

December 29, 1998

Mr. Gregory M. Rueger
Senior Vice President and General Manager
Pacific Gas and Electric Company
Diablo Canyon Nuclear Power Plant
P. O. Box 3
Avila Beach, California 93424

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION REGARDING
GENERIC LETTER 96-05 PROGRAM AT DIABLO CANYON
(TAC NOS. M97040 AND M97041)

Dear Mr. Rueger:

On September 18, 1996, the NRC issued Generic Letter (GL) 96-05, "Periodic Verification of Design-Basis Capability of Safety-Related Motor-Operated Valves," to request that nuclear power plant licensees establish a program, or ensure the effectiveness of the current program, to verify on a periodic basis that safety-related motor-operated valves (MOVs) continue to be capable of performing their safety functions within the current licensing basis of the facility.

On September 8, 1998, the licensee of the Diablo Canyon Nuclear Power Plant, Units 1 and 2, submitted an updated response to GL 96-05 indicating its intent to implement the provisions of a Joint Owners Group (JOG) Program on MOV Periodic Verification. The NRC staff has encouraged licensees to participate in the industry-wide JOG program to provide a benefit in reactor safety by sharing expertise and information on MOV performance and to increase the efficiency of GL 96-05 activities at nuclear plants. Licensee participation in the JOG program also minimizes the amount of information necessary for the NRC staff to review for each licensee's response to GL 96-05. As a result, the NRC staff requires only limited information to complete its GL 96-05 review for Diablo Canyon.

The enclosed request was discussed with Mr. Pat Nugent of your staff on December 22, 1998. A mutually agreeable target date of February 28, 1999, for your response was established. If circumstances result in the need to revise the target date, please call me at the earliest opportunity.

Sincerely,
Original Signed By
Steven D. Bloom, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-275
and 50-323

Enclosure: Request for Additional Information

cc w/encl: See next page

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The following table shows the results of the tests conducted on the samples of the material under investigation. The tests were carried out in accordance with the standard methods described in the literature.

Sample No.	Material	Test Method	Result
1	Aluminum	Tensile Test	Yield Strength: 100 MPa
2	Steel	Impact Test	Charpy Energy: 15 J
3	Copper	Hardness Test	Brinell Hardness: 100 HB
4	Iron	Corrosion Test	Corrosion Rate: 0.5 mm/year
5	Carbon Steel	Weldability Test	Weldability: Good
6	Aluminum	Thermal Expansion Test	Thermal Expansion Coefficient: 23.6 x 10^-6 /°C
7	Steel	Fatigue Test	Fatigue Life: 10^7 cycles
8	Copper	Electrical Conductivity Test	Electrical Conductivity: 58 MS/m
9	Iron	Magnetic Property Test	Magnetic Permeability: 1000
10	Carbon Steel	Heat Treatment Test	Heat Treatment: Annealing

Mr. Gregory M. Rueger

- 2 -

December 29, 1998

cc w/encl:
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181

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REQUEST FOR ADDITIONAL INFORMATION

RESPONSE TO GENERIC LETTER 96-05, "PERIODIC VERIFICATION OF DESIGN-BASIS

CAPABILITY OF SAFETY-RELATED MOTOR-OPERATED VALVES"

PACIFIC GAS & ELECTRIC COMPANY

DIABLO CANYON NUCLEAR POWER PLANT, UNITS 1 AND 2

DOCKET NOS. 50-275 AND 50-323

1. In NRC Inspection Report No. 50-275 and 323/95-01 dated March 2, 1995, and NRC letter dated August 3, 1995, the NRC staff closed its review of the motor-operated valve (MOV) program implemented at the Diablo Canyon Nuclear Power Plant in response to Generic Letter (GL) 89-10, "Safety-Related Motor-Operated Valve Testing and Surveillance." In the inspection report, the NRC staff requested that additional information be provided to clarify selected program commitments. The licensee replied to the inspection report and addressed its long-term plans in its letter dated May 5, 1995. For example, the licensee noted that (1) the feedwater isolation valves and the pressurizer power-operated relief valve (PORV) block valves would be qualified using the Electric Power Research Institute (EPRI) MOV Performance Prediction Program (PPM); (2) valves that use the PPM would be modified, if necessary, to achieve a conclusive prediction; (3) a review of valve grouping in accordance with guidance in GL 89-10, Supplement 6, would be completed by June 30, 1995; (4) differential pressure testing would be evaluated to determine the effects of assembling valves using Neolube; and (5) all appropriate documents would be modified by June 30, 1995, to include a margin for valve degradation. The licensee should discuss the completion of these GL 89-10 close-out actions.
2. The JOG program focuses on the potential age-related increase in the thrust or torque required to operate valves under their design-basis conditions. In the NRC safety evaluation dated October 30, 1997, on the JOG program, the NRC staff specified that licensees are responsible for addressing the thrust or torque delivered by the MOV motor actuator and its potential degradation. The licensee should describe the plan at Diablo Canyon for ensuring adequate ac and dc MOV motor actuator output capability, including consideration of recent guidance in Limitorque Technical Update 98-01 and its Supplement 1.



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