



REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR	:9908170073 DOC.DATE: 99/08/12 NOTARIZED: NO	DOCKET #
FACIL:50-275	Diablo Canyon Nuclear Power Plant, Unit 1, Pacific Ga	05000275
AUTH.NAME	AUTHOR AFFILIATION	
RUSSELL,R.	Pacific Gas & Electric Co.	
OATLEY, D.N.	Pacific Gas & Electric Co.	
RECIP.NAME	RECIPIENT AFFILIATION	

SUBJECT: LER 99-005-00:on 990713, noted that SDM calculation was not performed with one hour as required by TS 4.1.1.1.1a.Caused by personnel error.Surveillance testing was performed & rod control sys was declared inoperable.With 990812 ltr.

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NOTES:

	RECIPIENT ID CODE/NAME LPD4-2 PD	COPI LTTR 1	ES ENCL 1	RECIPIENT ID CODE/NAME BLOOM.S	COP LTTR 1	IES ENCL 1	O R
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EXTERNAL:	L ST LOBBY WARD NOAC POORE,W. NRC PDR	1 1 1	1 1 1	LMITCO MARSHALL NOAC QUEENER,DS NUDOCS FULL TXT	1 1 1	1 1 1	D

NOTE TO ALL "RIDS" RECIPIENTS: PLEASE HELP US TO REDUCE WASTE. TO HAVE YOUR NAME OR ORGANIZATION REMOVED FROM DISTRIBUTION LISTS OR REDUCE THE NUMBER OF COPIES RECEIVED BY YOU OR YOUR ORGANIZATION, CONTACT THE DOCUMENT CONTROL DESK (DCD) ON EXTENSION 415-2083

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Diablo Canyon Power Plant P.O. Box 56 Avila Beach, CA 93424

805.545.6000

August 12, 1999

PG&E Letter DCL-99-107

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, DC 20555-0001

Docket No. 50-275, OL-DPR-80 Diablo Canyon Unit 1 <u>Licensee Event Report 1-1999-005-00</u> Technical Specification 4.1.1.1.1a, Not Met Due to Personnel Error

Dear Commissioners and Staff:

PG&E is submitting the enclosed licensee event report regarding Technical Specification 4.1.1.1.1 a not being met due to personnel error.

This event was not risk significant and did not adversely affect the health and safety of the public.

Sincerely David H. Oatley

cc: Steven D. Bloom Ellis W. Merschoff David L. Proulx Diablo Distribution INPO

Attachment

DDM/2246/N0002100

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FACILITY NAME (1) Diablo Car	nyon Unit 1				-	0 5	DOCKET N	UMBER (2)	7	5	1	PAGE (3) OF	6		
Technical	Specificatio	ns 4.1.1.1	1.1a, N	lot Met	Due to	Perso	onnel E	rror		,			•		
EVENT DATE (5)	LER	NUMBER (6)		REP	ORT E (7)		a	OTHER FAC	ILITIES IN	VOLVED	(8)				
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MODE (9) 1 POWER	X	10 CFR OTHER	5	0.73(a)(2)(i)(B))				<u>.</u>		=			
100	LEVEL (10) (SPECIFY IN ABSTRACT BELOW AND IN TEXT, NRC FORM 366A) LICENSEE CONTACT FOR THIS LER (12)														
	LICENSEE CONTACT FOR THIS LER (12) TELEPHONE NUMBER AREA CODE														
Roger Rus	LICENSEE CONTACT FOR THIS LER (12) TELEPHONE NUMBER AREA COOE Roger Russell - Senior Regulatory Services Engineer Roger Russell - Senior Regulatory Services Engineer														
CAUSE SYSTEM	COMPONENT	OMPLETE ONE LI MANUFACTU	NE FOR EA RER R	CH COMPONE EPORTABLE TO EPIX	NT FAILURE CAUSE	DESCRIBE	O IN THIS RE	PORT (13) ONENT	MANU	FACTUR	ER	REPOR TO E	TABLE		
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ABSTRACT (Limits 140 On July percent was not 1 hour a rod cont TS 3.1.3 On July operator system of Plant op inoperat PG&E w whenever regardin TS to en	13, 1999, a power, Teo met when a is required l rol system 3.1.c. 16, 1999, a s satisfacto operable, ex erators faile ole rod cont ill provide g er TS 3.1.3. g managem sure that al	at 1930 PI shnical Sp a shutdow by TS 4.1 inoperable at 0146 PI orily perfor xiting TSs ad to iden rol system guidance f 1c is enten nent's exp il applicab	DT, with ecification of the contract of the co	th Unit 1 ation (TS rgin (SD a. At 18 entered lowing p surveilla at a SDN to ambig Addition ons to p veillance	⁵⁾ I in Mo S) 3.1.1 M) calo 30 PD the lim blanned nce tes A calou guity in tors to ally, all erform es are	de 1 (.1 "SI culatic T, pla hiting d d corre sting a d corre sting a lation TS 3. perfor plant a tho prope	Power of nutdowr on was r nt opera conditio ective m and decl was re 1.3.1c. m a SD operate rough a rly com	Operation Marg not per ators h in for o nainten lared ti quired OM calc ors will ind cor pleted.	tion) a in - M forme ad de perat ance he roo with culation be c mplete	at 10 lodes ed wi eclare ion fo , plar d con an ouns e revi	0 s 1-; thin ed t or nt ntrol elec iew	2," he d of			

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT

I. Plant Conditions

Unit 1 was in Mode 1 (Power Operation) at approximately 100 percent power.

II. Description of Problem

A. Summary

On July 13, 1999, at 1930 PDT, with Unit 1 in Mode 1 at 100 percent power, Technical Specification (TS) 3.1.1.1 "Shutdown Margin - Modes 1-2," was not met when a shutdown margin (SDM) calculation was not performed within 1 hour as required by TS 4.1.1.1.1a. At 1830 PDT, plant operators declared the rod control system (AA) inoperable and entered the limiting condition for operation for TS 3.1.3.1.c.

On July 16, 1999, at 0146 PDT, following planned corrective maintenance, plant operators satisfactorily performed surveillance testing and declared the rod control system operable, exiting TSs.

B. Background

The rod control system (AA) is a nonsafety-related system to move the rods into and out of the reactor core to control reactivity in response to measured reactor coolant parameters or manual demand input. The system sequentially operates stationary and movable grippers for each rod to control speed and direction of rod movement. The rod control system is self-monitored and will initiate an urgent failure alarm if significant abnormalities are detected. Removal of a power cabinet card results in an urgent alarm input that prevents rod motion.

An urgent failure alarm does not indicate a problem with the safetyrelated function of tripping the reactor. The rod control system cannot prevent the safety-related function of reactor trip which actuates by removing motive power from the rod control mechanisms allowing the rods to fall by gravity into the core.

TS 3.1.1.1, "Boration Control, Shutdown Margin - Tavg Greater than 200°F," TS 4.1.1.1.1a, requires the shutdown margin be determined to be greater than or equal to 1.6 percent $\Delta k/k$ "within 1 hour after detection of an inoperable control rod(s) and at least once per

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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C. Event D	escrip	tion	ł													•				
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	On July operator On July rod syst	On July 16, 1999, at 0034 PDT, following planned maintenance, plant operators successfully reset the rod control system failure alarm. On July 16, 1999, at 0126 PDT, plant operators restored the control od system in automatic with all alarms cleared. On July 16, 1999, at 0146 PDT, plant operators satisfactorily performed STP R-1A, "Exercising Full Length Control Rods," declared																	
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D. .	Inoperat the Ever	ole S nt	Stru	ctur	es,	Con	npo	nent	is, or S	yste	ms	that	Con	tribı	uted	to			
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E.	Dates ar	nd A	ppr	oxin	nate	Tin	nes	fori	Major (Dcci	ırrer	nces							
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F.	Other Sy	ster	ns c	or Se	eco	ndaı	уF	unci	tions A	ffect	ed		5						
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IV.	Analys	sis of the	Ev	<u>ent</u>															
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TEXT			<u></u>											.					
V.	<u>Corre</u>	ective Act	lions								٣								
	A.	Immedi	ate Co	rrec	ctive /	Actio	ons					*							
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	В.	Correct	ive Act	ions	s to F	reve	ent R	eci	urrence	;									
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