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Pacific Gas and Electric Company

Diablo Canyon Power Plant
P.O. Box 56
Avila Beach, CA 93424
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John D. Townsend
Vice President–Diablo Canyon
Operations and Plant Manager

July 15, 1992

U.S. NUCLEAR REGULATORY COMMISSION
Attention: Document Control Desk
Washington, DC 20555

RE: Docket No. 50-275 and 50-323
License No. DPR-80 and DPR-82
Monthly Operating Report for June 1992

GENTLEMEN:

Enclosed are the monthly operating reports for Diablo Canyon Units 1 and 2 for June 1992. This report is submitted in accordance with Section 6.9.1.7 of the Units 1 and 2 Technical Specifications.

Sincerely,

Don Moulin for

JDT:pgd

Enclosures

cc Mr. John B. Martin, Regional Administrator
U.S. Nuclear Regulatory Commission
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1450 Maria Lane, Suite 210
Walnut Creek, CA 94596-5268

Ms. Liz Hannon, President
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1700 K Street, NW, Suite 400
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bcc

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MONTHLY NARRATIVE REPORT
OF OPERATION
AND MAJOR MAINTENANCE EXPERIENCE

This report describes the operating and major maintenance experience for the month of June 1992. This narrative report was prepared by the Plant Staff and is submitted in accordance with Section 6.9.1.7 of the Units 1 and 2 Technical Specifications.

Narrative of Daily Significant Plant Events

On June 1, 1992: Unit 1 and Unit 2 started the month at 100% power.

On June 4, 1992: A 10 CFR 50.72(b)(2)(iii)(D) non-emergency four-hour report was made for Unit 2 regarding auxiliary building solenoid operated ventilation dampers and fan motor control circuits equipped with surge suppression diodes that were not qualified to a total integrated dose expected from a large-break loss-of-coolant accident. It was determined that the diodes could be removed without affecting the reliable operation of the dampers. For more information, see LER 2-92-005-00

On June 8, 1992: A 10 CFR 50.72(b)(1)(ii)(B) non-emergency one-hour report was made for Units 1 and 2 subsequent to a review of 10 CFR 50 appendix R requirements which identified inadequate separation of steam generator and reactor coolant system circuits in containment. For more information, see LER 2-92-001-01

On June 13, 1992 Unit 2 ramped down to 52% power for condenser cleaning.

On June 15, 1992 Unit 2 returned to 100% power.

On June 19, 1992 A 10 CFR 50.72(b)(1)(ii)(B) non-emergency one-hour report was made for Units 1 and 2 subsequent to a review of 10 CFR 50 appendix R requirements which identified inadequate isolation of alternate shutdown capability circuits in the control and cable spreading rooms. For more information, see LER 2-92-001-01.

On June 20, 1992 An Unusual Event was declared for Unit 2. An acid/caustic spill caused a chemical mist to enter portions of the turbine building. The event was reported to the NRC in accordance with 10 CFR 50.72 (a)(1)(i). The Unusual Event was terminated after restricted access was restored to the turbine

building. For more information, see LER 1-92-007-00.

On June 25, 1992 — A 10 CFR 50.72(b)(1)(ii)(B) non-emergency one-hour report was made for Units 1 and 2 subsequent to a review of 10 CFR 50 appendix R requirements which identified inadequate isolation of diesel-generator control circuits in the control and cable spreading rooms. For more information, see LER 2-92-001-01.

On June 26, 1992 — A 10 CFR 50.72(b)(1)(ii)(B) non-emergency one-hour report was made due to a Unit 1 Chemical and Volume Control System valve leaking above acceptable limits. For more information, see LER 1-92-009-00.

On June 28, 1992 — An Unusual Event was declared for Units 1 and 2 when a moderate seismic event was felt in the control room. The event was reported to the NRC in accordance with 10 CFR 50.72(a)(1)(i). The Unusual Event was terminated after inspection of both Units showed no abnormal conditions.

On June 30, 1992 — Unit 1 and Unit 2 ended the month at 100% power.

Summary of Plant Operating Characteristics, Power Reductions and Unit Shutdowns

Unit 1 operated this month with a unit availability factor of 100.0% and a unit capacity factor (using MDC Net) of 100.0%. Unit 1 did not reduce power by more than 20% this month.

Unit 2 operated this month with a unit availability factor of 100.0% and a unit capacity factor (using MDC Net) of 98.6%. Unit 2 reduced power once this month due to condenser cleaning.

Summary of Significant Safety Related Maintenance

A leak in a valve in the Unit 1 Chemical and Volume Control System was terminated by re-torquing the body to bonnet bolts. In addition, to prevent further leakage, a leak repair enclosure was installed on the valve.

A leak in the Unit 2 annubar pipe on Auxiliary Salt Water train 2-2 was temporarily patched. An investigation determined the cause of the leak was exterior corrosion of the piping. Plans are in progress to replace this pipe on both trains for both units and to inspect and evaluate other possible areas of corrosion.

The Centrifugal Charging Pump 2-1 motor coupling was replaced due to significant wear.

A motor operated valve (SI-2-8923A) would not fully open due to a worm cartridge bearing locknut setscrew being loose. The valve was inspected and repaired. PG&E is currently investigating the generic implications of this issue.

Actuations of Steam Generator Safety or Pressurizer Power Operated Relief Valves

There were no challenges to the steam generator safety valves or the pressurizer power operated relief valves.

OPERATING DATA REPORT

DOCKET NO.	50-275
UNIT	1
DATE	07/01/92
COMPLETED BY	P. DAHAN
TELEPHONE	(805) 545-4054

OPERATING STATUS

1.	Unit Name:	Diablo Canyon Unit 1
2.	Reporting Period:	June 1992
3.	Licensed Thermal Power (MWt):	3338
4.	Nameplate Rating (Gross MWe):	1137
5.	Design Electrical Rating (Net Mwe):	1086
6.	Maximum Dependable Capacity (Gross Mwe):	1124
7.	Maximum Dependable Capacity (Net MWe):	1073.4
8.	If changes occur in capacity ratings (items 3 through 7) since last report, give reasons:	N/A
9.	Power level to which restricted, if any (Net MWe):	N/A
10.	Reasons for restrictions, if any:	N/A

		This Month	YTD	Cumulative
11.	Hours In Reporting Period	720.0	4367.0	62685.3
12.	Number Of Hours Reactor Was Critical	720.0	4256.7	52571.3
13.	Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14.	Hours Generator On-Line	720.0	4241.7	51727.4
15.	Unit Reserve Shutdown Hours	0.0	0.0	0.0
16.	Gross Thermal Energy Generated (MWH)	2400268	13976716	162325162
17.	Gross Electrical Energy Generated (MWH)	810100	4687300	54629532
18.	Net Electrical Energy Generated (MWH)	772501	4461469	51799629
19.	Unit Service Factor	100.0	97.1	82.5
20.	Unit Availability Factor	100.0	97.1	82.5
21.	Unit Capacity Factor (Using MDC Net)	100.0	95.2	77.0
22.	Unit Capacity Factor (Using DER Net)	98.8	94.1	76.1
23.	Unit Forced Outage Rate	0.0	2.9	3.7

24. Shutdowns Scheduled Over Next 6 Months
(Type, Date, and Duration of Each): Refueling Outage - September 13, 1992 - 63 days

25. If Shut Down At End Of Report Period, Estimate Date of Startup: N/A

OPERATING DATA REPORT

DOCKET NO. 50-323
UNIT 2
DATE 07/01/92
COMPLETED BY P. DAHAN
TELEPHONE (805) 545-4054

OPERATING STATUS

1. Unit Name: Diablo Canyon Unit 2
2. Reporting Period: JUNE 1992
3. Licensed Thermal Power (MWt): 3411
4. Nameplate Rating (Gross MWe): 1164
5. Design Electrical Rating (Net MWe): 1119
6. Maximum Dependable Capacity (Gross MWe): 1137
7. Maximum Dependable Capacity (Net MWe): 1087
8. If changes occur in capacity ratings (items 3 through 7) since last report, give reasons: N/A
9. Power level to which restricted, if any (Net MWe): N/A
10. Reasons for restrictions, if any: N/A

	This Month	YTD	Cumulative
11. Hours In Reporting Period	720.0	4367.0	55244.0
12. Number Of Hours Reactor Was Critical	720.0	4255.9	46418.0
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	720.0	4234.8	45588.3
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	2416838	14262019	148492999
17. Gross Electrical Energy Generated (MWH)	807400	4750200	49413799
18. Net Electrical Energy Generated (MWH)	771772	4535406	46921275
19. Unit Service Factor	100.0	97.0	82.5
20. Unit Availability Factor	100.0	97.0	82.5
21. Unit Capacity Factor (Using MDC Net)	98.6	95.5	78.3
22. Unit Capacity Factor (Using DER Net)	95.8	92.8	75.9
23. Unit Forced Outage Rate	0.0	3.0	4.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): None
25. If Shut Down At End Of Report Period, Estimate Date of Startup: N/A

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-275
UNIT	1
DATE	07/01/92
COMPLETED BY	P. DAHAN
TELEPHONE	(805) 545-4054

JUNE 1992	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
	1	1068
	2	1069
	3	1069
	4	1069
	5	1069
	6	1073
	7	1068
	8	1073
	9	1068
	10	1073
	11	1073
	12	1072
	13	1068
	14	1078
	15	1072
	16	1073
	17	1073
	18	1077
	19	1073
	20	1073
	21	1078
	22	1072
	23	1077
	24	1072
	25	1077
	26	1077
	27	1073
	28	1077
	29	1077
	30	1077

The average monthly Electrical Power Level for June 1992 = 1073 MWe-Net

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-323
UNIT	2
DATE	07/01/92
COMPLETED BY	P.G. DAHAN
TELEPHONE	(805) 545-4054

JUNE 1992 DAY	AVERAGE DAILY POWER LE (MWe-Net)
1	1084
2	1088
3	1084
4	1084
5	1084
6	1084
7	1084
8	1084
9	1084
10	1084
11	1084
12	1084
13	1042
14	584
15	1087
16	1096
17	1096
18	1096
19	1096
20	1096
21	1096
22	1097
23	1092
24	1096
25	1096
26	1096
27	1097
28	1092
29	1097
30	1096

The Average Monthly Electrical Power Level for JUNE 1992 = 1072 MWe-Net

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-275
UNIT 1
DATE 07/01/92
COMPLETED BY P.G. DAHAN
TELEPHONE (805) 545-4054

REPORT MONTH: JUNE 1992

NO.	DATE	TYPE[1]	DURATION (HOURS)	REASON[2]	METHOD OF SHUTDOWN[3]	LICENSEE EVENT REPORT	SYSTEM CODE[4]	COMPONENT CODE[5]	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
None									
None									

1
Type:
F-Forced
S-Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F- Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation from previous month
5-Power reduction
6-Other

4
EIIS Systems List, Table 1

5
IEEE Std. 803A-1983, "IEEE Recommended Practice for Unique Identification in Power Plants and Related Facilities - Table 2"

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-323
UNIT 2
DATE 07/01/92
COMPLETED BY P. G. DAHAN
TELEPHONE (805) 545-4054

REPORT MONTH: JUNE 1992

NO.	DATE	TYPE[1]	DURATION (HOURS)	REASON[2]	METHOD OF SHUTDOWN[3]	LICENSEE EVENT REPORT	SYSTEM CODE[4]	COMPONENT CODE[5]	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
1	920613	S	0	B	5	NONE	SD	COND	Unit 2 ramped down to 52% power for condenser cleaning.
None									

1
Type:
F-Forced
S-Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F- Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation from previous month
5-Power reduction
6-Other

4
EIIS Systems List, Table 1

5
IEEE Std. 803A-1983, "IEEE Recommended Practice for Unique Identification in Power Plants and Related Facilities - Table 2"

REFUELING INFORMATION REQUEST

DOCKET NO.	50-275
UNIT	1
DATE	07/01/92
COMPLETED BY	M. L. Mayer
TELEPHONE	(805) 545-4674

1. Name of facility: Diablo Canyon Unit 1

2. Scheduled date for next refueling shutdown: September 13, 1992 (estimated)

3. Scheduled date for restart following refueling: November 1992 (estimated)

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)? If no such review has taken place, when is it scheduled?

   No. The PSRC is scheduled to review the cycle 6 core reload in September 1992 (estimated).

5. Scheduled date(s) for submitting proposed licensing action and supporting information: N/A

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures: N/A

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool:

   (a) 193 (b) 320

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

   Present 1324 Increase size by 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2006 (Loss of full core offload capability)

REFUELING INFORMATION REQUEST

DOCKET NO.	50-323
UNIT	2
DATE	07/01/92
COMPLETED BY	M. L. Mayer
TELEPHONE	(805) 545-4674

1. Name of facility: Diablo Canyon Unit 2

2. Scheduled date for next refueling shutdown: March 1, 1993 (estimated)

3. Scheduled date for restart following refueling: May 1993 (estimated)

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)? If no such review has taken place, when is it scheduled?

   No. The PSRC is scheduled to review the cycle 6 core reload in February 1993 (estimated).

5. Scheduled date(s) for submitting proposed licensing action and supporting information: NA

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures: NA

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool:

   (a) 193 (b) 308

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

   Present 1324 Increase size by 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2006 (Loss of full core offload capability)