REPLACEMENT MARKED-UP TECHNICAL SPECIFICATIONS

Marked-up Pages

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¹ This page was not affected by this supplement. However, this page has been updated to reflect license amendments issued since LAR 97-10 was submitted on July 30, 1997.

² This insert was not affected by this supplement. It is included for completeness.

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TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

FUN	NCTIONAL UNIT	TOTAL NO. OF_CHANNELS	CHANNELS TO_TRIP	MINIMUM CHANNELS <u>OPERABLE</u>	APPLICABLE MODES	ACTION
7.	Loss of Power (4.16 kV Emergency Bus Undervoltage)					
	a. First Level			3	1, 2, 3, 4	
	1) Diesel Start	1/Bus	- 1/Bus	1/Bus		16
	2) Initiation of Load Shed	2/Bus	2/Bus	2/Bus		15
	b. Second Level				1, 2, 3, 4	۵
	1) Undervoltage Relays	2/Bus	2/Bus	2/Bus	,	15 ()
	2) Timers to Start Diesel	1/Bus	1/Bus	1/Bus		16
	3) Timers to Shed Load	1/Bus	1/Bus	1/Bus		16
8.	Engineered Safety Features Actuation System Interlocks					
	a. Pressurizer Pressure, P-11	3	2	2	1, 2, 3	21
	b. DELETED	A				
\sim	c. Reactor Trip, P-4	2	2	2	1, 2, 3	23
9.	Residual Heat Removal Pump Trip On Refueling Water Storage Tank Level - Low	3	2	2	1, 2, 3, 4	36 ·

DIABLO CANYON - UNITS 1 & 2

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Unit 1 - Amendment 84) 12 Unit 2 - Amendment 83) 12



TABLE 3.3-3 (Continues)

ACTION STATEMENTS (Continued)

- ACTION 21 With less than the Minimum Number of Channels OPERABLE, within 1 hour determine by observation of the associated permissive annunciator window(s) that the interlock is in its required state for the existing plant condition, or apply Specification 3.0.3.
- ACTION 22 With the number of OPERABLE Channels one less than the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 6 hours or be in at least HOT STANDBY within the next 6 hours and in at least HOT SHUTDOWN within the following 6 hours: however, one channel may be bypassed for up to 4 hours for surveillance testing per Specification 4.3.2.1 provided the other channel is OPERABLE.
- ACTION 23 With the number of OPERABLE channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within 6 hours and in at least HOT SHUTDOWN within the following 6 hours.
- ACTION 24 With the number of OPERABLE channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or declare the associated pump or valve inoperable and take the ACTION required by Specification 3.7.1.5 or 3.7.1.2 as applicable.
- ACTION 25 With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 6 hours or be in at least HOT STANDBY within the next 6 hours; however, one channel may be bypassed for up to 4 hours for surveillance testing per Specification 4.3.2.1 provided the other channel is OPERABLE.
- ACTION 29 With the number of OPERABLE channels less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided that within 6 hours, for the affected RCS Loop Delta-T channel(s), either:
 - a. The Trip Time Delay threshold power level for zero seconds time delay is adjusted to 0% RTP, or
 - b. With the number of OPERABLE channels one less than the Total Number of Channels, the affected Steam Generator Water Level-Low-Low channels are placed in the tripped condition.
- ACTION 35 With the number of OPERABLE channels one less than the Total Number of Channels. STARTUP and/or POWER OPERATION may proceed provided the following conditions are satisfied:
 - a. The inoperable channel is placed in the trip condition within 6 hours, and
 - b. The Minimum Channels OPERABLE requirement is met; however, the inoperable channel may be bypassed for up to 4 hours for surveillance testing of other channels per Specification 4.3.2.1.

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DIABLO CANYON - UNITS 1 & 2

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ACTION 36 With the number of OPERABLE channels one less than the Total Number of Channels, within 6 hours place the inoperable channel in cut-out and restore the inoperable channel to OPERABLE status within 48 hours; or be in at least Hot Standby within the next 6 hours and be in Cold Shutdown within the next 30 hours.

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TABLE 3.3-4 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

FUNCTIONAL UNIT TRIP SETPOINT ALLOWABLE VALUES 7. Loss of Power (4.16 kV Emergency Bus Undervoltage) a. First Level > 0 volts with a 1) Diesel Start > 0 volts with a \leq 0.8 second time delay \leq 0.8 second time delay and and \geq 2583 volts with > 2583 volts with a \leq 10 second time delay \leq 10 second time delay 2) Initiation of Load Shed One relay **One** relav ≥ 0 volts with a \geq 0 volts with a \leq 4 second time delay < 4 second time delay</pre> and and \geq 2583 volts with a \leq 25 second time delay ≥ 2583 volts with a \leq 25 second time delay with one relay
≥ 2870 volts, instantaneous with one relay > 2870 volts, instantaneous b. Second Level \geq 3785 volts with a \leq 10 second time delay ≥ 3785 volts with a 1) Diesel Start Insert ≤ 10 second time delay
≥ 3785 volts with a
≤ 20 second time delay \geq 3785 volts with a \leq 20 second time delay 2) Initiation of Load Shed В 8. Engineered Safety Features Actuation System Interlocks a. Pressurizer Pressure, P-11 ≤ 1915 psig < 1917.5 psig b. DELETED c. Reactor Trip, P-4 N.A. N.A. NOTE 1: Time constants utilized in the lead-lag compensator for Steam Pressure - Low are τ_1 = 50 seconds and $\tau_2 = 5$ seconds. NOTE 2: Steam Generator Water Level Low-Low Trip Time Delay TD = $B1(P)^3 + B2(P)^2 + B3(P) + B4$ Where: P = RCS Loop ∆T Equivalent to Power (%RTP), P ≤ 50% RTP TD = Time delay for Steam Generator Water Level Low-Low (in seconds) B1 = -0.007128B2 = +0.8099 $B\bar{3} = -31.40$ B4 = +464.1NOTE 3: Time constants utilized in the rate-lag compensator for Negative Steam Line Pressure Rate-High are $\tau_3 = 50$ seconds and $r_4 = 50$ seconds. DIABLO CANYON - UNITS 1 & 2 Unit 1 - Amendment No. 37\\72\\84\\86\\92\\103\(\122 3/4 3-27 Unit 2 - Amendment No. 361/8 32972507.4a **TAB 10** 28 Ι

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9. Residual Heat Removal Pump Trip on Refueling Water Storage Tank Level-LOW

32.56%

≤ 33.68% and ≥ 31.44%

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Fun	<u>ICT I (</u>	<u>DNAL_UNIT</u>	CHANNEL <u>CHECK</u>	CHANNEL CALI- <u>BRATION</u>	CHANNEL OPERA- TIONAL TEST	TRIP ACTUATI DEVICE OPERA- TIONAL <u>TEST</u>	NG ACTUATION LOGIC_TEST	MASTER RELAY <u>TEST</u>	SLAVE RELAY TEST	MODES FOR WHICH SURVEILLANCE <u>IS REQUIRED</u>
6.	Aux	kiliary Feedwater (Continued)						٠		- -
	d.	Undervoltage - RCP	N.A.	R24	N.A.	R24	N.A.	N.A.	N.A.	1
	e.	Safety Injection	See Item	1. above	for all	Safety I	njection Sur	veillanc	e Requir	ements.
7.	Los	ss of Power								
	a.	4.16 kV Emergency Bus Level 1	N.A.	R	N.A.	R	N.A.	N.A.	N.A.	1, 2, 3, 4
	b.	4.16 kV Emergency Bus Level 2	N.A. *	R	N.A.	R	N.A.	N.A.	N.A.	1, 2, 3, 4
8.	Enc Act	jineered Safety Feature cuation System Interlocks								
	a.	Pressurizer Pressure, P-11	N.A.	R24	Q	N.A.	N.A.	N.A.	N.A.	1, 2, 3
	b.	DELETED								
~~	с.	Reactor Trip, P-4	N.A.	N.A.	N.A.	R24	N.A.	N.A.	N.A.	1, 2, 3
Ins	ert	C		TABLE NO	TATIONS					
$\begin{pmatrix} 1\\2 \end{pmatrix}$	Eac For per	the Containment Ventilation Exh formed at least once every 31 da	every 62 aust Radia ys.	days on a ation - Hi	STAGGER gh monit	ED TEST 1 or only,	BASIS. a CHANNEL F	UNCTIONA	L TEST sl	hall be
()	blo	cked below P-11 when Safety Inje	ction on S	Steam Line	Pressur	e-Low is	not blocked	•	iiu 15 au	comacically
(3)	For	Mode 3, the Trip Time Delay asson or equal to 464.1 seconds.	ociated wi	ith the St	eam Gene	rator Wa [.]	ter Level-Lou	w-Low cha	annel mus	st be less

TABLE 4.3-2 (Continued) ENGINEERED SAFETY FEATURES ACTUATION SYSTEM_INSTRUMENTATION SURVEILLANCE_REQUIREMENTS

DIABLO CANYON - UNITS 1 & 2

Unit 1 - Amendment $\delta I = \delta I$

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ENGINEER	ED SAFETY FE	BLE 4.3-2 ATURES AC VEILLANCE	(Continu TUATION S REQUIREM	ed) YSTEM_IN ENTS	STRUMENTATIO	N		
FUNCTIONAL UNIT	CHANNEL <u>CHECK</u>	CHANNEL CALI- BRATION	CHANNEL OPERA- TIONAL <u>TEST</u>	TRIP ACTUATI DEVICE OPERA- TIONAL <u>TEST</u>	NG ACTUATION LOGIC_TEST	MASTER RELAY <u>TEST</u>	SLAVE RELAY TEST	MODES FOR WHICH SURVEILLANCE IS_REQUIRED
9. Residual Heat Removal Pump Trip on Refueling Water Storage Tank Level - Low	S	R24	Q	N.A.	R 24	N.A.	Ν.Α	1,2,3,4

<u>Insert C</u>

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REACTOR PROTECTION SYSTEM and ENGINEERED SAFETY FEATURES ACTUATION SYSTEM **INSTRUMENTATION** (Continued)

ESF response times specified in Table 3.3-5, which include sequential operation of the RWST and VCT valves (Table Notations 4 and 5), are based on values assumed in the non-LOCA safety analyses. These analyses take credit for injection of borated water from the RWST. Injection of borated water is assumed not to occur until the VCT charging pump suction isolation valves are closed following opening of the RWST charging pump suction isolation valves. When the sequential operation of the RWST and VCT valves is not included in the response times (Table Notation 7), the values specified are based on the LOCA analyses. The LOCA analyses takes credit for injection flow regardless of the source. Verification of the response times specified in Table 3.3-5 will assure that the assumptions used for the LOCA and non-LOCA analyses with respect to the operation of the VCT and RWST valves are valid.

For slave relays in the ESF actuation system circuit that are Potter & Brumfield type MDR relays, the SLAVE RELAY TEST is performed on a refueling frequency. The test frequency is based on relay reliability assessments presented in WCAP-13878, "Reliability Assessment of Potter and Brumfield MDR Series Relays," WCAP-13900, "Extension of Slave Relay Surveillance Test Intervals," and WCAP-14117, "Reliability Assessment of Potter and Brumfield MDR Series Relays." These reliability assessments are relay specific and apply only to Potter and Brumfield MDR series relays. Note that for normally energized applications, the relays may have to be replaced periodically in accordance with the guidance given in WCAP-13878 for MDR relays.

Undervoltage protection will generate a loss of power diesel generator start in the event a loss of voltage or degraded voltage condition occurs. The diesel generators provide a source of emergency power when offsite power is either unavailable or is insufficiently stable to allow safe unit operation. The first level undervoltage relays (FLURs) detect the loss of bus voltage (less than 69% bus voltage). The second level undervoltage relays (SLURs) provide a second level of undervoltage protection which protects all Class 1E loads from short or long term degradation in the offsite power system. The SLUR allowable value is the minimum steady state voltage needed on the 4160 volt vital bus to ensure adequate voltage is available for safety related equipment at the 4160 volt, 480 volt, and 120 volt levels.



DIABLO CANYON - UNITS 1 & 2

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The RWST low level trip of the RHR pumps, the only automatic action in the switchover to the containment recirculation sump, assures that continued cooling is provided by the ECCS to remove decay heat. After the RWST low level trip is received, operators manually switch the source of water for the ECCS pumps to the containment recirculation sump. Switchover from the RWST to the containment sump must occur before the RWST empties to prevent damage to the ECCS pumps and a loss of core cooling capability. For similar reasons, switchover must not occur before there is sufficient water in the containment sump to support RHR pump suction. Furthermore, early switchover must not occur to ensure that sufficient borated water is injected from the RWST. This ensures the reactor remains shut down in the recirculation mode.

The RWST low level trip of the RHR pumps is included in the RHR pump control system and is not a part of the solid state protection system. Each of the three channels is provided with a cut-out feature, which allows the channel to be bypassed. A channel may be placed in the cut-out mode for up to 48 hours to allow maintenance and testing. This places the system in a two-out-of-two trip logic.

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REPLACEMENT PROPOSED TECHNCIAL SPECIFICATION PAGES

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TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

<u>Fun</u>	CTIONAL_UNIT	TOTAL NO. <u>OF CHANNELS</u>	CHANNELS TO TRIP	MINIMUM CHANNELS <u>OPERABLE</u>	APPLICABLE MODES	ACTION
7.	Loss of Power (4.16 kV Emergency Bus Undervoltage)					
	a. First Level		· •	-	1, 2, 3, 4	
	1) Diesel Start	1/Bus	1/Bus	1/Bus		16
	2) Initiation of Load Shed	2/Bus	2/Bus	2/Bus		15
	b. Second Level				1, 2, 3, 4	8
	1) Undervoltage Relays	2/Bus	2/Bus	2/Bus		15
	2) Timers to Start Diesel	1/Bus	1/Bus	1/Bus		16
	3) Timers to Shed Load	1/Bus	1/Bus	1/Bus		16
8.	Engineered Safety Features Actuation System Interlocks					
	a. Pressurizer Pressure, P-11	3	2	2	1, 2, 3	21
	b. DELETED			-		
	c. Reactor Trip, P-4	2	2	2	1, 2, 3	23
9.	Residual Heat Removal Pump Trip on Refueling Water Storage Tank Level - Low	3	2	2	1, 2, 3, 4	36

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TABLE 3.3-3 (Continued)

ACTION STATEMENTS (Continued)

- ACTION 21 With less than the Minimum Number of Channels OPERABLE, within 1 hour determine by observation of the associated permissive annunciator window(s). that the interlock is in its required state for the existing plant condition, or apply Specification 3.0.3.
- ACTION 22 With the number of OPERABLE Channels one less than the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 6 hours or be in at least HOT STANDBY within the next 6 hours and in at least HOT SHUTDOWN within the following 6 hours; however, one channel may be bypassed for up to 4 hours for surveillance testing per Specification 4.3.2.1 provided the other channel is OPERABLE.
- ACTION 23 With the number of OPERABLE channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within 6 hours and in at least HOT SHUTDOWN within the following 6 hours.
- ACTION 24 With the number of OPERABLE channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or declare the associated pump or valve inoperable and take the ACTION required by Specification 3.7.1.5 or 3.7.1.2 as applicable.
- ACTION 25 With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 6 hours or be in at least HOT STANDBY within the next 6 hours; however, one channel may be bypassed for up to 4 hours for surveillance testing per Specification 4.3.2.1 provided the other channel is OPERABLE.
- ACTION 29 With the number of OPERABLE channels less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided that within 6 hours, for the affected RCS Loop Delta-T channel(s), either:
 - a. The Trip Time Delay threshold power level for zero seconds time delay is adjusted to 0% RTP, or
 - b. With the number of OPERABLE channels one less than the Total Number of Channels, the affected Steam Generator Water Level-Low-Low channels are placed in the tripped condition.
- ACTION 35 With the number of OPERABLE channels one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided the following conditions are satisfied:
 - a. The inoperable channel is placed in the trip condition within 6 hours, and
 - b. The Minimum Channels OPERABLE requirement is met; however, the inoperable channel may be bypassed for up to 4 hours for surveillance testing of other channels per Specification 4.3.2.1.
- ACTION 36 With the number of OPERABLE channels one less than the Total Number of Channels, within 6 hours place the inoperable channel in cut-out and restore the inoperable channel to OPERABLE status within 48 hours; or be in at least Hot Standby within the next 6 hours and be in Cold Shutdown within the next 30 hours.

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TABLE 3.3-4 (Continued)

ENGINEERED_SAFETY FEATURES ACTUATION_SYSTEM_INSTRUMENTATION_TRIP_SETPOINTS

FUNCTIONAL UNIT

TRIP SETPOINT

ALLOWABLE VALUES

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7. Loss of Power (4.16 kV Emergency Bus Undervoltage) a. First Level 1) Diesel Start \geq 0 volts with a \leq 0.8 second time delay and > 0 volts with a \leq 0.8 second time delay and \geq 2583 volts with a \leq 10 second time delay \geq 2583 volts with \leq 10 second time delay 2) Initiation of Load Shed One relay One relay \geq 0 volts with a \leq 4 second time delay and \geq 0 volts with a \leq 4 second time delay and \geq 2583 volts with a \leq 25 second time delay ≥ 2583 volts with a
 ≤ 25 second time delay
 with one relay
 ≥ 2870 volts, instantaneous with one relay \geq 2870 volts, instantaneous b. Second Level ≥ 3785 volts with a
 ≤ 10 second time delay
 ≥ 3785 volts with a
 ≤ 20 second time delay ≥ 3785 volts with a
 ≤ 10 second time delay
 ≥ 3785 volts with a
 ≤ 20 second time delay 1) Diesel Start 2) Initiation of Load Shed 8. Engineered Safety Features Actuation System Interlocks a. Pressurizer Pressure, P-11 ≤ 1915 psig < 1917.5 psig</pre> b. DELETED c. Reactor Trip, P-4 N.A. N.A. \leq 33.68% and \geq 31.44% 9. Residual Heat Removal Pump Trip on 32.56% Refueling Water Storage Tank Level - Low NOTE 1: Time constants utilized in the lead-lag compensator for Steam Pressure - Low are τ_1 = 50 seconds NOTE 1: Time constants utilized in the reading compensator for compensator and $\tau_2 = 5$ seconds. NOTE 2: Steam Generator Water Level Low-Low Trip Time Delay TD = B1(P) + B2(P) + B3(P) + B4 Where: P = RCS Loop ΔT Equivalent to Power (%RTP), P \leq 50% RTP TD = Time delay for Steam Generator Water Level Low-Low (in seconds) B1 = -0.007128 B2 = +0.8099B3 = -31.40B4 = +464.1NOTE 3: Time constants utilized in the rate-lag compensator for Negative Steam Line Pressure Rate-High are $\tau_3 = 50$ seconds and $\tau_4 = 50$ seconds.

DIABLO CANYON - UNITS 1 & 2

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SEE PSRC INTERPRETATION 96-09

		TABLE 4.3-2	(Continued)	
ENGINEERED	SAFETY	FEATURES ACT	UATION SYSTEM	INSTRUMENTATION
		SURVEILLANCE	REQUIREMENTS	·······

				0	ACTUATI	NG			
FUN	<u>CTIONAL UNIT</u>	CHANNEL <u>CHECK</u>	CHANNEL CALI- <u>BRATION</u>	OPERA- TIONAL <u>TEST</u>	OPERA- TIONAL TEST	ACTUATION LOGIC TEST	MASTER RELAY <u>TEST</u>	SLAVE RELAY <u>TEST</u>	MODES FOR WHICH SURVEILLANCE <u>IS REQUIRED</u>
6.	Auxiliary Feedwater (Continued)								
	d. Undervoltage - RCP	N.A.	R24	N.A.	R24	N.A.	N.A.	N.A.	1
	e. Safety Injection	See Item	1. above	for all	Safety I	njection Sur	veillanc	e Requir	rements.
7.	Loss of Power								
	a. 4.16 kV Emergency Bus Level 1	N.A.	R	N.A.	R	N.A.	N.A.	N.A.	1, 2, 3, 4
	b. 4.16 kV Emergency Bus Level 2	N.A.	R .	N.A.	R	N.A.	N.A.	N.A.	1, 2, 3, 4
8.	Engineered Safety Feature Actuation System Interlocks								
	a. Pressurizer Pressure, P-11	N.A.	R24	Q.	N.A.	N.A.	N.A.	N.A.	1, 2, 3
	b. DELETED								
	c. Reactor Trip, P-4	N.A.	N.A.	N.A.	R24	N.A.	N.A.	N.A.	1, 2, 3
9.	Residual Heat Removal Pump Trip on Refueling Water Storage Tank Level - Low	S	R24	Q	N.A.	R24	N.A.	N.A.	1, 2, 3, 4
					•				

TABLE NOTATIONS

Each train shall be tested at least every 62 days on a STAGGERED TEST BASIS.
 For the Containment Ventilation Exhaust Radiation - High monitor only, a CHANNEL FUNCTIONAL TEST shall be performed at least once every 31 days.
 Irip function automatically blocked above P-11 (Pressurizer Pressure Interlock) setpoint and is automatically blocked below P-11 when Safety Injection on Steam Line Pressure-Low is not blocked.

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 (4) Deleted.
 (5) For Mode 3, the Trip Time Delay associated with the Steam Generator Water Level-Low-Low channel must be less than or equal to 464.1 seconds.

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BASES

<u>REACTOR PROTECTION SYSTEM and ENGINEERED SAFETY FEATURES ACTUATION SYSTEM</u> <u>INSTRUMENTATION</u> (Continued)

ESF response times specified in Table 3.3-5, which include sequential operation of the RWST and VCT valves (Table Notations 4 and 5), are based on values assumed in the non-LOCA safety analyses. These analyses take credit for injection of borated water from the RWST. Injection of borated water is assumed not to occur until the VCT charging pump suction isolation valves are closed following opening of the RWST charging pump suction isolation valves. When the sequential operation of the RWST and VCT valves is not included in the response times (Table Notation 7), the values specified are based on the LOCA analyses. The LOCA analyses takes credit for injection flow regardless of the source. Verification of the response times specified in Table 3.3-5 will assure that the assumptions used for the LOCA and non-LOCA analyses with respect to the operation of the VCT and RWST valves are valid.

For slave relays in the ESF actuation system circuit that are Potter & Brumfield type MDR relays, the SLAVE RELAY TEST is performed on a refueling frequency. The test frequency is based on relay reliability assessments presented in WCAP-13878, "Reliability Assessment of Potter and Brumfield MDR Series Relays," WCAP-13900, "Extension of Slave Relay Surveillance Test Intervals," and WCAP-14117, "Reliability Assessment of Potter and Brumfield MDR Series Relays." These reliability assessments are relay specific and apply only to Potter and Brumfield MDR series relays. Note that for normally energized applications, the relays may have to be replaced periodically in accordance with the guidance given in WCAP-13878 for MDR relays.

Undervoltage protection will generate a loss of power diesel generator start in the event a loss of voltage or degraded voltage condition occurs. The diesel generators provide a source of emergency power when offsite power is either unavailable or is insufficiently stable to allow safe unit operation. The first level undervoltage relays (FLURs) detect the loss of bus voltage (less than 69% bus voltage). The second level undervoltage relays (SLURs) provide a second level of undervoltage protection which protects all Class 1E loads from short or long term degradation in the offsite power system. The SLUR allowable value is the minimum steady state voltage needed on the 4160 volt vital bus to ensure adequate voltage is available for safety related equipment at the 4160 volt, 480 volt, and 120 volt levels.

The RWST low level trip of the RHR pumps, the only automatic action in the switchover to the containment recirculation sump, assures that continued cooling is provided by the ECCS to remove decay heat. After the RWST low level trip is received, operators manually switch the source of water for the ECCS pumps to the containment recirculation sump. Switchover from the RWST to the containment sump must occur before there is sufficient water in the containment sump to support RHR pump suction. Furthermore, early switchover must not occur to ensure that sufficient borated water is injected from the RWST. This ensures the reactor remains shut down in the recirculation mode.

The RWST low level trip of the RHR pumps is included in the RHR pump control system and is not a part of the solid state protection system. Each of the three channels is provided with a cut-out feature, which allows the channel to be bypassed. A channel may be placed in the cut-out mode for up to 48 hours to allow maintenance and testing. This places the system in a two-out-of-two trip logic.

DIABLO CANYON - UNITS 1 & 2

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REPLACEMENT MARKED-UP IMPROVED TECHNICAL SPECIFICATION PAGES

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Insert for Q 3,3-29

Enclosure 5A page 3.3-33 Insert ACTION K

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К.	One channel inoperable	K.1.1	Place the channel in cut-out.	6 hours
		AND		
		K.1.2	Return the inoperable channel to an OPERABLE status.	48 hours
		OR		•
		K.2.1	Be in MODE 3.	54 hours
			AND	
		K.2.2	Be in MODE 5.	84 hours

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SURVEILLANCE REC	DUIREMENTS (continued)	
	SURVEILLANCE	FREQUENCY
SR 3.3.2.11	Verification of setpoint not required. Perform TADOT.	3.3-61 DL ALL-001 Once-per-reactor trip breaker cycle18 months
SR 3.3.2.12	Peforn ACTUATION LOGIC TEST	24 months 3.3.29 DC ALL-002
5R3.3.2.13	Verification of setpoint not required for manual initiation functions. Perform TADOT.	(<u>3.3-139</u>) 18 Houth 3 DC All-001

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ESFAS Instrumentation 3.3.2

FUNCTION	SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPOINT
, b.Refueling Water Storage Tank (RWST) Level - Low Lo	1.2.3.4 2 6w	4	۴.	SR-3,3,2,1 SR-3,3,2,5 SR-3,3,2,9 SR-3,3,2,10 SR-3,3,2,10	<mark>≥ [15]* and</mark> < [-]*	<mark>≥_[_]</mark> _a ≤_[_]
Coincident with Safety Injection	Refer-to-Function requirements.	-1-(Safety-In	jection)-for- ;	all initiation fu	nctions—and	
	•			•		(continu
(a) Reviewer's No Setpoint Stuc	ote: Unit-specific imp dy-methodology-used by '	lementations m the-unit.	ay-contain-on	ly_Allowable_Valu	e-depending-on	
(2) Reviewer's No Setpoint Stud 7. Residual Removal on Refuell Storage T,	Heat 1,2,3,4 PumpTrip Two Drip Two Lister	lementations n the unit.	K	SR 3.3.2.1 SR 3.3.2.1 SR 3.3.2.9 SR 3.3.2.12	e <u>depending on</u> <u>≤</u> 33.68% ≥ 31.49%	ED 3.1 32.

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B 3.3 INSTRUMENTATION

B 3.3.2 Engineered Safety Feature Actuation System (ESFAS) Instrumentation

BASES

BACKGROUND The ESFAS initiates necessary safety systems. based on the values of selected unit parameters. to protect against violating core design limits and the Reactor Coolant System (RCS) pressure boundary. and to mitigate accidents.

The ESFAS instrumentation is segmented into three distinct but interconnected modules as identified below:

- Field transmitters or process sensors and instrumentation: provide a measurable electronic signal based on the physical characteristics of the parameter being measured:
- Signal processing equipment including analog digital protection system, field contacts, and protection channel sets: provide signal conditioning, bistable setpoint comparison, process algorithm actuation, compatible electrical signal output to protection system devices, and control board/control room/ miscellaneous indications; and
- Solid State Protection System (SSPS) including input, logic. and output bays: initiates the proper unit shutdown or engineered safety feature (ESF) actuation in accordance with the defined logic and based on the bistable outputs from the signal process control and protection system.

Field Transmitters or Sensors

To meet the design demands for redundancy and reliability, more than one, and often as many as four, field transmitters or sensors are used to measure unit parameters. In many cases, field transmitters or sensors that input to the ESFAS are shared with the Reactor Trip System (RTS). In some cases, the same channels also provide control system inputs. To account for calibration tolerances and instrument drift, which are assumed to occur between calibrations, statistical allowances are provided in the Trip Setpoint and Allowable

he residual hect remoral pump trip or refueling water storage tonk level-low is not processed by the ssps. The associated relays are located in the residual heat removal sumps control system

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BASES (continued)

APPLICABLE Ea SAFETY ANALYSES, C LCO, AND pr APPLICABILITY Fu



Each of the analyzed accidents can be detected by one or more ESFAS Functions. One of the ESFAS Functions is the primary actuation signal for that accident. An ESFAS Function may be the primary actuation signal for more than one type of accident. An ESFAS Function may also be a secondary, or backup, actuation signal for one or more other accidents. For example. Pressurizer Pressure Low is a primary actuation signal for small loss of coolant accidents (InCAs) and a backup actuation signal for steam line breaks (SLBs) outside containment. Functions such as manual initiation, not specifically credited in the accident safety analysis, are qualitatively credited in the safety analysis and the NRC staff approved licensing basis for the unit. These Functions may provide protection for conditions that do not require dynamic transient analysis to demonstrate Function performance. These Functions may also serve as backups to Functions that were credited in the accident analysis (Ref. 3).

The LCO requires all instrumentation performing an ESFAS Function to be OPERABLE. Failure of any instrument renders the affected channel(s) inoperable and reduces the reliability of the affected Functions.

The LCO generally requires OPERABILITY of four or three channels in each instrumentation function and two channels in each logic and manual initiation function. The two-out-of-three and the two-out-of-four configurations allow one channel to be tripped or OCALCOP bypassed during maintenance or testing without causing an ESFAS initiation. Two logic or manual initiation channels are required to ensure no single random failure disables the ESFAS.

The required channels of ESFAS instrumentation provide unit protection in the event of any of the analyzed accidents. ESFAS protection functions are as follows:

1. <u>Safety Injection</u>

Safety Injection (SI) provides two primary functions:

 Primary side water addition to ensure maintenance or recovery of reactor vessel water level (coverage of the active fuel for heat removal, clad integrity, and for limiting peak clad temperature to < 2200°F); and

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7. Residual Heat Removal Pump Trip on Refueling Water Storage Tank Level - Low

At the end of the injection phase of a LOCA, the RWST will be nearly empty. Continued cooling must be provided by the ECCS to remove decay heat. The source of water for the ECCS pumps is manually switched to the containment recirculation sump. This pump trip feature is blocked if the RHR pumps are already taking suction from the containment recirculation sump. The low head RHR pumps draw the water from the containment recirculation sump, the RHR pumps pump the water through the RHR heat exchanger, inject the water back into the RCS, and supply the cooled water to the other ECCS pumps. Switchover from the RWST to the containment sump must occur before the RWST empties to prevent damage to the RHR pumps and a loss of core cooling capability. For similar reasons, switchover must not occur before there is sufficient water in the containment sump to support RHR pump suction. Furthermore, early switchover must not occur to ensure that sufficient borated water is injected from the RWST. This ensures the reactor remains shut down in the recirculation mode.

During the injection phase of a LOCA, the RWST is the source of water for all ECCS pumps. The RHR pump trip on RWST low level provides protection against a loss of water for the ECCS pumps and indicates the end of the injection phase of the LOCA. The RWST is equipped with three level transmitters. These transmitters provide no control functions. Therefore, a two-out-of-three logic is adequate to initiate the protection function actuation.

The Allowable Value/Trip Setpoint upper limit is selected to ensure adequate water inventory in the containment sump to provide RHR pump suction. The high limit also ensures enough borated water is injected to ensure the reactor remains shut down.

The transmitters are located in an area not affected by HELBs or post accident high radiation. Thus, they will not experience any adverse environmental conditions and the trip setpoint reflects only steady state instrument uncertainties.

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Insert A (continued)

This Function must be OPERABLE in MODES 1, 2, 3, and 4 when there is a potential for a LOCA to occur, to ensure a continued supply of water for the ECCS pumps. This Function is not required to be OPERABLE in MODES 5 and 6 because there is adequate time for the operator to evaluate unit conditions and respond by manually starting systems, pumps, and other equipment to mitigate the consequences of an abnormal condition or accident. System pressure and temperature are very low and many ESF components are prevented from actuating to prevent inadvertent overpressurization of unit systems or are not required to be operable. ·

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ESFAS Instrumentation B 3.3.2

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APPLICABLE SAFETY ANALYSES.	a. <u>Automatic Switchover-to Containment Sump</u> <u>Automatic Actuation Logic and Actuation Relays</u>
APPLICABILITY (continued)	Automatic-actuation-logic-and-actuation-relays consist-of-the-same-features-and-operate-in-the-same manner-as-described-for-ESFAS-Function-1.b.
₽.	-C. <u>Automatic-Switchover-to-Containment-Sump-Refueling</u> Water Storage Tank (RWST) Level Low Low Coincident With Safety Injection and Coincident With Containment Sump-Level High
and the second s	During the injection phase of a LOCA, the RWST is the source of water-for all ECCS pumps. A low low level in the RWST-coincident with an <u>SI-signal</u> provides protection against a loss of water for the ECCS pumps and indicates the end of the injection phase of the LOCA. The RWST is equipped with four three level transmitters. These transmitters provide no control functions. Therefore, a two-out-of-threefour logic is adequate to initiate the protection function actuation. <u>Although-only-three-channels would-be</u> <u>sufficient. a fourth-channel-has-been-added-for</u> increased-reliability.
autinformation	The RWST - Low Low Allowable Value/Trip Setpoint has both upper and lower limits - The lower limit is selected to ensure switchover occurs before the RWST empties. to prevent ECCS pump damage. The upper limit is selected to ensure enough borated water is injected to ensure the reactor remains shut down. The high limit also ensures adequate water inventory in the containment sump to provide ECCS pump suction.
	The transmitters are located in an area not affected by HELBs or post accident high radiation. Thus, they will not experience any adverse environmental conditions and the Trip Setpoint reflects only steady state instrument uncertainties.
. \	Automatic-switchover occurs-only-if-the-RWST-low-low level-signal-is-coincident-with-SIThis-prevents

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ESFAS Instrumentation B 3.3.2

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APPLICABLE SAFETY ANALYSES. LCO, and APPLICABILITY	b. c.	Automatic_Switchover_to_Containment SumpRefueling_Water_Storage_Tank_(RWST) Level_Low_Low_Coincident_With_Safety_Injection and_Coincident_With_Containment_Sump_LevelHigh (continued)
		operationAccidental_switchover_could_damage_EGGS pumps_if_they_are_attempting_to_take_suction_from_an empty_sumpThe_automatic_switchover_Function requirements_for_the_SI_Functions_are_the_same_as_the requirements_for_their_SI_functionThereforethe requirements_are_not_repeated_in_Table_3.3.2_1 InsteadFunction_1SIis_referenced_for_all initiating_Functions_and_requirements.
		Reviewer's-Note: In-some units. additional protection from spurious switchover-is provided by requiring a Containment-Sump Level - High signal as well as RWST Level - Low Low and SI. This ensures sufficient water is available in-containment to support the recirculation phase of the accident. A Containment Sump Level - High signal must be present. in addition to the SI signal and the RWST Level - Low Low signal, to transfer the suctions of the RHR pumps to the containment sump. The containment sump is equipped with four level transmitters. These transmitters provide no control functions. Therefore, a two-out of four logic is adequate to initiate the protection function actuation. Although only three channels would be sufficient, a fourth channel has been added for increased reliability. The containment sump level Trip Setpoint/Allowable Value is selected to ensure enough borated water is injected to ensure the reactor remains shut down. The high limit also ensures adequate water inventory in the containment sump to provide containment and thus possibly experience adverse environmental conditions. Therefore, the trip setpoint reflects the inclusion of both steady state and environmental instrument uncertainties.
		Units-only-have-one-of-the-Functions-7.b-or-7.c.

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APPLICABLE SAFETY ANALYSES. LCO, and APPLICABILITY

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information

b. c. Automatic Switchover-to-Containment Refueling Water Storage Tank (RWST) Sump. Level-Low Low Coincident With Safety and Coincident With Containment Sump Leve (continued)

These This Functions must be OPERABLE in MODES 1. 2. 3. and 4 when there is a potential for a LOCA to occur, to ensure a continued supply of water for the ECCS pumps. These This Functions are is not required to be OPERABLE in MODES 5 and 6 because there is adequate time for the operator to evaluate unit conditions and respond by manually starting systems. pumps, and other equipment to mitigate the consequences of an abnormal condition or accident. System pressure and temperature are very low and many ESF components are administratively locked out or otherwise prevented from actuating to prevent inadvertent overpressurization of unit systems.

8. Engineered Safety Feature Actuation System Interlocks

To allow some flexibility in unit operations, several interlocks are included as part of the ESFAS. These interlocks permit the operator to block some signals. automatically enable other signals, prevent some actions from occurring, and cause other actions to occur. The interlock Functions back up manual actions to ensure bypassable functions are in operation under the conditions assumed in the safety analyses.

Engineered Safety Feature Actuation System а. Interlocks-Reactor Trip, P-4

> The P-4 interlock is enabled when a reactor trip breaker (RTB) and its associated bypass breaker is open. Once-the-P-4-interlock-is-enabled, automatic-SI initiation-is-blocked-after-a-[-]-second-time-delay. This Function allows operators to take-manual-control manually block reactuation of SI systems after the initial phase of injection is complete. Once SI is blocked, automatic actuation of SI cannot occur until the RTBs have been manually closed. The functions of the P-4 interlock are:

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ESFAS Instrumentation BLE BASES I.1 and I.2 ACTIONS (continued) partial trip condition where one additional tripped channel will result in actuation. The 6 hour Completion Time is justified in Ref. 8. Failure to restore the inoperable channel it DPERABLE status or place it in the tripped condition withing 6 hours: requires the Unit to be placed in MODE 2 with in the following 6 hours. The allowed Completion time of 6 hours is reasonable; based on operating experience, to reach MODE 2 from full power conditions in an orderly manner without challenging unit systems. In MODE 2. this Function is no longer required OPERABLE. The Required Actions are modified by a Note that allows the inoperable channel to be bypassed for up to (A4) hours for surveillance testing of other channels. The 6 hours allowed to place the inoperable channel in the tripped condition, and the 4 hours allowed for a second channel to be in the bypassed condition for testing, are justified in Reference 8. J.1 and J.2 0 3.3-127 INSELT ACTION J BABRES NOT USED Condition-J-applies-to-the-AFW-pump-start-on-trip-of-all-MFW pumps. This action addresses-the-train-orientation-of-the-SSPS-for-the auto-start-function-of-the-AFW-System-on-loss-of-all-MFW-pumps-The-OPERABILITY-of-the-AFW-System-must-be-assured-by-allowing automatic-start of the AFW System pumps .-- If a channel is inoperable, 48-hours-are-allowed-to-return-it-to-an-OPERABLE status .- If the function cannot be returned to an OPERABLE status.-6-hours-are allowed-to-place-the-unit-in-MODE-3.--The allowed-Completion-Time-of-6-hours-is-reasonable.-based-on operating-experience.-to-reach-MODE-3-from-full-power-conditions in-an-orderly-manner-and-without-challenging-unit-systems--_In MODE-3,-the-unit-does-not-have-any-analyzed-transients-or conditions-that-require-the-explicit-use-of-the-protection function-noted-above. -- The-allowance of 48-hours to return the train-to-an-OPERABLE-status-is-justified-in-Reference-8-K.I.I , K.I.2 KT. K.2.1 and K.2.2 - INSERT K, Condition K applies to + RWST-Level---Low-Low-Conincident-with-Safety-Injection--and RWST-Level—Low-Low-Coincident-with-Safety-Injection-and Coincident-with-Containment-Sump-Level-High-(continued)

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В INSERT К the Zendual Heat Romoval Pump Trip on K.1.1, K.1.2 , cut-out K.2.1 and K.2.2 Condition K applies to RWST Level / Low, which trips both RHR. pumps. Restoring the channel to OPERABLE status or placing the inoperable channel in the bypass condition within 6 hours is 003.3-Ed sufficient to ensure that the Function remains OPERABLE, and minimizes the time that the Function may be in a partial trip condition (assuming the inoperable channel has failed low). Placing the out-of-service channel in bypass will-generate a high level signal on that channel, which will ensure that under no cut-out removes errcumstances_can_a_failure_of_an_additional_channel-low-prevent the RHR-pumps-from-starting-as-the result-of-an-Si-signal. The that channel from cut-out 6 hour Completion Time is justified in Reference 8. If the con-channel cannot be placed in the bypass condition within 6 hours. 48 the trip loaic, Similar to à and returned to an OPERABLE status within F2 hours, the unit must bupass function. inmediczcy enter LCO 3.3.2 The 72 Nour Allowed Outage Times This provides a This-comparison-is-reasonable-because-the-possible-consequences two-out-of-two of-losing-a-second-level-channel-cangin-the-worst-sacar-be-notrip logic. more severe than the loss of one Riff pump, and the probability of tosing the level channer is even lower than that of losing an KMR the allowed Completion Times for shutdown are reasonable (223-29 Secondry. based on operating experience. to reach the required unit conditions from full power conditions in an orderly manner and without challenging unit systems. In MODE 5. the unit does not have any analyzed transients or conditions that require the explicit use of the pump trip function noted above be brought to HODE 3 withing the following thours' and MODE 5 within the next 20 hours 54 84 DC 3.3-Ed

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ESFAS Instrumentation. B 3.3.2

ACTIONS -	K.1, K.2.1 and K/2.2 (continued)
ACTIONS	<u>K.I. K.2.1 and K/2.2</u> (continued) RWST Level Low Which trips both RHR pumps Low Low Coincident With SI and Coincident With Containment Sump Level High provides actuation of switchover to the containment sump. Note that this Function requires the bistables to energize to perform their required action. The failure of up to two channels will not prevent the operation of this function. However, placing a failed channel in the tripped condition could result in a premature switchover to the sump, prior to the injection of the minimum volume from the DWST. Placing the inoperable channel in bypass results in a two out of three logic configuration, which satisfies the requirement to allow another failure without disabling actuation of the switchover when required. Restoring the channel to OPERABLE status or placing the inoperable channel in the bypass condition within 6 hours is sufficient to epsure that the Function remains OPERABLE, and minimizes the time that the function may be in a partial trip condition (assuming the noperable channel has farled high low. Placing the bur constants can failure of an additional channel low prevent the BMR pumps from starting as the result of an SI signal. The 6 hour Completion Time is justified in Reference 8. If the channel cannot be returned to OPERABLE status or placed in the bypass condition within 6 hours, and returned to an OPERABLE status within 72 hours, the unit must be brought to ANDE 3 Athin the following chours and MODE 5 within the next 30 hours instanting a second is allowed for one inoperable RHR pump. This comparison is reasonable because the possible construences of losing a second level channel can. In the worst case, be no more sovere than the loss of one RHR pump. and the probability of losing a second level channel can. In the worst case, be no more sovere than the loss of one RHR pump. and the probability of losing a second level channel is set than the that of Status of losing a second level channel can. In the worst case, be n
`	allowed Completion limes for sputdown are reasonable, based on operating experience, to reach the required unit conditions from
	full power conditions in an orderly manner and without challenging unit systems. In MODE 5, the unit does not have any analyzed transients or conditions that require the explicit use of the protection pump trip functions noted above.
	The Required Actions are modified by a Note that allows placing a second channel in the bypass condition for up to 4 hours for surveillance testing. The total of 12 hours to reach MODE 3 and 4 hours for a second channel to be bypassed is acceptable based on the results of Reference 8

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SURVEILLANCE REQUIREMENTS	<u>SR_3.3.2.11</u> (continued) 24 Trip (Interlock, and the Frequency The 28 month Frequency is based or demonstrating that undetected fai sometimes occurs when the RTB is (is-once-per-RTB-cycle. This n operating experience lure-of-the-P-4-interlock cycled.
Insert	The SR is modified by a Note that setpoints during the TADOT. The F associated setpoint.	excludes verification of Function tested has no
REFERENCES	1. FSAR, Chapter 5. 2. FSAR, Chapter 7.	
A	• 3. FSAR, Chapter 15.	•
•	5. 10 CFR 50.49.	DC ALL-002
	Westinghoue Setpoint Method Diablo Canyon Stations - Eag	ology for Protection Systems Te 21 Version, May 1993
,e 	Relay Surveillance Test inte 8. WCAP-10271-P-A. Supplement 2	rvals April 1994
	9. Technical Requirements Mapua Times. "None. NCAP-13878 () MDR Relays June 1994.	1. Section 15. "Response 23.3-Ed Hiability of Potter & Brumfield
	10. WCAP-14117 Reliability Ass MDR Series Relays	essment of Potter and Brumfield
~,(WCAP-13632-P-A, Revision 2, "Elimination Requirements," January 1996. 12. WCAP-11082, Rev. 5, "We	n of Pressure Sensor Response Time Testi Stinghouse Sensoint Methodolog
,	for Protection System:, 2 24 Month Fuel Curc 2 &	Diablo Canyon Unit: Total, Evaluation, " January 1997 De All-0
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SR 3.3.2.12

SR 3.3.2.12 is the performance of an ACTUATION LOGIC TEST as described in TS 1.0 "Definitions." This SR is applied to the RHR Pump Trip on RWST Level-Low actuation logic and relays which are not processed through the SSPS. This test is performed every refueling outage. The frequency is adequate based on site and industry operating experience, considering equipment reliability and history data.

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SR 3.3.2.13

SR 3.3.2.8 is the performance of a TADOT. This test is a check of the Manual Actuation Functions and AFW pump start on trip of all MFW pumps. It is performed every 9[18] months. Each Manual Actuation Function is tested up to, and including, the master relay coils. In some instances, the test includes actuation of the end device (i.e., pump starts, valve cycles, etc.). The Frequency is adequate, based on industry operating experience and is consistent with the typical refueling cycle. The SR is modified by a Note that excludes verification of setpoints during the TADOT for manual initiation Functions. The manual initiation Functions have no associated setpoints.

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