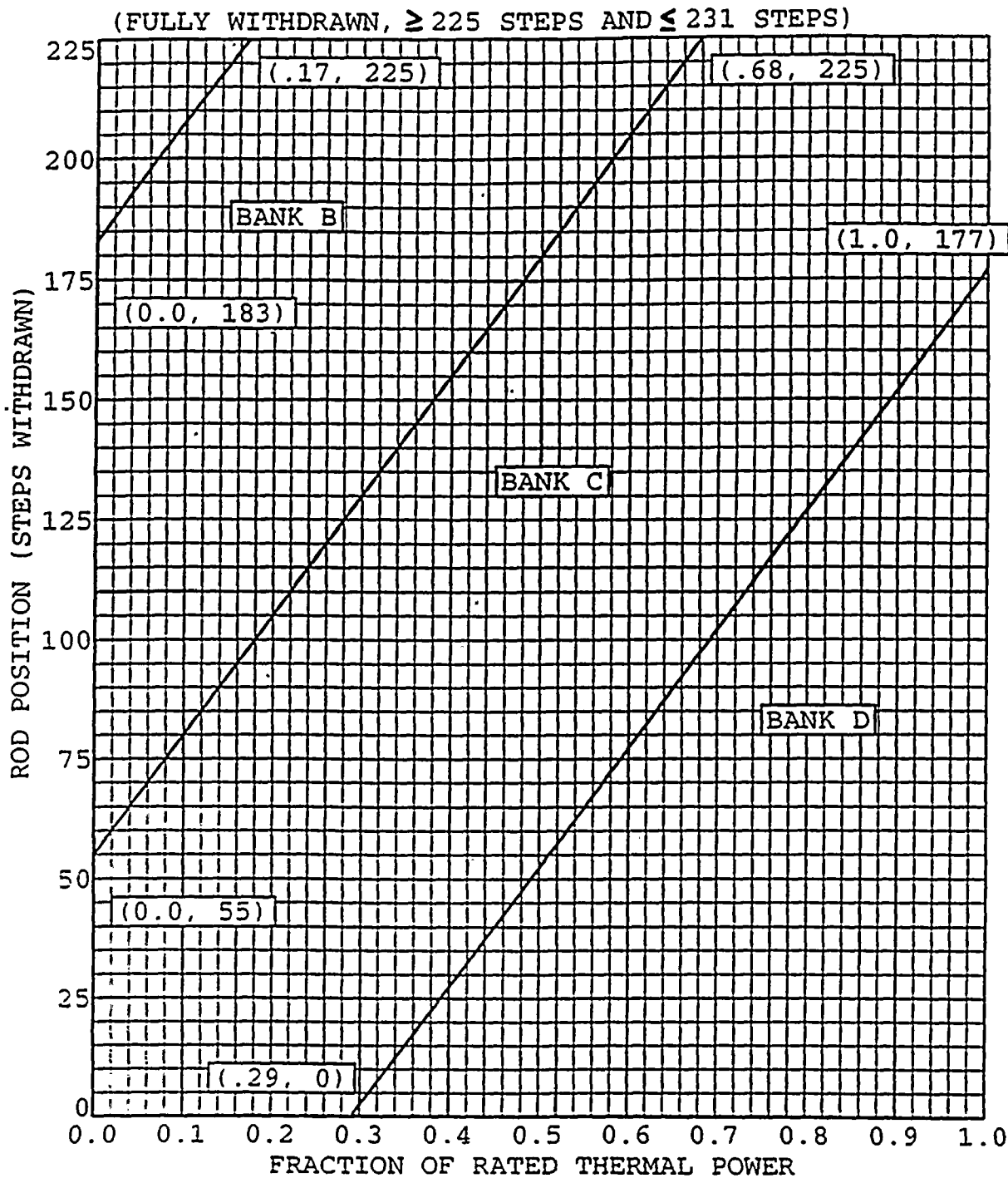


Figure 1

Rod Bank Insertion Limits Versus Rated Thermal Power



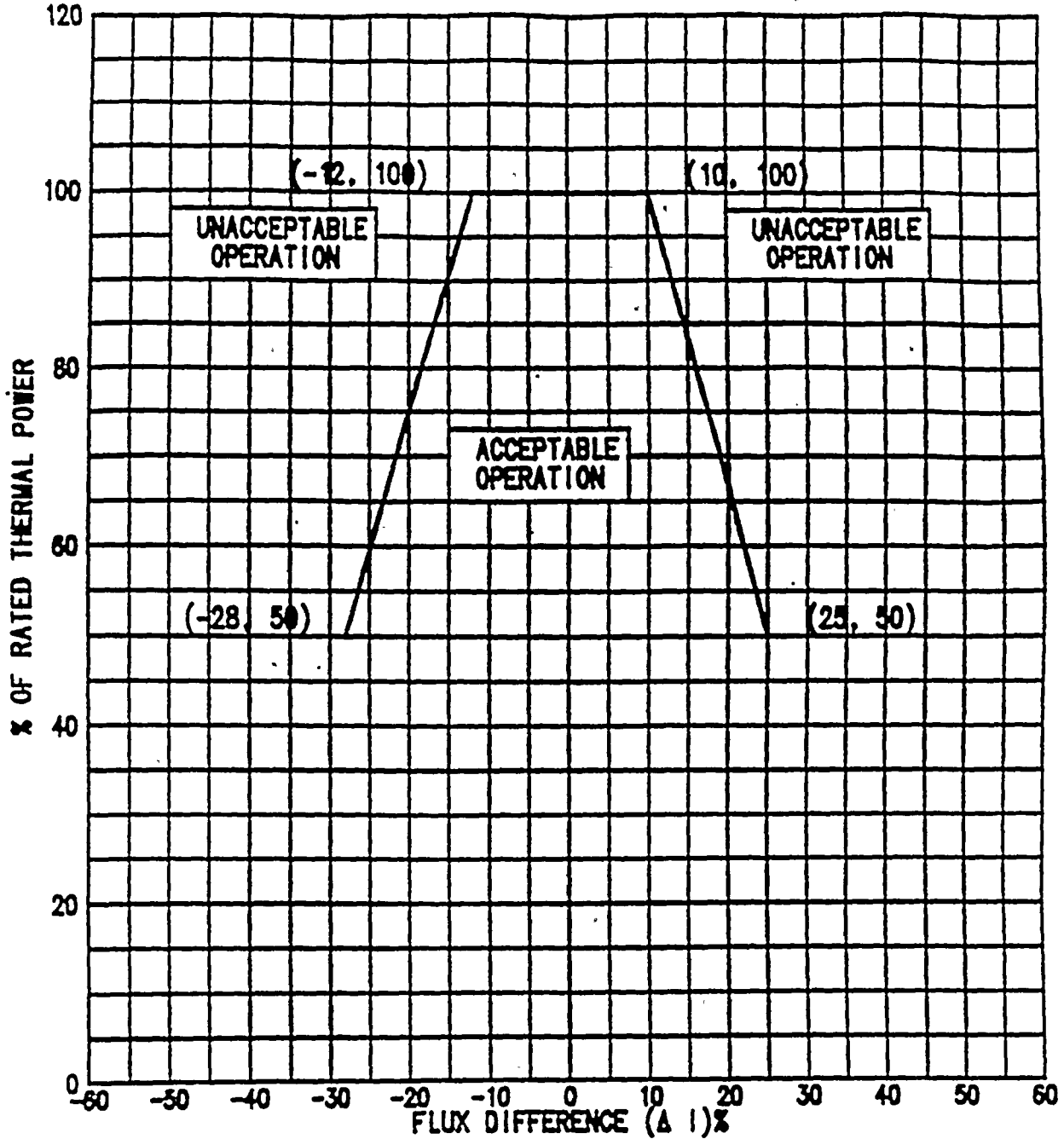


Figure 2

AFD Limits as a Function of Rated Thermal Power

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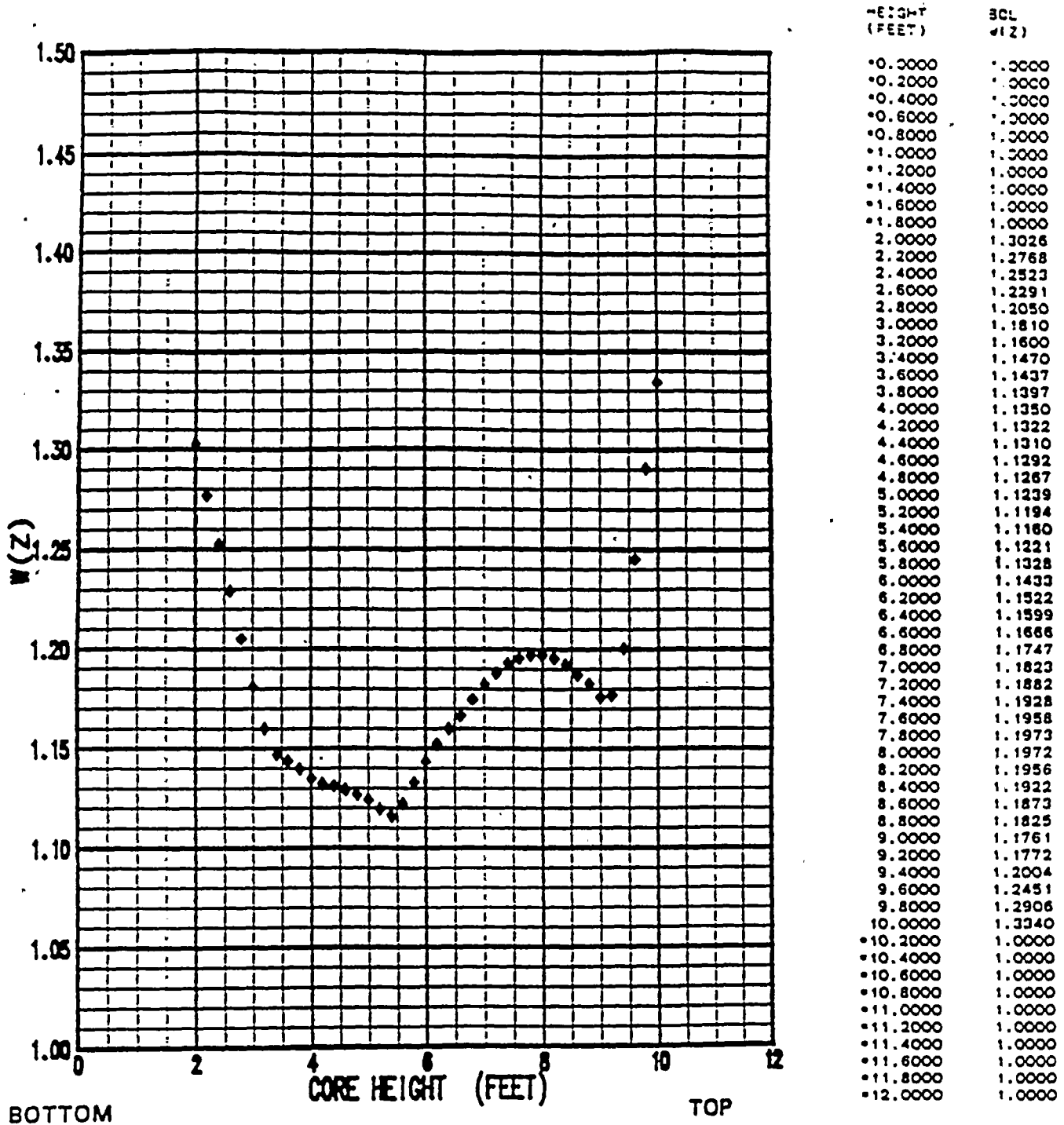


Figure 3

W(z) at 150 MWD/MTU

\* Top and Bottom 15% excluded as per TS 4.2.2.2.g

12 1  
12 1  
12 1  
12 1  
12 1

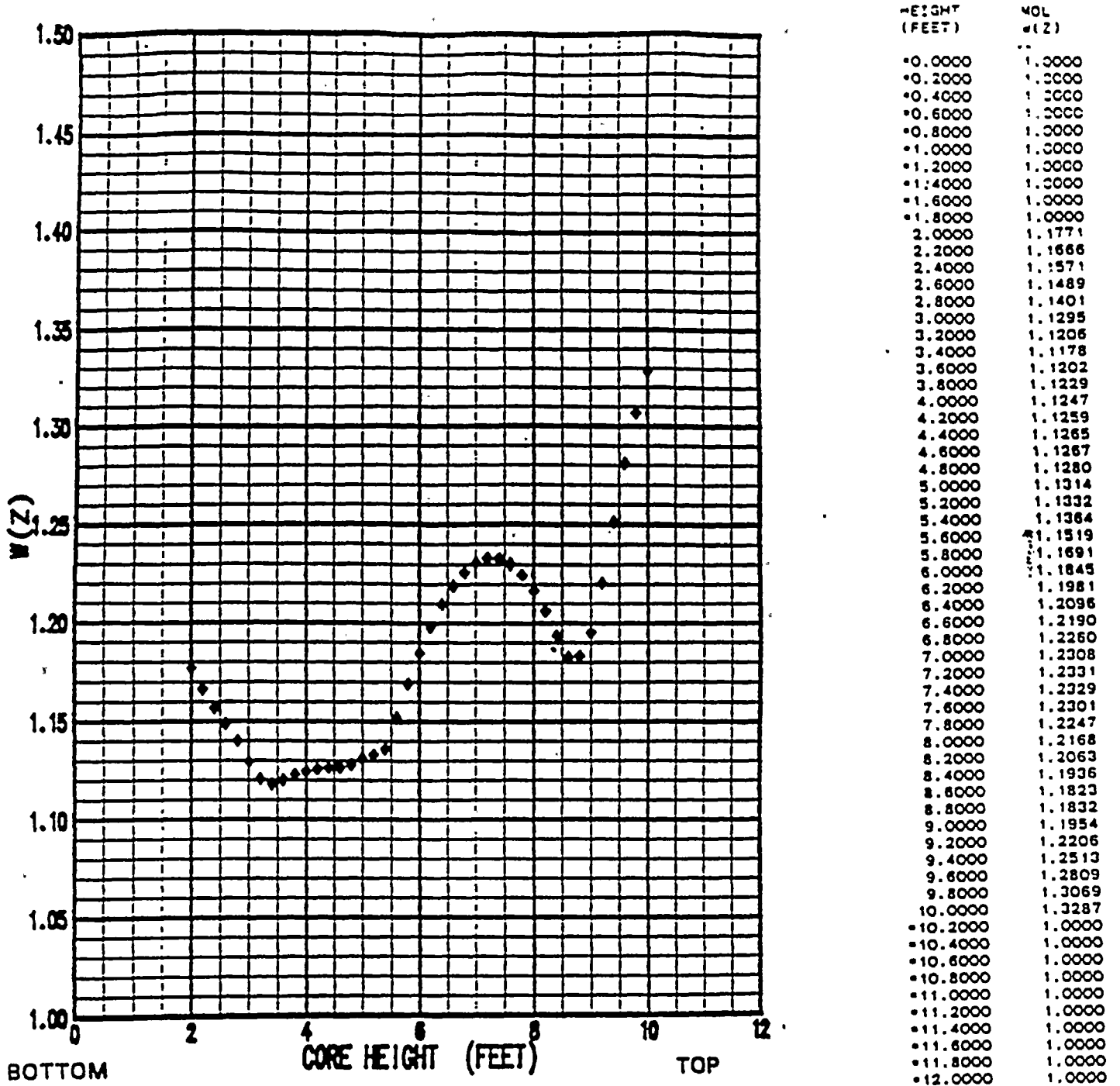


Figure 4

W(z) at 10,000 MWD/MTU

\* Top and Bottom 15% excluded as per TS 4.2.2.2.g

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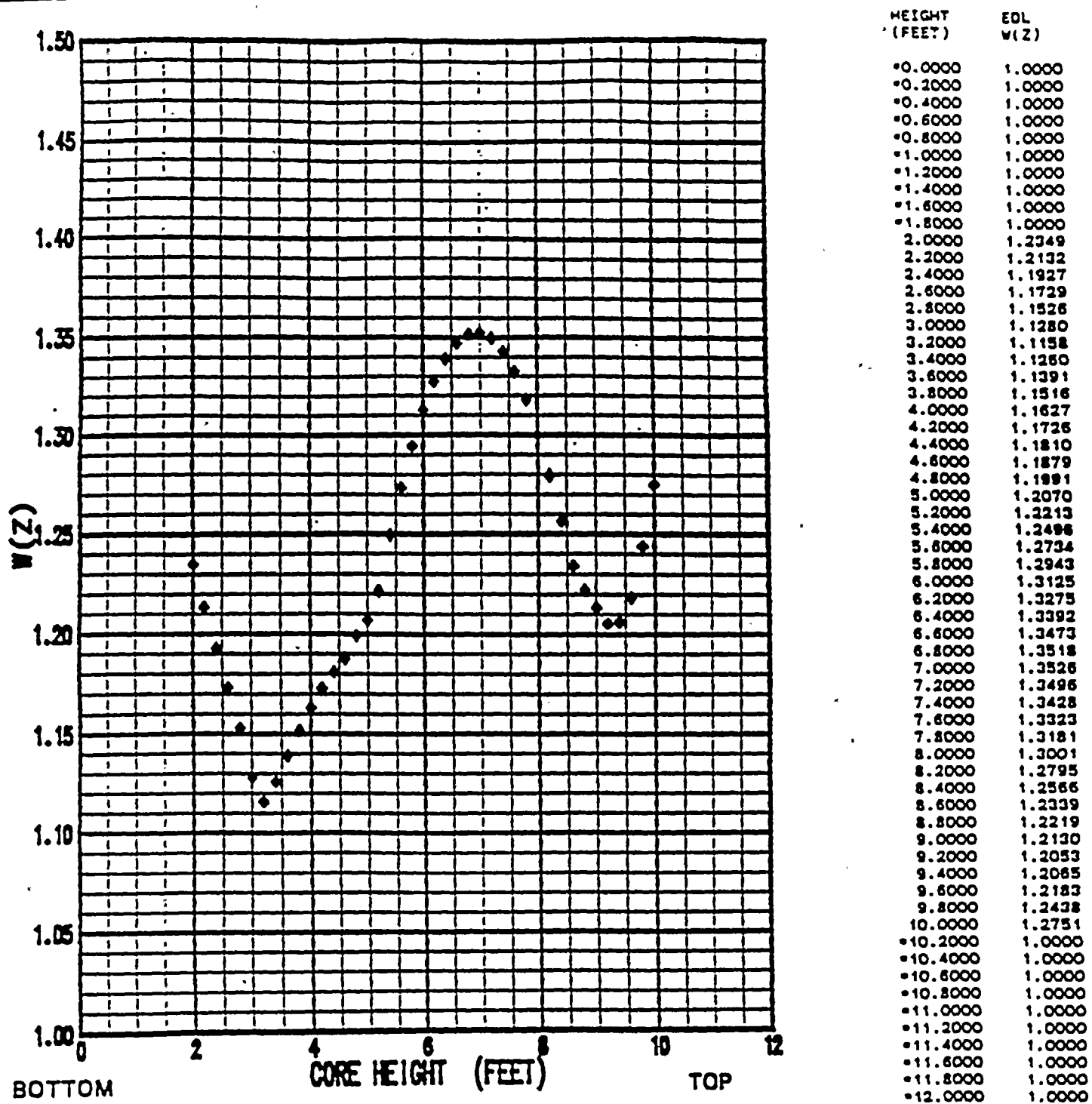


Figure 5

W(z) at 18,000 MWD/MTU

\* Top and Bottom 15% excluded as per TS 4.2.2.2.g

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Page

100  
101  
102

TITLE: COLR FOR DIABLO CANYON UNIT 1 CYCLE 7 ONLY

UNIT

1

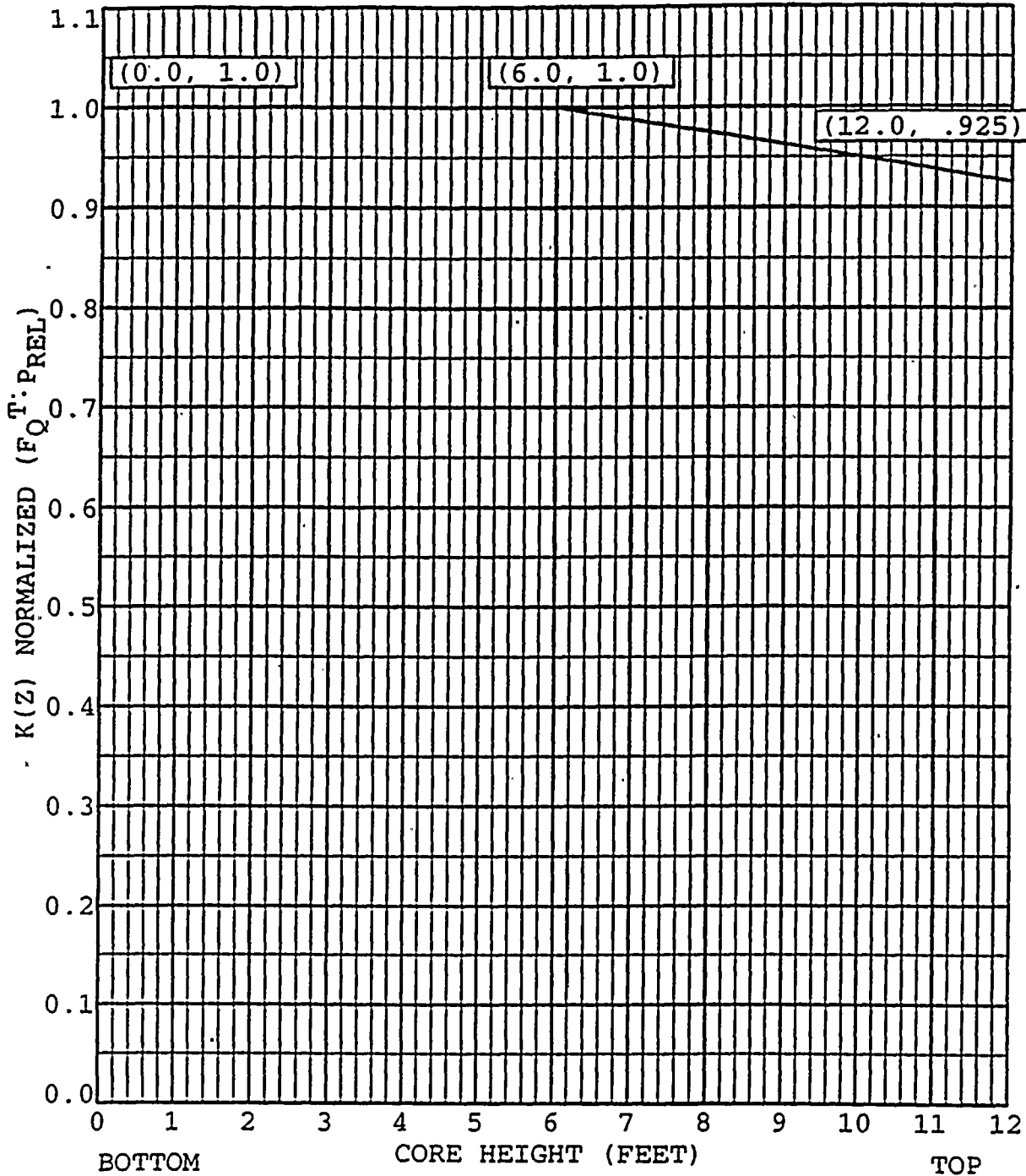


Figure 6

$K(z)$  - Normalized  $F_Q(z)$  as a Function of Core Height



1000  
1000  
1000

1000  
1000  
1000