

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8705190614 DOC. DATE: 87/05/11 NOTARIZED: NO DOCKET #
 FACIL: 50-323 Diablo Canyon Nuclear Power Plant, Unit 2, Pacific Ca 05000323
 AUTH. NAME AUTHOR AFFILIATION
 SISK, D. P. Pacific Gas & Electric Co.
 SHIFFER, J. D. Pacific Gas & Electric Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 87-005-00: on 870410, both RHR trains inoperable during refueling outage. Caused by air entrainment in RHR pump due to vortexing in pump suction line. RCS flooded from refueling water storage tank & RHR pump started. W/870511 ltr.

DISTRIBUTION CODE: IE22D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 2
 TITLE: 50.73 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:

RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
PD5 LA	1 1	PD5 PD	1 1
SCHIERLING, H	1 1	TRAMMELL, C	1 1
INTERNAL: ACRS MICHELSON	1 1	ACRS MOELLER	2 2
AEOD/DOA	1 1	AEOD/DSP/ROAB	2 2
AEOD/DSP/TPAB	1 1	DEDRO	1 1
NRR/DEST/ADE	1 0	NRR/DEST/ADS	1 0
NRR/DEST/CEB	1 1	NRR/DEST/ELB	1 1
NRR/DEST/ICSB	1 1	NRR/DEST/MEB	1 1
NRR/DEST/MTB	1 1	NRR/DEST/PSB	1 1
NRR/DEST/RSB	1 1	NRR/DEST/SGB	1 1
NRR/DLPQ/HFB	1 1	NRR/DLPQ/QAB	1 1
NRR/DOEA/EAB	1 1	NRR/DREP/EPB	1 1
NRR/DREP/RAB	1 1	NRR/DREP/RPB	2 2
NRR/PMAS/ILRB	1 1	NRR/PMAS/PTSB	1 1
REG FILE 02	1 1	RES DEPY GI	1 1
RGN5 FILE 01	1 1		
EXTERNAL: EG&G GROH, M	5 5	H ST LOBBY WARD	1 1
LPDR	2 2	NRC PDR	1 1
NSIC HARRIS, J	1 1	NSIC MAYS, G	1 1

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) DIABLO CANYON UNIT 2	DOCKET NUMBER (2) 050003123	PAGE (3) 1 OF 1
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TITLE (4)
INTERRUPTION OF RHR FLOW DURING RCS MID-LOOP OPERATION

EVENT DATE (5)			LER NUMBER (6)		REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
04	10	87	87	01015	010	05	11	87		050003123

OPERATING MODE (9) 5	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (11)	
POWER LEVEL (10) 0-1010		
<input checked="" type="checkbox"/> 10 CFR 50.73(a)(2)(v) <input type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 365A)		

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER
DAVID P. SISK, REGULATORY COMPLIANCE ENGINEER		AREA CODE
		815595-7341

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		
X	A	B	S	H	V	G	2	5	5	NO	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input type="checkbox"/> NO				
			06	15	87

ABSTRACT (16)

On April 10, 1987, at 2123 PDT, with the unit in Mode 5 (Cold Shutdown) during a refueling outage, both RHR trains (PB) were inoperable due to airbound RHR pumps (PB)(P). The four-hour significant event report required by 10 CFR 50.72 was made at 2230 PDT, April 10, 1987.

The reactor coolant system (RCS)(AB) had been drained to mid-loop level to facilitate removal of steam generator primary manways for nozzle dam installation in steam generators 1 and 4. In addition, preparations were in progress for a local leak rate test on containment isolation valves in the reactor coolant pump seal water return line (including draining of the penetration). Due to a leaking valve used as a clearance point in the piping to the penetration being drained, RCS inventory was lost to the reactor coolant drain tank (RCDT). This loss of inventory caused a decrease in RCS water level and resulted in air entrainment in the RHR pump because of vortexing in the pump suction line.

At 2251 PDT, after verification that the steam generator manways were still installed and after venting of the RHR pumps, the RCS was flooded from the RWST and an RHR pump started. RHR flow was interrupted for approximately 1 hour and 28 minutes. This resulted in some localized boiling, but no damage to the core or significant radiological release. The unit was stable at 0130 PDT, April 11, 1987, and was returned to normal Mode 5 mid-loop operation. A supplemental report will be submitted after PGandE and NRC investigations have been completed.

1430S/0050K
 8705190614 PDR
 870511 ADOCK
 05000323 PDR

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PACIFIC GAS AND ELECTRIC COMPANY

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JAMES D. SHIFFER
VICE PRESIDENT
NUCLEAR POWER GENERATION

May 11, 1987

PGandE Letter No.: DCL-87-105

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Re: Docket No. 50-323, OL-DPR-82
Diablo Canyon Unit 2
Licensee Event Report 2-87-005-00
Interruption of RHR Flow During RCS Mid-Loop Operation

Gentlemen:

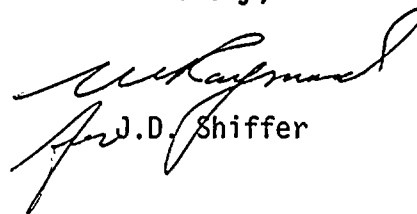
Pursuant to 10 CFR 50.73(a)(2)(v), PGandE is submitting the enclosed Licensee Event Report concerning the interruption of RHR flow during RCS mid-loop operation. This event did not affect the public's health and safety.

Kindly acknowledge receipt of this material on the enclosed copy of this letter and return it in the enclosed addressed envelope.

Sincerely,

Enclosure

cc: L. J. Chandler
J. B. Martin
M. M. Mendonca
P. P. Narbut
B. Norton
C. M. Trammell
CPUC
Diablo Distribution
INPO


J.D. Shiffer

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